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U. S. Nuclear Regulatory Commission
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Calvert Cliffs Nuclear Power Plant
Unit Nos. 1 & 2; Docket Nos. 50-317 & 50-318
Facility Operating License Nos. DPR-53 and DPR-69
Independent Spent Fuel Storage Installation, Docket No. 72-8

Subject: Annual Radioactive Effluent Release Report

References: (a) Calvert Cliffs Unit Nos. 1 and 2 Technical Specification 5.6.3
(b) Calvert Cliffs ISFSI Technical Specification 6.3

As required by References (a) and (b), Enclosure (1) is provided. Meteorological data is kept in an onsite file and is available upon request.

This letter contains no regulatory commitments.

Should you have questions regarding this matter, please contact Mr. Douglas E. Lauver at (410) 495-5219.

Respectfully,

Mark D. Flaherty
Plant Manager

MDF/PSF/bjd

Enclosure: Annual Radioactive Effluent Release Report for Calvert Cliffs Nuclear Power Plant and Independent Spent Fuel Storage Installation -2013

cc: W. M. Dean, NRC
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ENCLOSURE (1)

**ANNUAL RADIOACTIVE EFFLUENT RELEASE ANNUAL REPORT FOR
CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT
SPENT FUEL STORAGE INSTALLATION - 2013**

**This report covers the period January 1, 2013 to December 31, 2013
for Calvert Cliffs Nuclear Power Plant.**

**This report covers the period June 1, 2013 to May 31, 2014 for the
Independent Spent Fuel Storage Installation.**

ENCLOSURE (1)

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

Facility - Calvert Cliffs Nuclear Power Plant and Independent Spent Fuel Storage Installation
Licensee – Calvert Cliffs Nuclear Power Plant, LLC

This report covers the period January 1, 2013 to December 31, 2013 for Calvert Cliffs Nuclear Power Plant.

This report covers the period June 1, 2013 to May 31, 2014 for the Independent Spent Fuel Storage Installation.

I. REGULATORY LIMITS

A. Fission and Activation Gases

1. The instantaneous release rate of noble gases in gaseous effluents shall not result in a site boundary dose rate greater than 500 mrem/year to the whole body or greater than 3000 mrem/year to the skin (Offsite Dose Calculation Manual (ODCM) Rev. 0802, 3.11.2.1).
2. Gaseous Radwaste Treatment System and the Ventilation Exhaust Treatment System shall be used to reduce gaseous emissions when the calculated gamma-air dose due to gaseous effluents exceeds 1.20 mRad or the calculated beta-air dose due to gaseous effluents exceeds 2.4 mRad at the site boundary in a 92 day period (ODCM 3.11.2.4).
3. The air dose at the site boundary due to noble gases released in gaseous effluents shall not exceed (ODCM 3.11.2.2):
 - 10 mRad/qtr, gamma-air
 - 20 mRad/qtr, beta-air
 - 20 mRad/year, gamma-air
 - 40 mRad/year, beta-air
4. All of the above parameters are calculated according to the methodology specified in the ODCM.

B. Iodines and Particulates with Half Lives Greater than Eight Days

1. The instantaneous release rate of iodines and particulates in gaseous effluents shall not result in a site boundary dose-rate in excess of 1500 mrem/year to any organ (ODCM 3.11.2.1).
2. The Gaseous Radwaste Treatment System and the Ventilation Exhaust Treatment System shall be used to reduce radioactive materials in gaseous effluents when calculated doses exceed 1.8 mrem to any organ in a 92 day period at or beyond the site boundary (ODCM 3.11.2.4).
3. The dose to a member of the public at or beyond the site boundary from iodine-131 and particulates with half-lives greater than eight days in gaseous effluents shall not exceed (ODCM 3.11.2.3):
 - 15 mrem/qtr, any organ
 - 30 mrem/year, any organ

ENCLOSURE (1)

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

less than 0.1% of the above limits as a result of burning contaminated oil.

