

April R. Rice Manager New Nuclear Licensing

July 28, 2014 NND-14-0424 10 CFR 50.59 10 CFR 52 App D

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

Virgil C. Summer Nuclear Station (VCSNS) Units 2 and 3 Docket Numbers 52-027 and 52-028 Combined License Numbers NPF-93 and NPF-94

Subject:

Reporting of 10 CFR 50.59 Changes, Tests, and Experiments and 10 CFR 52

Appendix D Section VIII Departures

Reference:

1. Letter from Alfred M. Paglia (SCE&G) to Document Control Desk (NRC), dated

January 27, 2014, Reporting of 10 CFR 50.59 Changes, Tests, and Experiments and 10 CFR 52 Appendix D Section VIII Departures

In accordance with 10 CFR 50.59(d)(2), VCSNS Units 2 and 3 is required to submit a report to the NRC containing a brief description of any changes, tests or experiments made pursuant to 10 CFR 50.59(c), including a summary of the evaluation of each. This 10 CFR 50.59 report is for the period beginning January 28, 2014 and ending July 28, 2014. During that period there were no changes, tests or experiments made pursuant to paragraph (c) of 10 CFR 50.59.

Additionally, as required by paragraphs X.B.1 and X.B.3.b of Appendix D to 10 CFR Part 52, this submittal contains a report of all plant-specific departures made in this reporting period. The 10 CFR 52 Appendix D Departure Report is provided in Enclosure 1 to this letter and covers the period beginning January 28, 2014 and ending July 28, 2014.

This letter makes no new regulatory commitments.

Should you have any questions, please contact me by telephone at (803) 941-9858, or by email at arice@scana.com.

Sincerely,

April R. Rice Manager

**New Nuclear Licensing** 

BB/ARR/bb

Enclosure 1: V.C. Summer Nuclear Station Units 2 and 3 Departure Report: January 28, 2014 through July 28, 2014

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# South Carolina Electric & Gas

## NND-14-0424

## Enclosure 1

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-12-004** 

AD/Screening: LCE-13-008

Originating Document: APP-FSAR-GLN-062

Title: Chemical and Volume Control System (CVS) Changes

#### **Activity Description:**

The Chemical and Volume Control System (CVS) as described in the UFSAR is changed to provide a spring-assisted check valve (CVS-PL-V067) around the air-operated Reactor Coolant System (RCS) Purification Return Line Stop Check Valve (CVS-PL-V081); replace the CVS zinc addition inboard containment isolation lift check valve with an air-operated globe valve and a thermal relief valve; and separate the zinc and hydrogen injection paths and relocate the zinc injection point.

A second portion of this departure revised UFSAR, Table 3.2-3, AP1000 Classification of Mechanical and Fluid Systems, Components, and Equipment, to add new zinc addition valves CVS-PL-V065 and CVS-PLV095.

## **Summary of Evaluation:**

Portions of this departure involved Tier 1 information, Combined License (COL) Appendix C, and Tier 2 information in the UFSAR which involved changes to Tier 1 information; therefore, a License Amendment and Exemption Request (LAR 13-07) was submitted to the NRC.

The NRC approved this departure and exemption and issued License Amendment 11 to Combined Licenses NPF-93 and NPF-94 for VCSNS Units 2 and 3, respectively.

The second portion of this departure does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment. The second portion of the departure did not involve a change to Tier 1 information, Tier 2\* information or the Technical Specifications. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-14-052

AD/Screening: LCE-13-030

**Originating Document:** VSG-FSAR-GLN-002

**Title:** Turbine Building Structures & Layout Changes

#### **Activity Description:**

This change revised the Turbine Building arrangement to relocate the door in the southeast corner of Rm 20203 (Motor Driven Fire Pump Room), increase floor heights in the Turbine Building (except First Bay), and increase the thickness of walls near Column lines 1.2 and R.

Relocation of the door supports the structural design of the Turbine Building without impacting the room's design function. Changes in floor elevations allow room for piping and supports and the increased wall thicknesses support pressurization in event of a main steam line break in the Turbine Building First Bay.

## **Summary of Evaluation:**

Portions of this departure involved Tier 1 information, Combined License (COL) Appendix C, and Tier 2 information in the UFSAR which involved changes to Tier 1 and Tier 2\* information; therefore, a License Amendment and Exemption Request (LAR 13-13) was submitted to the NRC.

The NRC approved this departure and exemption and issued License Amendment 12 to Combined Licenses NPF-93 and NPF-94 for VCSNS Units 2 and 3, respectively.

Those portions of the activities, which do not involve a Tier 2\*, Technical Specifications or Tier 1 information (including ITAAC) change, do not adversely affect any design function, involve a procedure or method of control that affects the performance of a design function, involve a method of evaluation in UFSAR, involve a test or experiment, nor a design feature credited in the ex-vessel severe accident assessment. Therefore, a 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-095** 

AD/Screening: LCE-13-048

**Originating Document:** CR-NND-13-01258

**Title:** Changes to Initial Test Program in UFSAR Ch 14.2 and associated sections of Ch 13.

#### **Activity Description:**

These changes revise UFSAR Ch 14.2 to define the construction and installation tests as two parts, construction testing and component testing, and moves the component testing portion under the administrative controls of the ITP organization. There are also several administrative or editorial changes made for clarification. Conforming changes to Ch 13 are made to maintain consistency between Ch 14 and Ch 13.

