

SVP-14-057

August 1, 2014

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

Quad Cities Nuclear Power Station, Units 1 and 2

Renewed Facility Operating License Nos. DPR-29 and DPR-30

NRC Docket Nos. 50-254 and 50-265

Subject:

Owner's Activity Report Submittal

Fifth 10-Year Interval 2014 Refueling Outage Activities

References:

- Letter from T. Tate (USNRC) to M. Pacilio (Exelon), "Quad Cities Nuclear Power Station, Units 1 and 2 – Safety Evaluation in Support of Request for Relief Associated with the Fifth 10 Year Interval Inservice Inspection Program," dated September 30, 2013
- Letter from T. Hanley (Exelon) to USNRC, "90-Day Post Outage ISI Letter," dated July 5, 2013

This letter submits the Owner's Activity Report (i.e., Form OAR-1) and Invessel Visual Inspection (IVVI) Report for the Quad Cities Nuclear Power Station (QCNPS) Unit 2 refueling outage (Q2R22) which began on April 7, 2014, and was completed on May 7, 2014. This is the first refueling outage conducted in the first inspection period of the fifth 10-year inservice inspection interval for QCNPS Unit 2. A copy of the Owner's Activity Report and IVVI Report are provided as Attachments 1 and 2 to this letter.

This Owner's Activity Report is submitted in accordance with American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code Case N-532-4, "Repair/Replacement Activity Documentation Requirements and Inservice Summary Report Preparation and Submission," and the Reference 1 NRC Safety Evaluation, which authorized use of Code Case N-532-4 at QCNPS (i.e., Relief Request I5R-09). Code Case N-532-4 requires an Owner's Activity Report Form OAR-1 to be prepared and certified upon completion of each refueling outage. In accordance with the conditions of Code Case N-532-4, this OAR-1 form is being submitted within ninety days of the completion of the refueling outage.

The IVVI results are provided to report completed vessel internal inspections and to document B-N-1 and B-N-2 relief request exam completion. Additionally, no welds were reclassified or changed status within the IGSCC program this outage.

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August 1, 2014 U.S. Nuclear Regulatory Commission Page 2

Finally, Attachment 3 includes a corrected NIS-2 (from the fourth 10-year inservice inspection interval) that was previously submitted in Reference 2.

Should you have any questions concerning this letter, please contact Mr. Wally Beck, Regulatory Assurance Manager, at (309) 227-2800.

Respectfully,

Scott Darin

Site Vice President

Quad Cities Nuclear Power Station

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Attachments: 1. Owner's Activity Report, Form OAR-1