4. All of the above parameters are calculated according to the methodology specified in the ODCM.

C. Liquid Effluents

1. The concentrations of radionuclides in liquid effluents from the plant shall not exceed the values specified in 10 CFR Part 20, Appendix B, Table II, Column 2 for unrestricted areas (ODCM 3.11.1.1).
2. The liquid radwaste treatment system shall be used to reduce the concentration of radionuclides in liquid effluents from the plant when the calculated dose to unrestricted areas exceeds 0.36 mrem to the whole body, or 1.20 mrem to any organ in a 92 day period (ODCM 3.11.1.3).
3. The dose to a member of the public in unrestricted areas shall not exceed (ODCM 3.11.1.2):
 - 3 mrem/qtr, total body
 - 10 mrem/qtr, any organ
 - 6 mrem/year, total body
 - 20 mrem/year, any organ
4. All liquid dose parameters are calculated according to the methodology specified in the ODCM.

II. MAXIMUM PERMISSIBLE CONCENTRATIONS

A. Fission and Activation Gases

Prior to the batch release of gaseous effluents, a sample of the source is collected and analyzed by gamma spectroscopy for the principal gamma emitting radionuclides. The identified radionuclide concentrations are evaluated and an acceptable release rate is determined to ensure that the dose rate limits of ODCM 3.11.2.1 are not exceeded.

B. Iodines and Particulates with Half Lives Greater than Eight Days

Compliance with the dose rate limitations for iodines and particulates is demonstrated by analysis of the charcoal and particulate samples of the station main vents. The charcoal samples are analyzed by gamma spectroscopy for quantification of radioiodine. The particulate samples are analyzed by gamma spectroscopy for quantification of particulate radioactive material. All of the above parameters are calculated according to the methodology specified in the ODCM.

C. Liquid Effluents

The Maximum Permissible Concentrations (MPCs) used for radioactive materials released in liquid effluents are in accordance with ODCM 3.11.1.1 and the values from 10 CFR Part 20, Appendix B, Table II, Column 2 including applicable table

ENCLOSURE (1)

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

notes. In all cases, the more restrictive (lower) MPC found for each radionuclide is used regardless of solubility.

III. TECHNICAL SPECIFICATION REPORTING REQUIREMENTS

A. Calvert Cliffs Nuclear Power Plant (CCNPP), Technical Specification 5.6.3

1. 2013 Offsite Dose Due to Carbon-14

Carbon-14 releases from Calvert Cliffs were estimated using the EPRI Carbon-14 Source Term Calculator using specific reactor core data and reactor coolant chemistry to estimate the products of the activation reactions. The resulting estimate of Carbon-14 released from the Calvert Cliffs Site is 14.70 Curies.

Dose due to Carbon-14 in gaseous effluents was calculated using the following conditions:

- a. C-14 released to the atmosphere: 7.87 Curies of C-14 from Unit 1, 6.83 curies from Unit 2.
- b. Release was consistent throughout the year.
- c. 10% of the C-14 was in the chemical form of CO₂, which is the only dose contributor. The bulk of C-14 was in the chemical form of methane (CH₄) which would exhibit high upward velocity due to its low density relative to air. In addition, CH₄ does not have an uptake pathway for humans.
- d. Meteorological dispersion factor (X/Q) at the site boundary to the hypothetical maximally exposed member of the public is 1.87E-07 sec/m³.
- e. Dose calculations and dose factors are from Regulatory Guide 1.109 methodology.
- f. Pathways considered were inhalation and vegetation ingestion.
- g. Critical receptor is child at the site boundary in the SE direction.

Maximum dose values due to C-14 in gaseous effluents in 2013

Organ	Age	Pathway	mrem/yr	1 st Quarter	2 nd Quarter	3 rd Quarter	4 th Quarter
Bone	Child	Inhalation	3.13E-03	7.83E-4	7.83E-4	7.83E-4	7.83E-4
T. Body/Other	Child	Inhalation	5.86E-04	1.47E-4	1.47E-4	1.47E-4	1.47E-4
Bone	Child	Vegetation	3.05E-01	7.63E-02	7.63E-02	7.63E-02	7.63E-02
T. Body/Other	Child	Vegetation	6.11E-02	1.53E-02	1.53E-02	1.53E-02	1.53E-02