#### **Summary of Evaluation:**

The changes to UFSAR Ch 14 and associated sections of Ch 13 did not involve a change to Tier 2\* information, plant-specific Tier 1 information (including ITAAC) or a Technical Specifications change. The activities did not adversely affect any design function, involve a procedure or method of control that affects the performance of a design function, involve a method of evaluation described in the UFSAR, involve a test or experiment, nor involve a design feature credited in the ex-vessel severe accident assessment. A 10 CFR 50.59/52 Appendix D Section VIII review determined that no prior NRC approval is required.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-082

AD/Screening: LCE-14-002

**Originating Document:** APP-FSAR-GLN-117

Title: Containment Vessel Emissivity

#### **Activity Description:**

These changes make clarifications to UFSAR Table 6.2.1.1-8 concerning the property emissivity and its value. Changes are necessary due to testing of procured inorganic zinc coatings that does not comply with the licensing basis. In addition, it is conservative to make the assumption that wet emissivity is assumed to be that of dry emissivity.

The activity involved replacing the IOZ dry emissivity value of the CV protective coating with a value based on supplier test data for use in the WGOTHIC containment safety analysis. The change revises dry emissivity design criteria for IOZ protective coating in design documentation from 0.81 to 0.60 and changes the value in UFSAR Table 6.2.1.1-8 from 0.81 to 0.54. The conservative 10% reduction from 0.60 to 0.54 minimizes the radiation heat transfer as discussed in NRC AP1000 FSER (NUREG-1793), Subsection 6.2.1.1, "Containment Pressure and Temperature Response to High-Energy Line Breaks". The change also assumes the wet emissivity of each heat sink and coating material Table 6.2.1.1-8 are equal to the respective dry emissivity based on testing results.

## **Summary of Evaluation:**

The changes do not result in modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or on the control of the reactions in the core design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-14-017** 

AD/Screening: LCE-14-003

**Originating Document:** APP-FSAR-GLN-268

Title: Change to Specific Heat Values for IOZ and Epoxy Coatings

#### **Activity Description:**

This change revises UFSAR Table 6.2.1.1-8 to lower the specific heat values for the inorganic zinc (IOZ) coating materials.

Manufacturer tested values of the specific heat for IOZ coating for the surface of the containment vessel have established that it is less than the value given in Updated Final Safety Analysis Report (UFSAR). The values for the specific heat of the IOZ coating in Table 6.2.1.1-8 is changed from 0.13 BTU/lbm-°F to 0.11 BTU/lbm-°F for use in the peak containment pressure analyses. Evaluations have determined that this lower specific heat value is acceptable in design basis accident containment vessel pressure analyses and does not impact the calculated peak pressure reported in the licensing basis.

#### **Summary of Evaluation:**

The departure did not adversely affect any design function, involve a procedure or method of control that affects the performance of a design function, involve a method of evaluation described in the UFSAR, involve a test or experiment, nor involve a design feature credited in the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-14-010** 

AD/Screening: LCE-14-004

**Originating Document:** CR-NND-14-00076

Title: Incorporation error of a previous change

#### **Activity Description:**

The activity removes four additional columns that were incorrectly added to UFSAR Table 3.9-16 during incorporation of approved License Amendment 8.

Markups provided by Westinghouse contained a typographical error identifying the specific change and was incorporated into the UFSAR Table before it was discovered. The change is made to be consistent with NRC approved Amendment 8.

#### **Summary of Evaluation:**

This departure was screened and it was determined the change to Tier 2 Table 3.9-16 did not involve a change to Tier 2\* information, plant-specific Tier 1 information (including ITAAC) or a Technical Specifications change, the activities did not adversely affect any design function, involve a procedure or method of control that affects the performance of a design function, involve a method of evaluation described in the UFSAR, involve a test or experiment, nor a design feature credited in the ex-vessel severe accident assessment. The 10 CFR 50.59 *I* 52 Appendix D Section VIII review determined that no prior NRC approval is required.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-022

AD/Screening: LCE-14-006

**Originating Document:** APP-FSAR-GLN-167

Title: Liquid Radwaste System (WLS) Figure Changes

#### **Activity Description:**

The Piping and Instrumentation Diagram (P&ID) for the WLS, as shown in UFSAR Figures 11.2-2 is updated to depict the current design, which includes a heat exchanger; associated valves; and temperature elements in the seal water line from the degasifier separator pumps to the degasifier vacuum pumps. In addition, the Normal Residual Heat Removal System (RNS) relief valve discharge line to the WLS is relocated to ensure alignment to one of the two effluent holdup tanks.

#### **Summary of Evaluation:**

The change to the WLS P&ID to depict a heat exchanger and associated components in the seal water line and to reroute the RNS relief valve discharge line does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-088

AD/Screening: LCE-14-007

**Originating Document:** APP-FSAR-GLN-091

**Title:** Changes for Six Shield Building Pipe Penetrations

#### **Activity Description:**

A wall opening in the shield building at which six pipe anchor penetrations are located is eliminated and the reinforcement bars are now continuous through the closed-in opening. A steel structure, required to support the penetrations in the opening, is replaced with reinforced concrete surrounding the individual penetrations.

## **Summary of Evaluation:**

The change to replace a single large opening in the shield building to six individual penetrations does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3 Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-072** 

AD/Screening: LCE-14-008

**Originating Document:** APP-FSAR-GLN-247

Title: Gaseous Radwaste System (WGS) Delay Bed Design Temperature Change

## **Activity Description:**

UFSAR Chapter 11, Table 11.3-2, Component Data (Nominal) - Gaseous Radwaste System, Delay Bed design temperature is changed from 150°F to 200°F.