2. Invessel Visual Inspection Report

3. Revised form NIS-2 Owners Report of Repair or Replacement from 4th Interval

cc: Regional Administrator -- Region III

NRC Senior Resident Inspector, Quad Cities Station

Attachment 1 Owners Activity Report, Form OAR-1

FORM OAR-1 OWNER'S ACTIVITY REPORT

lant	Quad Cities Nuclear Power Stati	on, 22710 206 th Ave	. North, Cordova, IL 61242			
nit No. 2			Refueling Outage Number			
urrent Inspection Interval	5 th Inspection In	iterval (ISI), 2 nd Insp	pection Interval (Containment)			
urrent Inspection Period	5 th Inspection In	(1 st , 2 ^m , 3 ^m , 4 ^m) eriod (ISI), 2 nd Inspe	, other) ection Period (Containment)			
	ion XI applicable to the Inspection Plans	(1st, 2nd, 3nd) 2007 Edition with 2008 Addenda (ISI), 2001 Edition with 2003 Addenda (Containment)				
ate / Revision of Inspection	n Plans	9/19/12/	Revision 0			
dition and Addenda of Sect	ion XI applicable to repair/replacement a	ctivities, if different tha N/A	in the inspection plans			
ode Cases used:	# · ·	N-532-4	9			
	tements made in this report are correct; nd (c) the repair/replacement activities a	F8 (F)	nd tests, meet the Inspection Plan as	required by t R22 stage number)		
ASME Code, Section XI; a	tements made in this report are correct; nd (c) the repair/replacement activities a	(b) the examinations a nd evaluations support	nd tests, meet the Inspection Plan asing the completion of Q2 (refueling or	R22		
ASME Code, Section XI; a conform to the requireme	tements made in this report are correct; nd (c) the repair/replacement activities a nts of Section XI	(b) the examinations a nd evaluations support	nd tests, meet the Inspection Plan asing the completion of Q2 (refueling or	R22		
ASME Code, Section XI; a conform to the requireme	tements made in this report are correct; nd (c) the repair/replacement activities a nts of Section XI ISI Coordinator (Owner or Owner's designee. Title)	(b) the examinations a nd evaluations support	nd tests, meet the Inspection Plan as ing the completion of Q2 (refueling out	R22		
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I, the undersigned, herovince of Ill have inspected the items performed all activities replacement activities and manner for any personal in the control of	tements made in this report are correct; and (c) the repair/replacement activities and ints of Section XI ISI Coordinator (Owner or Owner's designee. Title) CERTIFICATE OF Interest of the second o	NSERVICE INSP National Board of Boile HSB Global Standar and state that to the bith the requirements of ployer makes any warranermore, neither the Inty kind arising from or coss	nd tests, meet the Inspection Plan as ing the completion of Q2 (refueling or Date 7/21/14 ECTION r and Pressure Vessel Inspectors and ds of Hartford, Connected of my knowledge and belief, the Section XI anty expressed or implied concerning ispector nor his employer shall be liai	R22 ttage number) the State or ticut Owner has		

Attachment 1 Owners Activity Report, Form OAR-1

TABLE 1 ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT REQUIRED EVALUATION FOR CONTINUED SERVICE

Examination Category and Item Number	Stem Description	Evaluation Description			
B-P, 815.10	RPV Water level Instrument Penetration - 2" Nominal Pipe Size	Quad Cities Unit 2 Instrument Nozzle J-Groove Weld LEFM Flaw Evaluation (Document No. 32-9215236-002) Ref. ML 14055A227 - Quad Cities Nuclear Power Station, Unit 2- Safety Evaluation Regarding Relief Request ISR-11 For Relief For Penetration N-11 B Repair (TAC NO. MF1462)			
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Attachment 1 Owners Activity Report, Form OAR-1

TABLE 2 ABSTRACT OF REPAIR/REPLACEMENT ACTIVITIES REQUIRED FOR CONTINUED SERVICE

Code Class	Item Description	Description Of Work	Date Completed	Repair/ Replacement Plan Number
3	New RHRSW hanger pre-service exam failures (IR 1470365)	WO 1454147 - New hangers which required rework prior to putting into service.	2/19/13	RRP 12-047, 12- 074, 12-088
3	2-5746-A -2A RHR ROOM COOLER HAS TUBESHEET CORROSION THAT NEEDS REPAIR (IR 01556124)	WO 1671617 - Excavate and weld repair 2A RHR room cooler tubesheet and install tube plugs	9/11/13	RRP 13-081
3	Leak found on LINE 2-101168-2"-D to the 28 RHRSW Cubicle Cooler (IR 1617892)	WQ 1710206 - 28 RHRSW Cubicle Cooler pipe replacement	2/7/14	RRP 14-010
MC	Areas of X-2 Hatch Moisture Barrier Needs Replacing (IR 1647016)	WO 1730517 - Replaced Moisture Barrier	4/25/14	RRP 14-027
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Attachment 2

Invessel Visual Inspection Report

The ASME Section XI inspections credited during Q2R22 IVVI activities include the once-per-period B-N-1 inspection of the reactor vessel interior and B-N-2 inspections of reactor vessel interior attachments. Credit is being taken for the BWRVIP examinations in place of the B-N-1 and B-N-2 examinations in accordance with Quad Cities Station Relief Request I5R-06, "Request for Relief for the Use of BWRVIP Guidelines in Lieu of Specific ASME Code Requirements on Reactor Vessel Internals and Components Inspection In Accordance with 10CFR50.55a(a)(3)(i) Alternative Provides Acceptable Level of Quality and Safety".