ENCLOSURE (1)

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

2. 2013 Dose Assessment Summary

	Actual Value	Percent of ODCM limit	ODCM Limit
Liquid Waste:			
Maximum Annual Organ Dose (mrem) ¹	0.001	0.01%	20
Maximum Annual Whole Body Dose (mrem)	0.001	0.02%	6
Gaseous Waste:			
Noble Gases:			
Maximum Quarterly Gamma Air Dose (mRad)	0.0008	0.01%	10
Maximum Quarterly Beta Air Dose (mRad)	0.006	0.03%	20
Iodines and Particulates:			
Maximum Annual Organ Dose (mrem) ²	0.02	0.07%	30

¹ The controlling pathway was the fish and shellfish pathway with adult as the controlling age group and the liver representing the organ with the highest calculated dose during the calendar year of 2013.

² The controlling pathway was the infant-thyroid pathway representing the organ with the highest calculated dose during the calendar year of 2013. There is currently no milk pathway.

3. 40 CFR 190 Total Dose Compliance

Based upon the calendar year 2013 and the ODCM calculations, the maximum exposed individual would receive 0.02% of the allowable dose. During the calendar year 2013, there were no on-site sources of direct radiation that would have contributed to a significant or measurable off-site dose. The direct radiation contribution is measured by both on-site and off-site thermoluminescent dosimeters (TLDs). The results of these measurements did not indicate any statistical increase in the off-site radiation doses attributable to on-site sources. Therefore, no increase in the calculated offsite dose is attributed to the direct exposure from on-site sources. A more detailed evaluation may be found in the Annual Radiological Environmental Operating Report.

4. Solid Waste Report Requirements

During 2013, the types of radioactive solid waste shipped from Calvert Cliffs were dry compressible waste, spent resins, and cartridge filters which were shipped in either High Integrity Containers (HICs) within NRC approved casks, Sea/Land containers, or steel boxes. Appendix A provides a detailed breakdown of the waste shipments for 2013 per Technical Specification 5.6.3. At CCNPP, methods of waste and materials segregation are used to reduce the volume of solid waste shipped offsite for processing, volume reduction, and burial.

ENCLOSURE (1)

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

5. Offsite Dose Calculation Manual (ODCM) and Process Control Program (PCP) Changes

The ODCM and PCP were not revised in 2013.

B. Radioactive Effluent Monitoring Instrumentation

1. The Unit 2 Steam Generator Blowdown Radiation Monitor 2-RE-4014 was inoperable during much of the calendar year 2013 due to component malfunction. The components were repaired and the Radiation Monitor was returned to service in 2014.

During the period the Radiation Monitor was inoperable, compensatory actions were performed in accordance with the ODCM requirements.

2. The Waste Gas Decay Tank Discharge Monitor 0-RE-2191 was declared inoperable for greater than 30 days as the criteria and methodology for testing the monitor did not meet the ODCM requirements.

During the period the Radiation Monitor was inoperable, compensatory actions were performed in accordance with the ODCM requirements.

C. Independent Spent Fuel Storage Installation (ISFSI), ISFSI Technical Specification 6.3

Zero (0) casks of spent fuel were transferred to the ISFSI during the reporting period. No quantity of radionuclides was released to the environment during the ISFSI operation in 2013. Additional information regarding the ISFSI radiological environmental monitoring program is included in the Annual Radiological Environmental Operation Report.

IV. AVERAGE ENERGY

Not Applicable.

V. MEASUREMENTS AND APPROXIMATIONS AND TOTAL RADIOACTIVITY

A. Fission and Activation Gases

1. Batch Releases

Prior to each batch release of gas from a pressurized waste gas decay tank or containment, a sample is collected and analyzed by gamma spectroscopy using a germanium detector for the principal gamma emitting noble gas radionuclides. The total activity released is based on the pressure/volume relationship (gas laws). The Plant Vent Stack Radiation Monitor typically monitors containment releases, and the values from the radiation monitor may be used to assist in the calculation of activity discharged from containment during venting. Carbon-14 is estimated per section III.A.1.