#### **Summary of Evaluation:**

The change to increase the design temperature of the WGS delay beds does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

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**Revision Notice:** RN-13-050

AD/Screening: LCE-14-009

**Originating Document:** APP-FSAR-GLN-210

Title: AP1000 Equipment Classification Updates

#### **Activity Description:**

UFSAR Chapter 3, Table 3.2-3, AP1000 Classification of Mechanical and Fluid Systems, Components, and Equipment is updated to reflect AP1000 Class, Seismic Category, and Principal Construction Code of components in various plant systems for consistency with design information. Additional components are added to this table to provide greater level of detail.

## **Summary of Evaluation:**

The change to update Table 3.2-3 does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-056** 

AD/Screening: LCE-14-010

Originating Document: APP-FSAR-GLN-222

Title: Containment Penetration Classification

#### **Activity Description:**

This change involved ASME classification of containment penetrations in UFSAR Table 3.2-3 due to design change of penetrations.

This activity modified the design and subsequent classification of five mechanical penetration sleeves. A possible overstress issue was identified with the previous design, and a correction was made to address this. In accordance with the UFSAR defined classification methodology, the new penetration sleeves required a change to their principal construction codes from ASME III, Class 2 to ASME III, Class MC. This information is not included in Tier 1, Tier 2\*, or the Technical Specifications.

#### **Summary of Evaluation:**

The activity/departure did not involve a Tier 2\*, Technical Specifications or plant-specific Tier 1 information (including ITAAC) change. The departure did not adversely affect any design function, involve a procedure or method of control that affects the performance of a design function, involve a method of evaluation described in the UFSAR, involve a test or experiment, nor a design feature credited in the ex-vessel severe accident assessment. The 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that prior NRC approval is not required.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-030

AD/Screening: LCE-14-011

**Originating Document:** APP-FSAR-GLN-197

Title: WLS Drain Safety Classification and Valve Depictions

#### **Activity Description:**

This change involves the revision of Tier 1 Figure 2.3.1 0-1 and involved Tier 2 changes. The original configuration on the figures does not indicate that the drains are nonsafety-related. The activity also involved modifying the depiction of the piping connection types for WLS drain lines, mobile equipment connection configurations, valve types, and other aspects of the WLS for consistency with the detailed Tier 2 figures and Tier 1 drawing nomenclature. The activity further clarified the safety classification of the WLS drain hubs.

#### **Summary of Evaluation:**

Portions of this departure involved Tier 1 information and involved Tier 2 changes; therefore, a License Amendment and Exemption Request (LAR 13-32) was submitted to the NRC.

The NRC approved this departure and exemption and issued License Amendment 10 to Combined Licenses NPF-93 and NPF-94 for VCSNS Units 2 and 3, respectively.

The activity which did not involve a Tier 2\* information, a plant-specific Tier 1 information (including ITAAC) or a Technical Specifications change, did not affect any design function, involve a procedure or method of control that affects the performance of a design function, involve a method of evaluation described in the UFSAR, involve a test or experiment, nor a design feature credited in the ex-vessel severe accident assessment. For those portions of this departure that did not involve a change to Tier 2\* information, Tier 1 information or the Technical Specifications, the 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-14-005

AD/Screening: LCE-14-012

**Originating Document:** APP-FSAR-GLN-395

Title: Inconsistency Between Current Licensing Basis (CLB) and Current Design on CA20

#### **Activity Description:**

Comparison of text and figures presented in the licensing basis with design finalization for the CA20 Module identified a clarification that was needed related to the description of the thickness of minor walls in the auxiliary building structural modules.

The purpose of this change was to clarify the licensing basis related to the thickness of structural wall modules in the auxiliary building. The description of the structural modules in the auxiliary building in UFSAR Subsection 3.8.4.1.2 is revised to include the walls (e.g. barrier access and labyrinth walls) less than 2'-6" thick and to specify that the 2'-6" to 5'-0" thickness range is applicable to key structural walls. The revision makes the UFSAR description consistent with the walls shown in UFSAR Figure 3.8.4-4 (Sheet 1) and the detail design. The subject walls are on Elevation 66'-6" and do not extend above Elevation 82'-6".

#### **Summary of Evaluation:**

The activity/departure did not involve a Tier 2\*, Technical Specifications or plant-specific Tier 1 information (including ITAAC) change. The departure did not adversely affect any design function, involve a procedure or method of control that affects the performance of a design function, involve a method of evaluation described in the UFSAR, involve a test or experiment, nor a design feature credited in the ex-vessel severe accident assessment. The 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that prior NRC approval is not required.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-14-002

AD/Screening: LCE-14-013

Originating Document: APP-FSAR-GLN-236

**Title:** Changes to Radwaste System Pump Description

#### **Activity Description:**

This change affects the pumps which support the functions of the radwaste systems in the AP1000 including those in the Liquid Radwaste System (WLS), and the Solid Radwaste System (WSS).

This activity makes the following UFSAR changes: (1) revises Subsection 12.3.1.1.1 to add that there are threaded connections for radwaste pumps in addition to flanged connections. These threaded pump connections only apply to the air lines to the radwaste pumps which are not expected to have radioactive waste in them; and (2) revises UFSAR Table 11.4-10 for the resin mixing pumps to change the design flow rate from 120 gpm to 100 gpm, and to delete the line item for the pump head.

#### **Summary of Evaluation:**

The activity/departure did not involve a Tier 2\*, Technical Specifications or plant-specific Tier 1 information (including ITAAC) change. The departure did not adversely affect any design function, involve a procedure or method of control that affects the performance of a design function, involve a method of evaluation described in the UFSAR, involve a test or experiment, nor a design feature credited in the ex-vessel severe accident assessment. The 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that prior NRC approval is not required

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-026

AD/Screening: LCE14-014

Originating Document: APP-FSAR-GLN-164

**Title:** CMT Flow Basis for Providing Injection

#### **Activity Description:**

This departure clarifies Technical Specification (TS) Bases B 3.5.2 regarding the statements for the duration of CMT flow injection after CMT actuation, by identifying the duration as approximately 20 minutes.