To implement the requirements of the Boiling Water Reactor Vessel Internals Program (BWRVIP), GE was contracted to perform the In-Vessel Visual Inspections (IVVI). The following components and assemblies were examined:

- Visual inspections of the Steam Dryer wall support lugs
- Visual inspections of Core Shroud Vertical welds V5, V6, V7, V26, V27, and V28
- Visual inspections of Instrument Nozzle Upper and Lower RPV interior surfaces

To implement the requirements of the Boiling Water Reactor Vessel Internals Program (BWRVIP), GE was contracted to perform the In-Vessel Visual Inspections (IVVI). The following components and assemblies were examined:

- 42 components on jet pump assemblies, including UT on 20 jet pump beams, in accordance with the BWRVIP-41 Rev. 3, "BWR Jet Pump Assembly Inspection and Flaw Evaluation Guidelines".
- 36 core spray piping welds in accordance with the BWRVIP-18 Revision 1-A, "BWR Core Spray Internals Inspection and Flaw Evaluation Guidelines".
- All of the core spray sparger brackets, 50% of the sparger nozzle welds, and all of the sparger pipe end cap welds in accordance with the BWRVIP-18 Revision 1-A, "BWR Core Spray Internals Inspection and Flaw Evaluation Guidelines".
- 6 core shroud vertical welds in accordance with BWRVIP-76 Revision 1, "BWR Core Shroud Inspection and Flaw Evaluation Guidelines".
- 4 Attachment welds for the steam dryer support lugs in accordance with BWRVIP-48-A, "Vessel ID Attachment Weld Inspection and Flaw Evaluation Guidelines".
- Steam dryer examinations of the tee to cap doubler welds. Dryer inspections were performed in accordance with BWRVIP-139-A, "Steam Dryer Inspection and Flaw Evaluation Guidelines".

Attachment 2

Invessel Visual Inspection Report

In addition to the BWRVIP inspection guidance, the following augmented examinations were also performed as part of the Q2R22 IVVI activities:

- 48 jet pump components not required by BWRVIP-41 Rev. 3, "BWR Jet Pump Assembly Inspection and Flaw Evaluation Guidelines" per programmatic requirements.
- Selected cast austenitic stainless steel components (CASS) in accordance with license renewal commitments.
- Accessible interior surfaces on four instrument nozzles per programmatic requirements.
- Reinspection of mechanical damage at 3 steam separator locations per programmatic requirements.
- Reinspection of mechanical damage at 3 steam dryer locations per programmatic requirements.

Multiple indications (gouges and rub marks) were identified on all four of the Steam Dryer wall support lugs. These indications were previously observed in Q2R21. A comparison of the inspection results from Q2R21 to Q2R22 found no apparent change to the previous indications and no new indications were identified. These indications are documented in IR 1648005 and evaluated for continued service under EC 397522. Quad Cities will continue to monitor these conditions in accordance with the Reactor Internals Program.

Attachment 3

Revised form NIS-2 Owners Report of Repair or Replacement from 4th Interval

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FORM NIS-2 OWNERS REPORT OF REPAIR OR REPLACEMENT As Required by the Provisions of ASME Code Section MI

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3.	Work Performed by: Qual Cities Generating Station 22710 206th Aug. North, Conducts E. 61242				Type Code Symbol Status: Name Authorization No.: Name Expiration Date: NA				
4.	. Identificat	tion of S	vstem: <u>Rankhusl Heat</u>	Busional Class.2					
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