ENCLOSURE (1)

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

2. Continuous Releases

A gas sample is collected at least weekly from the main vents and analyzed by gamma spectroscopy using a germanium detector for the principal gamma emitting noble gas radionuclides. The total activity released for the week is based on the total sample activity decay corrected to the sample time multiplied by the main vent flow for the week. The Plant Vent Stack Radiation Monitor continuously measures routine plant vent stack releases, per design, and the values from the radiation monitor may be used to assist in the calculation of activity discharged in routine plant vent stack discharges.

During each containment purge, a gas sample is collected and analyzed by gamma spectroscopy using a germanium detector to determine the concentration of principal gamma emitting noble gas radionuclides inside containment. Total activity released during a containment purge is based on continuous radiation monitor responses, grab samples, and purge fan flow rate.

A monthly composite sample is collected from the main vents and analyzed by liquid scintillation for tritium. The total tritium release for the month is based on the sample analysis and the main vent flow.

Carbon-14 is estimated per section III.A.1.

B. Iodine and Particulates

1. Batch Releases

The total activities of radioiodines and particulates released from pressurized waste gas decay tanks, containment purges, and containment vents are accounted for by the continuous release methodology discussed in section V.B.2.

2. Continuous Releases

During the release of gas from the main vents, samples of iodines and particulates are collected using a charcoal and particulate filter, respectively. The filters are removed weekly (or more often) and are analyzed by gamma spectroscopy using a germanium detector for significant gamma emitting radionuclides. The total activity released for the week is based on the total sample activity decay corrected to the midpoint of the sample period multiplied by the main vent flow for the week. A plate-out correction factor is applied to the results to account for the amount of iodine lost in the sample lines prior to sample collection. The weekly particulate filters are then combined to form monthly and quarterly composites for the gross alpha, strontium-89, and strontium-90 analyses. Iron-55 is analyzed twice per year.

ENCLOSURE (1)

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

C. Liquid Effluents

1. Batch Releases

Prior to the release of liquid from a waste tank, a sample is collected and analyzed by gamma spectroscopy for the principal gamma emitting radionuclides. To demonstrate compliance with the concentration requirements addressed in Section I.C.1 above, the measured radionuclide concentrations are compared with the allowable MPCs; dilution in the discharge conduit is considered, and an allowable release rate is verified.

The total activity released in each batch is determined by multiplying the volume released by the concentration of each radionuclide. The actual volume released is based on the difference in tank levels prior to and after the release. A proportional composite sample is also withdrawn from each release, and this is used to prepare monthly tritium and gross alpha, iron-55, nickel-63, strontium-89, and strontium-90 samples for analysis are prepared quarterly for offsite analysis.

Batch discharges of secondary (normally uncontaminated) waste streams are also monitored for radioactivity. No activity is normally detected in these secondary waste streams (excluding tritium).

There were no major changes to the liquid radwaste system in calendar year 2013.

2. Continuous Releases

To account for activity from continuous releases, a sample is collected and analyzed by gamma spectroscopy for the principal gamma emitting radionuclides. The measured radionuclide concentrations are compared with the allowable MPC concentrations in the discharge conduit, and an allowable release rate is verified.

When steam generator blowdown is discharged to the circulating water conduits, it is sampled and gamma isotopic analysis is performed at a minimum once per week. These results are multiplied by the actual quantity of blowdown to determine the total activity released. The weekly sample is also used to prepare monthly composites for tritium analysis.

During the monitoring for primary-to-secondary leakage low levels of tritium have been detected in the Turbine Building sumps. This water is sampled and analyzed for principal gamma emitting radionuclides weekly and composited. The composite sample is analyzed at least monthly for tritium. The results are multiplied by the actual quantity of liquid released to determine the total activity released.

ENCLOSURE (1)

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

D. Estimation of Total Error

Total error for all releases was estimated using, as a minimum, the random counting error associated with typical releases. In addition to this random error, the following systematic errors were also examined:

1. Liquid
 - a. Error in volume of liquid released prior to dilution during batch releases.
 - b. Error in volume of liquid released via steam generator blowdown.
 - c. Error in amount of dilution water used during the reporting period.
2. Gases
 - a. Error in main vent release flow.
 - b. Error in sample flow rate.
 - c. Error in containment purge release flow.
 - d. Error in gas decay tank pressure.