## **Summary of Evaluation:**

The change to TS Bases B 3.5.2 to clarify the statements regarding the duration of CMT flow injection after CMT actuation does not involve a test or experiment not described in the UFSAR or modification or addition that adversely affects: 1) a design function of an SSC described In the UFSAR, 2) a method of performing or controlling the design function as described In the UFSAR, or 3) an evaluation for demonstrating that intended design function will be accomplished. There Is no modification, addition to, or removal of an SSC such that any design function is adversely affected. There is no affect on a design feature credited in the severe accident assessment in the UFSAR.

Further, this departure has no adverse effect on the frequency or consequences of accidents or malfunction of an SSC Important to safety. There is no adverse impact on any fission product barrier design basis limit, and there is no failure of an SSC leading to a different result than previously analyzed. Consequently, this change has no impact on ex-vessel *severe* accident likelihood or consequences. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-076** 

AD/Screening: LCE-14-015

**Originating Document:** APP-FSAR-GLN-170

Title: Gaseous Radwaste System (WGS) Clarifications

#### **Activity Description:**

UFSAR Chapter 11, Section 11.3, Gaseous Waste Management System (WGS) is changed to clarify that the WGS discharge isolation valve opens during system operation and fails in the closed position. In addition, the WGS Piping and Instrumentation Diagram (P&ID) is updated to depict additional design details.

## **Summary of Evaluation:**

The change to clarify and update WGS design information in the UFSAR does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-085

AD/Screening: LCE-14-016

**Originating Document:** APP-FSAR-GLN-251

Title: Liquid Radwaste System (WLS) Degasifier Vacuum Pump Changes

#### **Activity Description:**

In UFSAR Chapter 11, Table 11.2-2, Component Data-Liquid Radwaste System, the design temperature of the WLS degasifier vacuum pumps is changed from 200°F to 150°F; and the design flow is changed from 0.5 steady standard cubic feet per minute (scfm) hydrogen flow and 150 scfm hogging flow to 641 scfm total hydrogen and water vapor flow.

## **Summary of Evaluation:**

The change to the degasifier vacuum pump design temperature and design flow does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

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**Revision Notice:** RN-14-018

AD/Screening: LCE-14-017

Originating Document: APP-FSAR-GLN-429

Title: Liquid Radwaste System (WLS) Chemical Waste Tank Volume

#### **Activity Description:**

The volume of the chemical waste tank in the WLS is reduced from 8,900 gallons to 7,700 gallons, and the description of chemical wastes that the chemical waste tank may contain is changed to differentiate between chemical and detergent wastes and list the types of detergent wastes and expanded to indicate that processing may occur without mobile equipment.

#### **Summary of Evaluation:**

The change to the chemical waste tank volume and to the description of chemical wastes does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-14-011** 

AD/Screening: LCE-14-019

**Originating Document:** APP-FSAR-GLN-172

**Title:** Fire Protection Analysis Changes

#### **Activity Description:**

UFSAR Chapter 9, Appendix 9A, Fire Protection Analysis is updated to add Fuel Handling Area Room 12562 to the list of fire zones; and to change the fire zone number for the lower annulus southeast and the room numbers for the Spent Fuel System (SFS) Pump Rooms A and B on Nuclear Island Fire Area Figure 9A-1.

#### **Summary of Evaluation:**

The change to update the fire zone and room information in the UFSAR fire protection analysis does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-14-006** 

AD/Screening: LCE-14-020

Originating Document: APP-FSAR-GLN-288

Title: Makeup Filter and Reactor Coolant Filters in the Chemical and Volume Control System

#### **Activity Description:**

Design pressure parameters for the makeup filter and reactor coolant filters in the CVS are updated in UFSAR Subsection 9.3.6, consistent with design parameters. This is an editorial change.

UFSAR Table 9.3.6-2 correctly shows the makeup pump and reactor coolant filter design pressure as 3100 psig; however, section 9.3.6.5 states that the reactor coolant filter is designed for the reactor coolant system hydrostatic test pressure. Therefore, section 9.3.6.5 is revised to show that the correct reactor coolant filter design pressure is the same as that of the makeup pump, i.e. 3100 psig.

#### **Summary of Evaluation:**

The change to the design pressure parameters for the makeup filter and reactor coolant filters does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-080** 

AD/Screening: LCE-14-021

**Originating Document:** APP-FSAR-GLN-310

Title: Changes to Components in the Liquid Radwaste System (WLS) and Chemical and

Volume Control System (CVS)

#### **Activity Description:**

WLS and CVS component information and parameters in UFSAR Section 9.3 and Section 11.2 are updated to reflect current design information. These changes are editorial changes being made to remove inconsistencies between certain sections of the UFSAR and other sections of the UFSAR and the design documentation. There are no technical changes.

Changes include (1) revise the letdown flow isolation valve isolation signal, (2) revise the number of sample points for WLS discharge monitoring, (3) note that secondary system effluent is processed by the waste water system, (4) remove reference to the WLS displays and control panels as being portable, (5) correct the WLS discharge line isolation valve number, and (6) correct the description of conditions under which CVS letdown is diverted to the WLS system.

