

November 13, 2014

MEMORANDUM TO: Nathan Sanfilippo, Chief
Performance Assessment Branch
Division of Inspection and Regional Support
Office of Nuclear Reactor Regulation

FROM: Andrew Waugh, Reactor Operations Engineer /RA/
Performance Assessment Branch
Division of Inspection and Regional Support
Office of Nuclear Reactor Regulation

SUBJECT: SUMMARY OF THE REACTOR OVERSIGHT PROCESS WORKING
GROUP PUBLIC MEETING HELD ON OCTOBER 22, 2014

On October 22, 2014, the U.S. Nuclear Regulatory Commission (NRC) staff hosted the Reactor Oversight Process (ROP) Working Group (WG) public meeting with the Nuclear Energy Institute (NEI) ROP Task Force and other industry representatives. Enclosure 1 contains the meeting attendance list; Enclosure 2 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14314A512) contains the Frequently Asked Questions (FAQs) Log and the FAQs discussed during the meeting. Meeting attendees discussed topics related to ROP FAQs.

Staff and industry discussed the following FAQs (see Enclosure 2):

- FAQ 14-02: This FAQ was discussed. This is a site-specific FAQ developed by Fort Calhoun Station (FCS) to address the validity of Mitigating System Performance Index (MSPI) performance indicators as a result of an extended shutdown. FCS predicted future MSPI values using a Pressurized Water Reactor Owners Group "What-if" tool and expected plant data. Based on the results of this analysis, FCS proposes that the high pressure injection system MSPI, heat removal system MSPI, and cooling water system MSPI become valid on the 4th quarter of 2014. However, FCS proposes that the Emergency AC power and residual heat removal system components of the MSPI become valid on the 1st quarter of 2015 and 1st quarter of 2016, respectively. FCS provided additional information on their analysis of the heat removal system MSPI validity. The staff will review this information and provide a response during the next ROP WG public meeting.
- FAQ 14-03: This FAQ is tentative final. This is a site-specific FAQ for Arkansas Nuclear One (ANO), Unit 2. This FAQ addresses the Unplanned Scrams with Complications performance indicator guidance on the flowchart question regarding the status of main feedwater availability after the scram.

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- Specifically, ANO proposes that because of the unique design of their feedwater system, the intention of the flowchart question regarding the availability of feedwater is met and that the scram should not be considered complicated.
- ANO provided additional information about the procedures to recover main feedwater with loss of condenser vacuum. The NRC staff acknowledged that main feedwater could have been recovered within 30 minutes and concluded that this event should not count under the Unplanned Scrams with Complications PI. The staff's response to this FAQ will be revised and this FAQ is expected to become final in the next ROP WG public meeting
- FAQ 14-05: This FAQ is final. This is a site-specific FAQ for Hatch. This FAQ addresses the validity of the Alert and Notification System PI for Hatch. The licensee proposes to report the results in accordance with NEI 99-02 and enter zeroes for trailing quarters until four quarters of data have accumulated. This FAQ will become effective in the reporting of 1Q2015 PI data.
- FAQ 14-06: This FAQ was discussed. This is a site-specific FAQ for Vermont Yankee. This FAQ addresses the Unplanned Power Changes per 7,000 Critical Hours performance indicator guidance on identification and response to degrading conditions. The staff noted that this FAQ might not be resolved prior to the Vermont Yankee's expected transition to decommissioning. However, the staff recognized that this FAQ provides an opportunity to explore clarifying the terms "sudden degradation" and "rapid response" in NEI 99-02, "Regulatory Assessment Performance Indicator Guidance." The staff will review this FAQ and the need for revising NEI 99-02 and will provide a response during the next ROP WG public meeting.
- FAQ 14-07: This FAQ was introduced. This is a site-specific FAQ for Point Beach Nuclear Plant (PBNP). This FAQ discusses the transition of siren responsibility from Kewaunee to PBNP for those sirens in the area of overlap of Emergency Planning Zones (EPZ) between both sites. This FAQ proposes a revision to NEI 99-02 Appendix D, "Plant-Specific Design Issues," to account for the transition in responsibility for the sirens in PBNP's EPZ.

The next ROP WG public meeting is scheduled to be held on November 19, 2014.

Enclosures:

1. Attendance List – October 22, 2014
2. ROP Task Force FAQ Log – October 22, 2014

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Enclosures:

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ROP Task Force FAQ Log – October 22, 2014

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ADAMS Package No. ML14314A587

OFFICE	NRR/DIRS/IPAB	NRR/DIRS/IPAB	BC:NRR/DIRS/IPAB
NAME	AWaugh	LCruz	NSanfilippo
DATE	11/7/14	11/7/14	11/13/14

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**REACTOR OVERSIGHT PROCESS PUBLIC MEETING
ATTENDANCE LIST
October 22, 2014**

Nathan Sanfilippo	NRC
Andrew Waugh	NRC
Luis Cruz	NRC
Steve Vaughn	NRC
Chase Franklin	NRC
Mary Anderson	NRC
Eric Schrader	NRC
Jim Slider	NEI
Larry Parker	STARS Alliance
Robin Ritzman	FENOC
Adrienne Driver	Duke Energy
Bruce Mrowca	ISC
Tony Zimmerman	Duke
Ken Heffner	Certrec
Suzanne Leblang	Entergy
Steve Catron	NextEra Energy
Lenny Sueper	Xcel Energy
Elijah DeVaughn	Southern Nuclear
Carlos Cisco	Winston
Roy Linthicum	PWROG/Exelon
Ron Gaston	Exelon
Kevin Boston	OPPD
Shirelle Allen	Duke
Gary Miller	Dominion
Peggy Rescheske	TVA
Justin Wearne	PSEG
Jana Bergman	Sciencetech
Steve Huntzater	
Colley Chapel	
James Coyle	
Justin Hiller	
Enrique Melendez Asensio	