Where errors could be estimated they are usually considered additive.

E. Reporting and Recordkeeping for Decommissioning

In accordance with 10 CFR 50.75.g, each licensee shall keep records of information important to the safe and effective decommissioning of the facility in an identified location until the license is terminated by the Commission. If records of relevant information are kept for other purposes, reference to these records and their locations may be used. Information the Commission considers important to decommissioning consists of records of spills or other unusual occurrences involving the spread of contamination in and around the facility, equipment, or site. These records may be limited to instances when significant contamination remains after any cleanup procedures or when there is reasonable likelihood that contaminants may have spread to inaccessible areas as in the case of possible seepage into porous materials such as concrete. These records must include any known information on identification of involved nuclides, quantities, forms, and concentrations.

To assist in the decommissioning, and to provide early and advance detection of any unmonitored releases of radioactive material from the site, groundwater is routinely sampled. These groundwater samples are analyzed for gamma and tritium activity (see Tables below). Sample size and/or count times are adjusted to achieve analytical sensitivities better than the environmental LLDs for gamma emitters (listed in ODCM Table 4.12-1). Established LLD limits for tritium are approximately 350 pCi/l for tritium.

Groundwater samples were collected from nine on-site piezometer tubes throughout 2013. A piezometer tube is a shallow monitoring well which allows access to groundwater at a depth of approximately 40 feet beneath the site. Eight additional wells were installed in 2013 and were included in fourth quarter monitoring. Of the piezometer tubes sampled, only #11 Piezometer Tube showed any plant related activity. This activity was previously identified and evaluated in December of 2005.

ENCLOSURE (1)

**ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS
NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION -
2013**

The activity consists of tritium originating from normal radwaste discharges and was previously reported in the Annual Radioactive Effluent Release Reports. The tritium contamination is contained on site. No drinking water has been affected; the groundwater at this location does not impact any drinking water pathway.

The 2013 analysis result for tritium and gamma are shown in the following tables.

ENCLOSURE (1)

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

**Concentration of Tritium in Groundwater
(Results in units of pCi/L +/- 2σ)**

Sample Date	Piezometer Tube																
	11	12	13	15	18	19	20	21	22	23	24	25	26	27	28	29	30
03/02/13	464 +/- 197	<319	<314	<314	<318	<318	<327	<327	--	--	--	--	--	--	--	--	--
03/14/13	--	--	--	--	--	--	--	--	<328	--	--	--	--	--	--	--	--
03/22/13	--	--	--	--	--	--	--	--	<322	--	--	--	--	--	--	--	--
04/21/13	--	--	--	--	<327	<327	<332	<319	<325	--	--	--	--	--	--	--	--
06/25/13	952 +/- 211	<336	<336	<332	--	--	--	--	--	--	--	--	--	--	--	--	--
08/05/13	1030 +/- 209	<342	<342	<332	<332	<332	<332	<340	<340	--	--	--	--	--	--	--	--
11/10/13	1280 +/- 202	<314	<313	<313	<315	<315	<321	<321	<317	--	--	--	--	--	--	--	--
12/7/13	--	--	--	--	--	--	--	--	--	<305	--	<306	<307	<305	<307	<306	<304
12/24/13	--	--	--	--	--	--	--	--	--	--	<313	--	--	--	--	--	--

ENCLOSURE (1)

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

**Gross Concentration of Gamma Emitters
(Results in units of pCi/L +/- 2σ)**

Sample Date	Piezometer Tube																
	11	12	13	15	18	19	20	21	22	23	24	25	26	27	28	29	30
03/02/13	*	*	*	*	*	*	*	*	*	--	--	--	--	--	--	--	--
04/21/13	--	--	--	--	*	*	*	*	*	--	--	--	--	--	--	--	--
06/25/13	*	*	*	*	--	--	--	--	--	--	--	--	--	--	--	--	--
08/05/13	*	*	*	*	*	*	*	*	*	--	--	--	--	--	--	--	--
11/10/13	*	*	*	*	*	*	*	*	*	--	--	--	--	--	--	--	--
12/7/13	--	--	--	--	--	--	--	--	--	*	--	*	*	*	*	*	*
12/24/13	--	--	--	--	--	--	--	--	--	--	*	--	--	--	--	--	--