#### **Summary of Evaluation:**

The change to update the design information for components in the WLS and CVS does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

Revision Notice: RN-14-012

AD/Screening: LCE-14-022

Originating Document: CR-NND-13-1004

Title: Corrections to the referenced ASME sections in UFSAR Chapter 5

#### **Activity Description:**

Minor changes to the UFSAR Sections 5.2.4.5 and 5.2.4.6 to clarify the correct sections of the ASME code that are used when inspecting and testing Class 1 components.

Improperly referenced sections of the ASME code were used. Two of these were referencing sections that do not exist. The third was a transposition of words which, in context of the sentence, created an inconsistency.

#### **Summary of Evaluation:**

The corrections to the referenced ASME sections in UFSAR Chapter 5 do not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

Revision Notice: RN-13-042

AD/Screening: LCE-14-023

Originating Document: APP-FSAR-GLN-203

Title: Spent Resin Tank Usable Volume

#### **Activity Description:**

The usable volume of each spent resin storage tank is changed from 275 cubic feet to 250 cubic feet as described in UFSAR Chapter 11, Section 11.4, Solid Waste Management.

#### **Summary of Evaluation:**

The change to the usable volume of each spent resin storage tank does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-046

AD/Screening: LCE-14-024

**Originating Document:** APP-FSAR-GLN-101

**Title:** Fire Area Updates

#### **Activity Description:**

The figure showing the fire rating of the wall between the Main Control Room (MCR) area and the Operator Break Room and Shift Supervisor's Office is changed to be consistent with the description provided in the Fire Protection Analysis, Appendix 9A of the UFSAR. Several other changes and updates to fire zones within the Auxiliary Building are also made that do not affect a fire area boundary.

## **Summary of Evaluation:**

The change to the Auxiliary Building fire area information in the UFSAR does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-12-055** 

AD/Screening: LCE-14-025

**Originating Document:** APP-FSAR-GLN-060

**Title:** Modifications to the RNS ME1 C Heat Exchangers

#### **Activity Description:**

This activity addresses the drain configurations of the two normal residual heat removal system (RNS) heat exchangers that were previously upgraded from 6-tube pass to 8-tube pass units for more heat removal capacity. The two 6-tube pass RNS heat exchanger head drain isolation valve assemblies (one valve and associated piping per each 6-tube pass heat exchanger) are removed and ten new RNS heat exchanger head drain isolation valve assemblies are added (five independent valves and associated piping feeding a common drain header per each 8-tube pass heat exchanger). The RNS 8-tube pass heat exchanger has five zones requiring draining and, therefore, it requires five drain valve assemblies for complete draining.

## **Summary of Evaluation:**

By changing the drain configurations of the two RNS heat exchangers such that they adequately drain, the RNS design function is unchanged. These drain configurations changes have no affect on structural analysis and do not impact the Aircraft Impact Assessment. There is no adverse impact on RNS heat exchanger operation. Additionally, changing the drain configurations does not impact security barriers, and does not significantly impact radiation protection and shielding safety analyses. These changes do not affect any procedure, method of evaluation, or test and experiment. The changes do not have an impact on ex-vessel severe accident consequences and do not impact containment pressurization. A 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required .

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-068

AD/Screening: LCE-14-026

**Originating Document:** APP-FSAR-GLN-208

Title: Spent Fuel System (SFS) Piping Requirement Changes

#### **Activity Description:**

The description of the piping requirements for the SFS as specified in UFSAR Subsection 9.1.3 is changed to clarify that portions of the system (rather than the entire system) are stainless steel, and that the SFS piping also contains threaded caps.

## **Summary of Evaluation:**

The change to the SFS piping requirements does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the exvessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-064

AD/Screening: LCE-14-028

**Originating Document:** APP-FSAR-GLN-159

Title: Chemical and Volume Control System (CVS) Changes

#### **Activity Description:**

The CVS, as described in UFSAR Chapter 3 and Chapter 9 is changed to add a bypass line around the CVS ion exchangers; add a pressure regulating valve and flow meter in the demineralized water supply line to the CVS three-way blend valve; add an air intrusion prevention tank; and change the CVS purification bypass line three-way valve from a plug valve to a globe valve.

#### **Summary of Evaluation:**

The change to the CVS as described in the UFSAR does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the exvessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-14-032

AD/Screening: LCE-14-029

**Originating Document:** CR-NND-14-00202

**Title:** Correction to an Error Made During Incorporation of Approved Amendment

## **Activity Description:**

This activity applies to NRC approved UFSAR Tier 2\* text which was approved in LAR 13-02, Amendment 2, but not incorporated correctly in section 3.8.4.4.1. The change corrects the text implementation in the UFSAR to be consistent with NRC approved Amendment 2.

#### **Summary of Evaluation:**

This departure corrects the incorrect implementation of NRC approved Tier 2\* text added to UFSAR section 3.8.4.4.1. The activity does not include changes to Tier 1 information, Tier 2\* information, or Technical Specifications. A 10 CFR 50.59-10 CFR 52 Appendix D Section VIII review determined that no prior NRC approval is required for this activity.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-096** 

AD/Screening: LCE-14-031

Originating Document: APP-FSAR-GLN-211

Title: Unit Auxiliary Transformers (UATs) and Reserve Auxiliary Transformers (RATs) Changes

#### **Activity Description:**

The size of the UATs and RATs is increased from 70/35/35 to 76/38/38 megavolt ampere (MVA), as described in UFSAR Chapter 8, Table 8.3.103, and Figure 8.3.1-1.

#### **Summary of Evaluation:**

The change to the size of the UATs and RATs does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the exvessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-14-037

AD/Screening: LCE-14-033

**Originating Document:** APP-FSAR-GLN-428

Title: Polar Crane Girder Fabrication Hole Blocking

#### **Activity Description:**

Fabrication holes in the top surface of the Polar Crane Girder (PCG) are blocked with  $\frac{1}{4}$ " thick plates and seal welds. UFSAR Figure 3.8.2-1 is changed to eliminate the fabrication holes.

