



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 30, 2014

Vice President, Operations
Entergy Nuclear Operations, Inc.
Indian Point Energy Center
450 Broadway, GSB
P.O. Box 249
Buchanan, NY 10511-0249

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 2 – CORRECTION
LETTER TO AMENDMENT NO. 277 RE: H* ALTERNATE REPAIR CRITERIA
FOR STEAM GENERATOR TUBE INSPECTION AND REPAIR
(TAC NO. MF3369)

Dear Sir or Madam:

By letter dated September 5, 2014, the Nuclear Regulatory Commission (NRC) issued Amendment No. 277 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consisted of changes to the Technical Specifications (TSs) in response to your application dated January 16, 2014, as supplemented by letters dated April 2 and April 15, 2014.

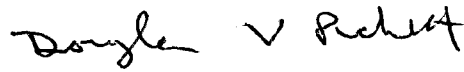
The amendment revised TS 3.4.13, "RCS Operational Leakage," and TS 5.5.7, "Steam Generator (SG) Program," to exclude portions of the SG tube below the top of the SG tubesheet from periodic inspections and plugging by implementing the H* alternate repair criteria on a permanent basis. In addition, TS 5.6.7, "Steam Generator Tube Inspection Report," was also revised to include additional reporting requirements.

The NRC staff subsequently identified an error associated with reissued TS page 3.4.13-1. Amendment No. 275 for Indian Point Nuclear Generating Unit No. 2, issued on July 7, 2014, revised a number of TS pages including TS page 3.4.13-1 to risk-inform requirements regarding selected Required Action End States. Specifically, Amendment No. 275 permitted an end state of Mode 4 rather than an end state of Mode 5 consistent with Technical Specification Task Force (TSTF) Traveler TSTF 432-A, Revision 1, "Change in Technical Specifications End States WCAP-16294." Reissued TS page 3.4.13-1 in Amendment No. 277 inadvertently removed the changes of Amendment No. 275 and reinserted the former Required Action End State of Mode 5. A corrected copy of TS page 3.4.13-1 that reinserts the Required Action End State of Mode 4 is enclosed.

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Please feel free to contact me at 301-415-1364 if you have any questions.

Sincerely,

A handwritten signature in black ink that reads "Douglas V. Pickett". The signature is written in a cursive style with a large, prominent "V" in the middle.

Douglas V. Pickett, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosure:
Corrected TS page 3.4.13-1

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3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.13 RCS Operational LEAKAGE

LCO 3.4.13 RCS operational LEAKAGE shall be limited to:

- a. No pressure boundary LEAKAGE,
- b. 1 gpm unidentified LEAKAGE,
- c. 10 gpm identified LEAKAGE, and
- d. 85 gallons per day primary to secondary LEAKAGE through any one steam generator (SG).

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. RCS operational LEAKAGE not within limits for reasons other than pressure boundary LEAKAGE or primary to secondary LEAKAGE.	A.1 Reduce LEAKAGE to within limits.	4 hours
B. Required Action and associated Completion Time of Condition A not met. <u>OR</u> Pressure boundary LEAKAGE exists. <u>OR</u> Primary to secondary LEAKAGE not within limit.	B.1 Be in MODE 3. <u>AND</u> ----- NOTE ----- LCO 3.0.4.a is not applicable when entering MODE 4. ----- B.2 Be in MODE 4.	6 hours 12 hours

Please feel free to contact me at 301-415-1364 if you have any questions.

Sincerely,

/RA/

Douglas V. Pickett, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosure:

Corrected TS page 3.4.13-1

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