*All Non-Natural Gamma Emitters < MDA

ENCLOSURE (1)

**ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS
NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION -
2013**

VI. BATCH RELEASES

	<u>2013</u>			
	<u>1ST</u> <u>QUARTER</u>	<u>2ND</u> <u>QUARTER</u>	<u>3RD</u> <u>QUARTER</u>	<u>4TH</u> <u>QUARTER</u>
A. <u>Liquid</u>				
1. Number of batch releases	16	13	12	10
2. Total time period for batch releases (min)	6.33E+03	4.55E+03	4.82E+03	6.11E+03
3. Maximum time period for a batch release (min)	6.55E+02	6.09E+02	7.10E+02	7.17E+02
4. Average time period for batch releases (min)	3.96E+02	3.50E+02	4.02E+02	6.11E+02
5. Minimum time period for a batch release (min)	2.00E+01	2.00E+01	1.80E+01	5.31E+02
6. Average stream flow during periods of effluent into a flowing stream (liters/min of dilution water)	4.45E+06	4.60E+06	4.57E+06	4.59E+06
B. <u>Gaseous</u>				
	<u>1ST</u> <u>QUARTER</u>	<u>2ND</u> <u>QUARTER</u>	<u>3RD</u> <u>QUARTER</u>	<u>4TH</u> <u>QUARTER</u>
1. Number of batch releases	24	24	26	18
2. Total time period for batch releases (min)	1.03E+04	9.45E+03	8.44E+03	6.45E+03
3. Maximum time period for a batch release (min)	2.36E+03	1.44E+03	7.45E+02	5.08E+02
4. Average time period for batch release (min)	4.29E+02	3.94E+02	3.25E+02	3.58E+02
5. Minimum time period for a batch release (min)	4.70E+01	4.00E+00	2.00E+00	1.52E+02

ENCLOSURE (1)

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT FOR CALVERT CLIFFS
NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION -
2013

VII. ABNORMAL RELEASES

	<u>2013</u>			
	<u>1ST</u> <u>QUARTER</u>	<u>2ND</u> <u>QUARTER</u>	<u>3RD</u> <u>QUARTER</u>	<u>4TH</u> <u>QUARTER</u>
A. <u>Liquid</u>				
1. Number of releases	- 0 -	- 0 -	- 0 -	- 0 -
2. Total activity released (Curies)	- 0 -	- 0 -	- 0 -	- 0 -
B. <u>Gaseous</u>				
1. Number of releases	- 0 -	- 0 -	- 0 -	- 0 -
2. Total activity releases (Curies)	- 0 -	- 0 -	- 0 -	- 0 -

APPENDIX A

ANNUAL EFFLUENT AND WASTE DISPOSAL REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