#### **Summary of Evaluation:**

The change to block the polar crane girder fabrication holes does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-059** 

AD/Screening: LCE-14-034

**Originating Document:** APP-FSAR-GLN-189

Title: Electrical Penetration Number Change

#### **Activity Description:**

UFSAR Chapter 9, Table 9A-2, Safe Shutdown Components, electrical penetration number 16 for the Class 1E dc and UPS System (IDS) channel D is changed from EYP16Z to EY-P16Y.

## **Summary of Evaluation:**

The change to the IDS channel D electrical penetration number does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-14-038

AD/Screening: LCE-14-035

**Originating Document:** APP-FSAR-GLN-155

Title: Concrete Containing Fly Ash in Counting Room and Laboratory Area

#### **Activity Description:**

UFSAR Section 12.3, Radiation Protection Design Features, is changed to permit the use of fly ash in the concrete used for the counting room and laboratory areas, subject to design limits on radiation levels.

## **Summary of Evaluation:**

The change to permit the use of concrete containing fly ash in the counting room and laboratory areas does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-14-003** 

AD/Screening: LCE-14-036

**Originating Document:** APP-FSAR-GLN-301

Title: Pressure Control for Electrodeionization (EDI) Reject Radiation Monitor

#### **Activity Description:**

The Steam Generator Blowdown System (BDS) is changed to add a pressure regulating valve on the EDI reject BDS line between the Radiation Monitor, BDS-RE011, takeoff and return taps. A description of the new pressure regulator valve is added to UFSAR Subsection 10.4.8 and depicted on Figure 10.4.8-1.

## **Summary of Evaluation:**

The change to add a pressure regulating valve in the BDS does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-14-028** 

AD/Screening: LCE-14-037

**Originating Document:** APP-FSAR-GLN-252

Title: Gland Steam Seal System (GSS) Changes

### **Activity Description:**

The GSS is changed to add a stem drain line to each of the four Main Stop Valves (MSVs) in the Main Steam System (MSS). The new drain lines are routed to the Gland Steam Condenser, and a second drain line off each MSV to the low pressure turbine is eliminated. Gland Steam Exhauster Suction Isolation Valves, APP-GSS-PL-V015A and APP-GSS-PLV015B, are changed from motor actuated valves to manually operated valves. Additional clarifications are made to UFSAR Figure 10.4.3-1 to depict flow direction arrows on gland seal steam from the high pressure turbine exiting rather than entering the turbine glands and drainage from the GSS exhauster casing to the main condenser in lieu of the condenser recovery tank.

## **Summary of Evaluation:**

The change to the GSS drains and valves does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the exvessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-14-031** 

AD/Screening: LCE-14-038

Originating Document: APP-FSAR-GLN-244

Title: General Arrangement and Layout Figures Changes

### **Activity Description:**

UFSAR Figures are being revised to incorporate the following design changes: revising removable fuel handling area hatch configurations, adding details for the personnel basket for air inlets on the shield building, addition of interior platforms and vertical access in the CVS Room, adding fire doors to the Auxiliary Building Containment Isolation Valve Room and Primary Sampling System Room, deleting doors and an associated interconnection access way in the Auxiliary Building, and addition of a ladder from the Fuel Handling Area Tool Room to the Fuel Handling Area Steam Relief Panel.

# **Summary of Evaluation:**

The changes involve modifications and changes to non-critical building details. The activity/departure did not involve a Tier 2\*, Technical Specifications or plant-specific Tier 1 information (including ITAAC) change. The departure did not affect any design function, involve a procedure or method of control that affects the performance of a design function, involve a method of evaluation described in the UFSAR, involve a test or experiment, or involve a design feature credited in the ex-vessel severe accident assessment. The 10 CFR 50.59/10 CFR 52 Appendix D Section VIII review determined that prior NRC approval is not required.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-099** 

AD/Screening: LCE-14-039

Originating Document: APP-FSAR-GLN-314

Title: Changes to CA55 Floor

### **Activity Description:**

The location of the Reactor Vessel (RV) Head Stand on the operating deck of CA55 is moved 2'-0" east, and 0'-4" north. The vertical columns which support the Head Stand are changed from circular to rectangular. Hydrogen Igniter #9 and #10 inside containment are relocated. The personnel hatches on El. 135'3" and 161'-0" are moved north by 3'-9" and west by 0'-4".

#### **Summary of Evaluation:**

The change to the layout of equipment and structures associated with CA55 does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-14-040

AD/Screening: LCE-14-040

**Originating Document: APP-FSAR-GLN-387** 

Title: Containment Penetrations

#### **Activity Description:**

Two new mechanical containment penetration details are added to the design, as described in UFSAR Subsection 3.8.2.1.5 and depicted on UFSAR Figure 3.8.2-4. UFSAR Subsection 3.8.2.1.5 is changed to describe these four penetration types as representative rather than typical. Detailed dimensions of the insert plates for the main steam and feedwater penetrations, as shown in Figure 3.8.2-4, are changed, and some dimensions are added or removed.

### **Summary of Evaluation:**

The change to the containment penetration details provided in the UFSAR does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-077** 

AD/Screening: LCE-14-042

**Originating Document:** APP-FSAR-GLN-273

Title: In-containment Refueling Water Storage Tank (IRWST) Screens

### **Activity Description:**

UFSAR Chapter 6, Figure 6.3-3, Passive Safety Injection, is revised to clarify there are three IRWST screens rather than two.

