



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 26, 2014

Mr. John A. Dent, Jr.
Site Vice President
Entergy Nuclear Operations, Inc.
Pilgrim Nuclear Power Station
600 Rocky Hill Road
Plymouth, MA 02360-5508

SUBJECT: PILGRIM NUCLEAR POWER STATION - REQUEST FOR ADDITIONAL
INFORMATION REGARDING THE HEAVY LOADS LICENSE AMENDMENT
REQUEST (TAC NO. MF3237)

Dear Mr. Dent:

By letter dated November 26, 2013, as supplemented by letters dated July 11, 2014, and September 11, 2014, Entergy Nuclear Operations, Inc. submitted a license amendment request to modify Technical Specification (TS) 4.3.4.a to permit heavy load handling (excess of 2,000 lbs) over irradiated fuel in a Multi-Purpose Canister using a single-failure-proof handling system for Pilgrim Nuclear Power Station. The September 11, 2014, supplement revised TS 4.3.4.b to reflect the removal of the energy absorbing pad from the spent fuel pool and installation of a leveling platform.

The Nuclear Regulatory Commission (NRC) staff is reviewing the submissions and has determined that additional information is needed to complete its review. The specific questions are found in the enclosed request for additional information (RAI). The NRC staff is requesting a response to the RAI by October 1, 2014.

If you have any questions regarding this issue, please contact me at (301) 415-1016.

Sincerely,

A handwritten signature in black ink, appearing to read "Nadiyah S. Morgan", written over a horizontal line.

Nadiyah S. Morgan, Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-293

Enclosure:
RAI

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REQUEST FOR ADDITIONAL INFORMATION
REGARDING THE HEAVY LOADS LICENSE AMENDMENT REQUEST
ENTERGY NUCLEAR OPERATIONS, INC.
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

By letter dated November 26, 2013 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML13346A026), as supplemented by letters dated July 11, 2014 (ADAMS Accession No. ML14237A328), and September 11, 2014 (ADAMS Accession No. ML14258A179), Entergy Nuclear Operations, Inc. submitted a license amendment request to modify Technical Specification (TS) 4.3.4.a to permit heavy load handling (excess of 2,000 lbs) over irradiated fuel in a Multi-Purpose Canister (MPC) using a single-failure-proof handling system for Pilgrim Nuclear Power Station (Pilgrim). The September 11, 2014, supplement revised TS 4.3.4.b to reflect the removal of the energy absorbing pad from the spent fuel pool (SFP) and installation of a leveling platform. In order to complete its review, the NRC staff needs the following additional information:

1. Please provide a discussion of the details of the leveling platform point of reference such as the design, materials of construction, layout, operational characteristics, etc.
2. Provide a discussion of the technical evaluation performed to confirm the stability of the transfer cask placement on the leveling platform point of reference during a design basis seismic event.
3. Provide a brief discussion of the term leveling platform "point of reference."
4. In its September 11, 2014, letter, the licensee stated, "To accommodate the Holtec system, the energy absorbing pad was removed and replaced with the leveling platform." The licensee also stated that, "The energy absorbing pad was installed to protect the liner of the SFP from damage due to the drop of a spent fuel cask or other heavy loads in the SFP cask loading area using the original non-single-failure-proof Reactor Building crane."

With the removal of the energy absorbing pad, provide a discussion of the protection(s) in-place for the SFP liner.

5. Section 10.3.6 of the Pilgrim Updated Final Safety Analysis Report, Revision 27, contains the following statement regarding protection of the liner following a cask drop during transport cask movement in the SFP:

Analysis has demonstrated that with the energy absorber in place, damage to the floor will not result in a leakage rate greater than the pool makeup capability.

Although the use of a new single failure proof crane precludes the need to consider a cask drop, the increased weight of the MPC relative to the transport cask and the design of the leveling platform affect the loading on the SFP liner. Describe any analyses completed to verify the SFP liner integrity during MPC loading operations, including the mechanical loads considered (dead, live, and seismic), design features of the leveling platform that affect loading on the SFP liner, important analysis assumptions, and analysis results.

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/ra/

Nadiyah S. Morgan, Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-293

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RAI

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