**TABLE 1A - REG GUIDE 1.21
GASEOUS EFFLUENTS - SUMMATION OF ALL RELEASES**

A. FISSION AND ACTIVATION GASES	UNITS	1ST QUARTER	2ND QUARTER	3RD QUARTER	4TH QUARTER	EST. TOTAL ERROR, %
1. Total Release	Ci	5.83E+01	2.99E+00	5.67E+00	1.47E+00	±1.20E+01
2. Average release rate for period	μCi/sec	7.39E+00	3.80E-01	7.19E-01	1.87E-01	
3. Percent of ODCM limit (1)	%	1.57E-02	2.35E-03	4.63E-03	3.13E-03	
4. Percent of ODCM limit (2)	%	8.20E-02	7.98E-02	1.93E-05	7.08E-03	
5. Percent of ODCM limit (3)	%	5.44E-03	5.88E-04	9.40E-04	8.58E-04	
6. Percent of ODCM limit (4)	%	2.77E-03	3.03E-03	3.43E-03	3.76E-03	
7. Percent of ODCM limit (5)	%	1.84E-02	2.23E-03	4.33E-03	1.00E-03	
8. Percent of ODCM limit (6)	%	9.21E-03	1.03E-02	1.24E-02	1.29E-02	
B. IODINES						
1. Total Iodine - 131	Ci	7.90E-04	2.56E-05	3.30E-05	2.81E-05	±6.50E+00
2. Average release rate for period	μCi/sec	1.00E-04	3.25E-06	4.18E-06	3.57E-06	
3. Percent of ODCM limit (7)	%	7.73E-03	2.39E-03	3.89E-03	3.23E-03	
4. Percent of ODCM limit (8)	%	1.19E-01	4.57E-03	7.39E-03	5.58E-03	
5. Percent of ODCM limit (9)	%	5.95E-02	7.43E-02	7.89E-02	8.07E-02	
C. PARTICULATES						
1. Particulates with half lives greater than 8 days	Ci	3.09E-06	(10)	(10)	(10)	±1.20E+01
2. Average release rate for period	μCi/sec	3.92E-07	(10)	(10)	(10)	
3. Percent of ODCM limit (7)	%	9.59E-02	(10)	(10)	(10)	
4. Percent of ODCM limit (8)	%	1.19E-01	(10)	(10)	(10)	
5. Percent of ODCM limit (9)	%	5.95E-02	(10)	(10)	(10)	
D. TRITIUM						
1. Total Release	Ci	2.25E+00	2.57E+00	3.62E+00	2.50E+00	±1.32E+01
2. Average release rate for period	μCi/sec	2.85E-01	3.26E-01	4.59E-01	3.18E-01	
D. GROSS ALPHA						
1. Total Release	Ci	3.83E-07	(10)	(10)	9.55E-07	±2.50E+01
2. Average release rate for period	μCi/sec	4.86E-08	(10)	(10)	1.21E-07	

APPENDIX A

ANNUAL EFFLUENT AND WASTE DISPOSAL REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT
SPENT FUEL STORAGE INSTALLATION - 2013

TABLE 1A - REG GUIDE 1.21
GASEOUS EFFLUENTS - SUMMATION OF ALL RELEASES

NOTES TO TABLE 1A

- (1) Percent of I.A.1 whole body dose rate limit (500 mrem/year)
- (2) Percent of I.A.1 skin dose rate limit (3000 mrem/year)
- (3) Percent of I.A.3 quarterly gamma-air dose limit (10 mRad)
- (4) Percent of I.A.3 yearly gamma-air dose limit (20 mRad)
- (5) Percent of I.A.3 quarterly beta-air dose limit (20 mRad)
- (6) Percent of I.A.3 yearly beta-air dose limit (40 mRad)
- (7) Percent of organ dose rate limit (1500 mrem/year)
- (8) Percent of I.B.3 quarterly organ dose limit (15 mrem)
- (9) Percent of I.B.3 yearly organ dose limit (30 mrem)
- (10) Less than minimum detectable activity which meets the lower limit of detection (LLD) requirements of ODCM Surveillance Requirement 4.11.2.1.2.

APPENDIX A

ANNUAL EFFLUENT AND WASTE DISPOSAL REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

