



Tennessee Valley Authority, 1101 Market Street, Chattanooga, Tennessee 37402

CNL-14-178

October 31, 2014

10 CFR 50.36(a)

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555-0001

Watts Bar Nuclear Plant, Unit 2
Construction Permit No. CPPR-92
NRC Docket No. 50-391

**Subject: Watts Bar Nuclear Plant Unit 2 – Technical Specification 5.7.2.19,
“Containment Leakage Rate Testing Program”**

Reference: TVA Letter to NRC, “Watts Bar Nuclear Plant Unit 2 – Submittal of Developmental Revision I of the Unit 2 Technical Specification & Technical Specification Bases and Developmental Revision D of the Unit 2 Technical Requirements Manual and Technical Requirements Manual Bases,” dated June 16, 2014 (ML14169A525)

This letter provides an update to the Tennessee Valley Authority (TVA) Watts Bar Nuclear Plant (WBN) Unit 2 Developmental Revision I Technical Specification (TS). The Tennessee Valley Authority proposes to revise TS 5.7.2.19, “Containment Leakage Rate Testing Program,” to clarify the maximum allowable internal containment pressure of 15.0 pounds per square inch (psig) utilized for P_a (peak calculated containment internal pressure). This revision is considered to be administrative in nature. The revision brings TS 5.7.2.19, TS Bases 3.6, “Containment Systems,” and WBN Unit 2 Final Safety Analysis Report (FSAR) Section 6.2.6, “Containment Leakage Testing” into agreement.

Enclosure 1 provides the WBN Unit 2 Developmental Revision I TS pages marked-up to show the changes for TS 5.7.2.19, “Containment Leakage Rate Testing Program.”

Enclosure 2 provides the WBN Unit 2 Developmental Revision I TS pages retyped with the changes incorporated for TS 5.7.2.19, “Containment Leakage Rate Testing Program.”

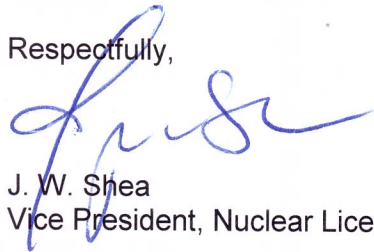
The proposed revision to the Developmental Revision I TS has been incorporated into TS 5.7.2.19, “Containment Leakage Rate Testing Program,” interim Developmental Revision J and the master word processing document, Revision 0.

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There are no new regulatory commitments associated with this submittal. If you have any questions, please contact Gordon Arent at (423) 365-2004.

I declare under penalty of perjury that the foregoing is true and correct. Executed on the 31st day of October, 2014.

Respectfully,



J. W. Shea
Vice President, Nuclear Licensing

- Enclosures:
1. Marked-Up WBN Unit 2 Developmental Revision I Pages for TS 5.7.2.19, "Containment Leakage Rate Testing Program"
 2. Re-Typed Pages for WBN Unit 2 Interim Developmental Revision J TS 5.7.2.19, "Containment Leakage Rate Testing Program"

cc (Enclosures):

NRC Regional Administrator – Region II
NRC Senior Resident Inspector – Watts Bar Nuclear Plant, Unit 2
NRC Project Manager – Watts Bar Nuclear Plant, Unit 2

ENCLOSURE 1

Marked-Up WBN Unit 2 Developmental Revision I Pages for TS 5.7.2.19, "Containment Leakage Rate Testing Program"

NOTE: Mark-up deletions identified by bold strikethrough (~~deletion~~) and additions identified by bold underline (addition).

5.7 Procedures, Programs, and Manuals

5.7.2.18 Safety Function Determination Program (SFDP) (continued)

A loss of safety function exists when, assuming no concurrent single failure, a safety function assumed in the accident analysis cannot be performed. For the purpose of this program, a loss of safety function may exist when a support system is inoperable, and:

- a. A required system redundant to the system(s) supported by the inoperable support system is also inoperable; or
- b. A required system redundant to the system(s) in turn supported by the inoperable supported system is also inoperable; or
- c. A required system redundant to the support system(s) for the supported systems (a) and (b) above is also inoperable.

The SFDP identifies where a loss of safety function exists. If a loss of safety function is determined to exist by this program, the appropriate Conditions and Required Actions of the LCO in which the loss of safety function exists are required to be entered.

5.7.2.19 Containment Leakage Rate Testing Program

A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50 Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide (RG) 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995.

~~The peak calculated containment internal pressure for the design basis loss of coolant accident, P_a is 15.0 psig.~~

For containment leakage rate testing purposes, a value of 15.0 psig, which is equivalent to the maximum allowable internal containment pressure, is utilized for P_a to bound the peak calculated containment internal pressure for the design basis loss of coolant accident.

The maximum allowable containment leakage rate, L_a , at P_a , is 0.25% of the primary containment air weight per day.

(continued)

Watts Bar-Unit 2
(developmental)

5.0-24

AJ

ENCLOSURE 2

**Re-Typed Pages for WBN Unit 2 Interim Developmental Revision J
TS 5.7.2.19, "Containment Leakage Rate Testing Program"**

5.7 Procedures, Programs, and Manuals

5.7.2.18 Safety Function Determination Program (SFDP) (continued)

A loss of safety function exists when, assuming no concurrent single failure, a safety function assumed in the accident analysis cannot be performed. For the purpose of this program, a loss of safety function may exist when a support system is inoperable, and:

- a. A required system redundant to the system(s) supported by the inoperable support system is also inoperable; or
- b. A required system redundant to the system(s) in turn supported by the inoperable supported system is also inoperable; or
- c. A required system redundant to the support system(s) for the supported systems (a) and (b) above is also inoperable.

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