### **Summary of Evaluation:**

The change to clarify there are three IRWST screens does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-016** 

AD/Screening: LCE-14-043

Originating Document: APP-FSAR-GLN-034

Title: Pressure Relief Design for the Standby Diesel Fuel Oil System (DOS) Day Tanks

#### **Activity Description:**

This change upgrades the nonsafety-related diesel generator day tanks to be ASME code stamped, and also to be provided with overpressure relief valves as required by the ASME code to address a postulated tank overfill condition. The DOS isolation valve in the fuel oil return line from each diesel fuel oil day tank is deleted. A pressure relief valve is installed at the top of each diesel fuel oil day tank. The discharge piping for the relief valve is connected to the existing overflow line.

### **Summary of Evaluation:**

The change to the pressure relief design of the diesel fuel oil day tanks does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-093

AD/Screening: LCE-14-045

**Originating Document:** APP-FSAR-GLN-231

Title: Turbine Building Ventilation System (VTS) Clarification

### **Activity Description:**

UFSAR Tier 2 Section 11.3.3.3 is modified to remove redundant and inconsistent wording regarding turbine building ventilation releases. Previous wording suggested that the Turbine Building Ventilation System (VTS) has a release point via the turbine building exhaust vent. However, "Ventilation releases", are simply a subset of "turbine building vents" and so the statement is grammatically redundant and inconsistent with other portions of the UFSAR. Removal of the term "turbine building ventilation releases" from section 11.3.3.3 resolves this inconsistency.

### **Summary of Evaluation:**

The change to remove the statement "turbine building ventilation releases" from UFSAR section 11.3.3.3 does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-058** 

AD/Screening: LCE-14-046

Originating Document: APP-FSAR-GLN-154

Title: Fire Protection System (FPS) Deluge Valves Inside Containment

### **Activity Description:**

A new line is added to UFSAR Tier 2 Table 9.5.1-3, Exceptions To NFPA Standard Requirements, that takes exception to NFPA 15 Section 5.7.2.1 and describes how the deluge motor operated valve (MOV) meets its intent.

The change also replaces the pilot operated deluge valves with motor operated valves that will not spuriously open when the containment isolation valve for FPS is opened after being isolated.

# **Summary of Evaluation:**

The change to the requirements for the deluge valves inside containment does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-053

AD/Screening: LCE-14-047

**Originating Document:** APP-FSAR-GLN-070

Title: Heater Drain System Sample Point Locations

#### **Activity Description:**

Secondary Sampling System (SSS) sampling locations that monitor water quality within the Heater Drain System (HDS) are changed, as described in UFSAR Table 9.3.4-2, Secondary Sampling System (Selective Measurements).

## **Summary of Evaluation:**

The change to the sampling locations within the HDS does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

Revision Notice: RN-13-081

AD/Screening: LCE-14-049

**Originating Document:** APP-FSAR-GLN-071

Title: Potable Water System (PWS) and Sanitary Drainage System (SDS) Class Changes

### **Activity Description:**

UFSAR Table 3.2-3 is changed to provide additional code classes for the PWS and SDS, consistent with design documents, and the Piping and Instrumentation Diagram (P&ID) Legend, Figure 1.7-2, is changed to update the information in the Line and Valve / Damper Specification Table.

#### **Summary of Evaluation:**

The change to add additional code classes for the PWS and SDS and to update the P&ID legend does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-14-045** 

AD/Screening: LCE-14-050

Originating Document: APP-FSAR-GLN-460

Title: Personnel Airlock Clarifications

#### **Activity Description:**

UFSAR Figure 3.8.2-3 is changed to clarify the minimum plate thickness and diameter of the insert plate of the personnel airlock.

# **Summary of Evaluation:**

The change to UFSAR Figure 3.8.2-3 depicting the personnel airlock dimensions does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-14-048

AD/Screening: LCE-14-052

Originating Document: APP-FSAR-GLN-442

Title: Passive Containment Cooling System (PCS) Air Baffle Changes

### **Activity Description:**

The PCS air baffle design is changed, as depicted in UFSAR Figure 3.8.4-1 and described in Subsection 3.8.4.1.3. Design changes include changing the air baffle panel material from galvanized steel to aluminum, increasing the thickness of the u-supports, modifying the flow guides, eliminating the clear observation panels, and changing the seal design from a sliding metal plate to a flexible seal.

### **Summary of Evaluation:**

The change to the PCS air baffle design does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-14-051** 

AD/Screening: LCE-14-053

Originating Document: 132177-S-C-00011-EVAL

Title: Nuclear Island Basemat Elevation for V.C. Summer Units 2 and 3.

### **Activity Description:**

This activity relates to the elevation of the Nuclear Island (NI) basemat for V.C. Summer Units 2 and 3. Following first nuclear concrete pour of both VC Summer Units 2 and 3, the NI basemats were measured to confirm the final as-built elevation as stated in the UFSAR. However, certain sections of the UFSAR state an elevation for the NI basemats that is different from that which were confirmed by detailed measurements. The starting elevation of the NI basemat foundation for each unit has been established at 360.5 feet in design and construction drawings. However, UFSAR section 2.5.4.2.2 and Table 2.5.4-217 show the basemat founded at 360 feet. This revision resolves this inconsistency.

