UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 31, 2014

Mr. Eric A. Larson, Site Vice President
FirstEnergy Nuclear Operating Company
Beaver Valley Power Station
Mail Stop A-BV-SEB1
P.O. Box 4, Route 168
Shippingport, PA 15077

SUBJECT: BEAVER VALLEY POWER STATION, UNITS 1 AND 2 – CORRECTION LETTER REGARDING AMENDMENT NOS. 275 AND 156 REGARDING ISSUANCE OF AMENDMENT FOR THE 8-PERCENT EXTENDED POWER UPRATE (TAC NOS. MC4645 AND MC4646)

Dear Mr. Larson:

By letter dated July 19, 2006,[1] the Nuclear Regulatory Commission (NRC) issued Amendment No. 275 to Facility Operating License No. DPR-66 for the Beaver Valley Power Station, Unit No. 1 (BVPS-1) and Amendment No. 156 to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit No. 2 (BVPS-2) in response to your application dated October 4, 2004.[2] Amendment Nos. 275 and 156 consisted of changes to the Technical Specifications and Facility Operating License, authorizing an approximate 8-percent increase in the licensed rated thermal power from 2,689 megawatts thermal (MWt) to 2,900 MWt.

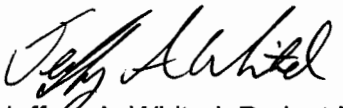
Amendment No. 275 was effective as of July 19, 2006, and was to be implemented prior to startup following the fall 2007 refueling outage. Amendment No. 156 was effective as of July 19, 2006, and was to be implemented prior to startup following the spring 2008 refueling outage.

During a conversation between Mr. Phil Lashley, of your staff, and the NRC staff, it was noted that Table 4, "Parameters and Assumptions Used in Radiological Consequence Calculations Fuel Handling Accident for BVPS-1 and 2," on page 172 of the NRC staff's Safety Evaluation (SE) dated July 19, 2006,[3] contained errors. Specifically, the values of Iodine 131 (I-131) and Krypton 85 (Kr-85) Core fractions released from damaged rods listed in the table are incorrect. In the NRC staff's SE the values are listed as 0.12 for I-131 and 0.14 for Kr-85. This was an inadvertent typographical error. The values that were submitted in the application and the values that should be in Table 4 of the SE are 0.08 for I-131 and 0.10 for Kr-85. To correct this error, a revised copy of page 172 is enclosed. Please replace the page 172 that was issued by letter dated July 19, 2006, with the enclosed page.

1 Agencywide Documents Access and Management System (ADAMS) Accession No. ML061720274.
2 ADAMS Accession No. ML042920300.
3 ADAMS Accession No. ML061720376.

If you have any questions, please contact me at 301-415-4090.

Sincerely,



Jeffrey A. Whited, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

Enclosure:
As stated

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Control room	
0 to 2 hours	5.01E-4
2 to 8 hours	3.58E-4

*Note: The partition coefficient of 1.0 is applied only to the flashed portion of the break flow, A partition coefficient of 100 is applied to the unflashed portion of the break flow.

Table 4
Parameters and Assumptions Used in Radiological Consequence Calculations Fuel Handling Accident For BVPS-1 and 2

Parameter	Value
Power level, MWt	2918
Peaking factor	1.75
Number of fuel rods in fuel assemblies	264
Total number of fuel assemblies	157
Number of fuel rods damaged	137
Reactor shutdown time before fuel movement, hr	100
Core fractions released from damaged rods	
I-131	0.08
Other halogens	0.05
Kr-85	0.10
Other noble gases	0.05
Alkali metals	0.12
Iodine effective pool decontamination factor	200
Duration of release	Instantaneous
Atmospheric dispersion values (sec/m^3)	
Exclusion area boundary	
0 to 2 hours	1.25E-3 (BVPS-1)/1.04E-3 (BVPS-2)
Low population zone	
0 to 8 hours	6.04E-5
Control room	

	BVPS-1	BVPS-2
0 to 2 hours	4.75E-3	9.39E-4
2 to 8 hours	3.66E-3	6.69E-4

If you have any questions, please contact me at 301-415-4090.

Sincerely,

/RA/

Jeffrey A. Whited, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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As stated

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