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Carolina Power & Light Company Raleigh, N.C. 27602 Mr. N.B. Bessac	2-20-74	2-25-74	X			

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J.F. O'Leary	3 signed			XXX	XXX

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DESCRIPTION:
Ltr submitting the Containment Structural Test notice....adv that this test will be conducted during the scheduled shutdown in April of 1974.. ...

ENCLOSURES:

ACKNOWLEDGED

DO NOT REMOVE

PLANT NAME: H.B. Robinson

FOR ACTION/INFORMATION 2-26-74 JB

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Regulatory File Cy.

CP&L

Carolina Power & Light Company

February 20, 1974

30 CFR

File: NG-5430 & NG-3514

Serial: NG-74-223

Mr. John F. O'Leary, Director
Directorate of Licensing
Office of Regulation
U. S. Atomic Energy Commission
Washington, D. C. 20545

Dear Mr. O'Leary:

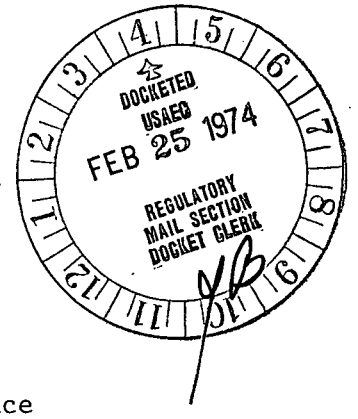
50-261

H. B. ROBINSON UNIT NO. 2
LICENSE DPR-23
CONTAINMENT STRUCTURAL TEST

In accordance with Section 4.4.4.3.c of the Technical Specifications for H. B. Robinson Unit No. 2, notification of subject test is given. The containment structural test will be conducted during the refueling shutdown presently scheduled to commence April 20, 1974. Acceptance criteria are as follows:

Section 4.4.4.3 (Acceptance Criteria) of the Technical Specifications for the H. B. Robinson Unit No. 2 states:

b. Observation of the structural test at design pressure indicating no significant differences in containment growth and crack pattern spacing and width from that during the proof test shall be considered as demonstrating the continual integrity of the structure. It is realized that the deflections, in the prestressed direction particularly, will be small, that the significance of differences in these small deflections will be difficult to evaluate, and therefore that only a gross difference in the structure, such as a large loss of prestress force, would be apparent from the measurements. The difference in measurements, if any, will be examined considering the predictable range of variation of time dependent changes in material properties, the thermal conditions at the time of the test, instrument error and other pertinent factors.



1519

336 Fayetteville Street • P. O. Box 1551 • Raleigh, N. C. 27602

As quantification of the above acceptance criteria, the following are the maximum acceptable displacements:

a. Radial Displacements

1. Containment wall 0.700 inch
2. Equipment hatch 0.575 inch

b. Vertical Displacements

Containment wall 0.146 inch

The maximum acceptable criteria are derived using the response of the containment structure during the initial proof test plus a 20 percent tolerance.

Acceptable crack patterns are as follows:

1. Width of crack 0.027 inch
2. Average spacing 17 inches (vertical)

Within six months following the test a report and evaluation will be submitted to the Atomic Energy Commission.

Yours very truly,

N. B. Bessac
Manager
Nuclear Generation

DLF:mvp
Attachment

cc: Messrs. T. E. Bowman
B. J. Furr
W. B. Howell
D. V. Menscer
E. E. Utley
D. B. Waters