# **Summary of Evaluation:**

The revision to the UFSAR to correct inconsistencies regarding the elevation of the nuclear island basemat does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-049** 

AD/Screening: LCE-14-055

**Originating Document:** APP-FSAR-GLN-195

Title: Condenser Air Removal System Design Update

### **Activity Description:**

The condenser air removal system (CMS) as described in UFSAR Chapter 10, Subsection 10.4.2, Main Condenser Evacuation System, is changed. The shell and tube heat exchanger for cooling the seal water is changed to a plate and frame heat exchanger; the silencer on the air discharge from the air/water separator is eliminated; and the source of makeup water from the Condensate System (CDS) to the CMS for establishing and maintaining level in the CMS moisture separator tanks is changed to the Demineralized Water Transfer and Storage System (DWS).

# **Summary of Evaluation:**

The change to the CMS design does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-14-009** 

AD/Screening: LCE-14-056

Originating Document: APP-FSAR-GLN-289

Title: Steam Generator Blowdown System (BDS) Electrodeionization (EDI) Demineralizing Unit

Changes

# **Activity Description:**

The design of EDI demineralizing units, as described in UFSAR Subsection 10.4.8, is changed to eliminate the brine system and associated brine pump, add a drain for maintenance, and to clarify that the units are not self-cleaning.

#### **Summary of Evaluation:**

The change to the design of EDI demineralizing units does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-079** 

AD/Screening: LCE-14-057

Originating Document: APP-FSAR-GLN-264

Title: Add Solenoid Valve Information in UFSAR Chapter 3 for Primary Sampling System (PSS)

Valves PSS-PL-V001A and PSS-PL-V001B

### **Activity Description:**

Solenoid valve information is added to UFSAR Chapter 3 for PSS-PL-V001A/B, consistent with existing level of detail for other safety-related valves.

## **Summary of Evaluation:**

The change to UFSAR Chapter 3 to add additional information on the solenoid valves for PSS-PLV001A/B does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-057

AD/Screening: LCE-14-058

Originating Document: APP-FSAR-GLN-104

Title: Turbine Building Egress and Restroom Changes

### **Activity Description:**

An additional door is added from the Turbine Building secondary sample room to exit into the stairwell tower, and the direction of opening of the exit door leaving the main portion of the turbine building and going into the secondary sample room on elevation 120'-6" is changed to open in the direction of egress. In addition, the women's and men's restrooms are deleted from elevation 170'-0" in the turbine building.

### **Summary of Evaluation:**

The change to the Turbine Building and secondary sample room egress and the deletion of the restrooms does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-070** 

AD/Screening: LCE-14-059

**Originating Document:** APP-FSAR-GLN-135

**Title:** Changes to Steam Generator and Pressurizer Support Figures

### **Activity Description:**

UFSAR Figure 3.8.3-5, Steam Generator Supports, and Figure 3.8.3-6, Pressurizer Supports, is changed to remove unnecessary dimensions, elevations, drawing references and other details.

### **Summary of Evaluation:**

The change to remove details on the steam generator and pressurize support figures in the UFSAR does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-14-041** 

AD/Screening: LCE-14-061

Originating Document: 132177-S-C-00013-EVAL

**Title:** Changes to the RWS supply to the FPS storage tanks

#### **Activity Description:**

This change modified the RWS supply to the FPS storage tanks from two dedicated lines to one fill line that feeds both tanks. As a result, UFSAR Figure 9.2-201 was revised to incorporate this change.

## **Summary of Evaluation:**

The modification is specific to the piping only. No changes were made to the FPS valve layout, operational strategies or procedure for filling the tanks. The change does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice: RN-13-015** 

AD/Screening: LCE-14-063

**Originating Document:** APP-FSAR-GLN-130

Title: Passive Containment Cooling System (PCS) Piping & Instrument Diagram (P&ID) Update

### **Activity Description:**

The UFSAR P&ID for the PCS is revised to depict the PCS annulus drains from the sides of the annulus and discharging to the Waste Water System (WWS); change flow instrumentation tagging to reflect local indication; indicate the number of heating elements in the Passive Containment Cooling Ancillary Water Storage Tank (PCCAWST) to accomplish the design heating function, and eliminate a piping class break at the Passive Containment Cooling Water Storage Tank (PCCWST) water makeup line entry.

### **Summary of Evaluation:**

The changes to the UFSAR to clarify the PCS P&ID do not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the exvessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-13-078

AD/Screening: LCE-14-067

**Originating Document:** APP-FSAR-GLN-280

**Title:** Floor Opening For Room 40341

### **Activity Description:**

UFSAR Figure 3.7.2-19 is changed to depict the floor opening in the southeast corner of the Annex Building (Room 40341) to be at elevation 107'-2" rather than elevation 100'-0", consistent with other figures within the UFSAR.

## **Summary of Evaluation:**

The change to correct the location of the floor opening for Room 40341 does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.

V.C. Summer Nuclear Station Units 2 and 3

Departure Report: January 28, 2014 through July 28, 2014

**Revision Notice:** RN-14-027

AD/Screening: LCE-14-071

Originating Document: APP-FSAR-GLN-234

Title: Shielding Provisions in the Turbine Building

### **Activity Description:**

A shielding box is added around the Steam Generator Blowdown sample panel resin columns of the Secondary Sampling System (SSS), and space provisions are added to allow temporary shielding to be installed around the Steam Generator Blowdown System (BDS) filters and electrodeionization (EDI) units.

## **Summary of Evaluation:**

The change to add shielding and shielding provisions in the Turbine Building does not result in a modification, addition to, or removal of a structure, system, or component (SSC) such that a design function is adversely affected, has no impact on plant operating procedures or a method of control that adversely affects a design function, does not result in an adverse change to a method of evaluation or use of an alternate method of evaluation, does not represent tests or experiments outside the reference bounds of the design basis, and does not alter the assumptions or results of the ex-vessel severe accident assessment.