UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

January 6, 2015

Mr. Michael J. Pacilio
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION - REQUEST FOR ADDITIONAL INFORMATION REGARDING THE FOURTH 10-YEAR INTERVAL INSERVICE INSPECTION PROGRAM PLAN REQUESTS FOR RELIEF (TAC NOS. MF3406 AND MF3407)

Dear Mr. Pacilio:

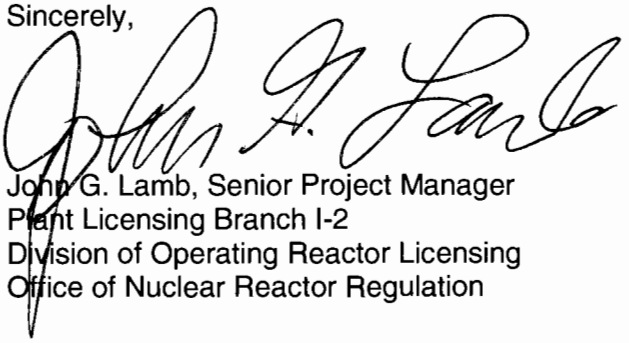
By letter dated January 7, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14028A579), as supplemented by letter dated November 14, 2014 (ADAMS Accession No. ML14321A044), Exelon Generation company, LLC (Exelon or the licensee) submitted Requests for Relief R-44 AND R-45 from the requirements of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI for Oyster Creek Nuclear Generating Station (Oyster Creek). The requests for relief apply to the fourth 10-year inservice inspection interval, in which the licensee adopted the 1995 Edition through the 1996 Addenda of ASME Code Section XI as the code of record.

The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the Relief Requests R-44 and R-45, and has determined that a request for additional information (RAI) is needed to complete its technical review. The NRC staff's RAI is contained in the enclosure. A draft of these questions was previously sent to Mr. Dave Helker of your staff on August 28, 2014, with an opportunity to have a teleconference to ensure that the licensee understood the questions and their regulatory basis, as well as to verify that the information was not previously docketed.

A conference call was held on December 23, 2014, and Mr. Tom Loomis of your staff agreed that Exelon would respond to the RAI in 30 days from the date of the letter. Please note that if you do not respond to the RAI within 30 days from the date of the letter, or provide an acceptable alternate date in writing, the NRC staff may reject your LAR under the provisions of Title 10 of the *Code of Federal Regulations*, Section 2.108, "Denial of application for failure to supply information."

If you have any questions regarding this letter, please feel free to contact me at (301) 415-3100.

Sincerely,



John G. Lamb, Senior Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosure:
Request for Additional Information

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION

RELATED TO RELIEF REQUESTS R-44 AND R-45

OYSTER CREEK NUCLEAR GENERATING STATION

EXELON GENERATING COMPANY, LLC

DOCKET NO. 50-219

By letter dated January 7, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14028A579), as supplemented by letter dated November 14, 2014 (ADAMS Accession No. ML14321A044), Exelon Generation Company, LLC (Exelon or the licensee) submitted Requests for Relief R-44 AND R-45 from the requirements of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI for Oyster Creek Nuclear Generating Station (Oyster Creek). The requests for relief apply to the fourth 10-year inservice inspection (ISI) interval, in which the licensee adopted the 1995 Edition through the 1996 Addenda of ASME Code Section XI as the code of record.

The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the Relief Requests R-44 and R-45, and has determined that a request for additional information (RAI) is needed to complete its technical review. The NRC staff's RAIs are below.

1.0 REQUEST FOR ADDITIONAL INFORMATION (RAI)

1) The examination limitations for ASME Code, Section XI, Examination Category B-F, Item B5.11, weld NR02 4-565D, N1D Recirc Outlet Nozzle-to-Safe End, is discussed in the licensee's RAI response dated November 14, 2014. Attachment A.40 to the licensee's submittal dated January 7, 2014, contains a sketch of this weld. However, this sketch appears to be a drawing of a cladded nozzle "set-in" design weld rather than a nozzle-to-safe end weld. Please clarify this sketch or provide another sketch clearly indicating the Item B5.11 nozzle and the safe end components. Please include cross-sectional plots showing volumetric coverage obtained with specific volumetric coverages near the inner surface, using appropriate ultrasonic wave angles and modalities. These are typically the most highly stressed regions, exposed to operating environments where service degradation would be expected to occur, should it be manifested.

   The photos provided in the original submittal in Attachments A30 and A40 for the Item B5.11 welds are indecipherable. Please provide replacement images that are clear or alternative information to clarify the limitations these photos were intended to represent.

2) For R-45, Examination Category B-J, Item B9.11 welds NE 5-0002, NE 5-0022A, and NG-5-D-0002, shown in Figures 2, 3, and 4, respectively, please provide cross-sectional plots showing volumetric coverage obtained with specific volumetric coverages near the inner surface, using appropriate ultrasonic wave angles and modalities.

If you have any questions regarding this letter, please feel free to contact me at (301) 415-3100.

Sincerely,

/RA/

John G. Lamb, Senior Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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ADAMS Accession No.: ML15005A069 *via memo

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NAME	JLamb	ABaxter	SRosenberg*	MKhanna	JLamb
DATE	12/19/2014	12/22/2014	12/16/2014	1/5/2015	1/6/2015

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