**TABLE 1C - REG GUIDE 1.21
GASEOUS EFFLUENTS - GROUND LEVEL RELEASES**

		UNITS	CONTINUOUS MODE				BATCH MODE			
			1ST QUARTER	2ND QUARTER	3RD QUARTER	4TH QUARTER	1ST QUARTER	2ND QUARTER	3RD QUARTER	4TH QUARTER
1 FISSION AND ACTIVATION GASES										
Argon	-41	Ci	(2)	(2)	(2)	(2)	2.63E-02	3.39E-02	2.48E-02	2.41E-02
Krypton	-85	Ci	2.43E+01	(2)	1.33E-01	(2)	6.49E+00	2.85E+00	4.71E+00	7.26E-01
Krypton	-85m	Ci	(2)	(2)	(2)	(2)	(2)	5.79E-06	1.06E-05	(2)
Krypton	-87	Ci	(2)	(2)	(2)	(2)	(2)	2.75E-05	(2)	(2)
Krypton	-88	Ci	(2)	(2)	(2)	(2)	(2)	(2)	(2)	(2)
Xenon	-131m	Ci	(2)	(2)	(2)	(2)	7.57E-03	(2)	2.24E-04	(2)
Xenon	-133	Ci	2.62E+01	(2)	4.92E-01	2.96E-01	1.35E-01	1.34E-02	4.56E-02	4.10E-02
Xenon	-133m	Ci	1.30E-01	(2)	(2)	(2)	8.13E-05	(2)	(2)	(2)
Xenon	-135	Ci	9.39E-01	9.65E-02	2.60E-01	3.85E-01	1.80E-04	4.02E-04	7.43E-04	3.32E-04
Xenon	-135m	Ci	(2)	(2)	(2)	(2)	(2)	(2)	(2)	(2)
Xenon	-138	Ci	(2)	(2)	(2)	(2)	(2)	(2)	(2)	(2)
Total for Period		Ci	5.16+01	9.65E-02	8.85E-01	6.81E-01	6.66E+00	2.90E+00	4.78E+00	7.92E-01
2 IODINES										
Iodine	-131	Ci	7.90E-04	2.56E-05	3.30E-05	2.81E-05	(1)	(1)	(1)	(1)
Iodine	-132	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Iodine	-133	Ci	1.92E-04	4.62E-05	4.21E-04	2.89E-04	(1)	(1)	(1)	(1)
Iodine	-135	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Total for Period		Ci	9.82E-04	7.18E-05	4.54E-04	3.17E-04	(1)	(1)	(1)	(1)
3 PARTICULATES (half life > 8 days)										
Manganese	-54	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Iron	-55	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Iron	-59	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Cobalt	-58	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Cobalt	-60	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Zinc	-65	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Strontium	-89	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)

APPENDIX A

ANNUAL EFFLUENT AND WASTE DISPOSAL REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

**TABLE 1C - REG GUIDE 1.21
GASEOUS EFFLUENTS - GROUND LEVEL RELEASES**

		UNITS	CONTINUOUS MODE				BATCH MODE			
			1ST QUARTER	2ND QUARTER	3RD QUARTER	4TH QUARTER	1ST QUARTER	2ND QUARTER	3RD QUARTER	4TH QUARTER
Strontium	-90	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Molybdenum	-99	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Cesium	-134	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Cesium	-137	Ci	(2)	(2)	(2)	(2)	3.09E-06	(1)	(1)	(1)
Cerium	-141	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Cerium	-144	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	(1)
Total for period		Ci	(2)	(2)	(2)	(2)	3.09E-06	(1)	(1)	(1)
4 GROSS ALPHA RADIOACTIVITY										
.										
Gross Alpha		Ci	3.83E-07	(2)	(2)	9.55E-07	(2)	(2)	(2)	(2)
5 TRITIUM										
.										
Tritium		Ci	2.25E+00	2.57E+00	3.62E+00	2.50E+00	6.54E-07	(2)	(2)	(2)

NOTES TO TABLE 1C

- (1) Iodines and particulates in batch releases are accounted for with the main vent continuous samplers when the release is made through the plant main vent.
- (2) Less than minimum detectable activity which meets the LLD requirements of ODCM Surveillance Requirement 4.11.2.1.2.

APPENDIX A

ANNUAL EFFLUENT AND WASTE DISPOSAL REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

**TABLE 2A - REG GUIDE 1.21
LIQUID EFFLUENTS - SUMMATION OF ALL RELEASES**

	UNITS	1ST QUARTER	2ND QUARTER	3RD QUARTER	4TH QUARTER	EST. TOTAL ERROR, %
A. FISSION AND ACTIVATION PRODUCTS						
1. Total Release (not including tritium, gases, alpha)	Ci	3.46E-03	3.30E-03	4.29E-03	3.66E-03	±1.03E+01
2. Average diluted concentration during period	µCi/ml	1.62E-12	1.35E-12	1.74E-12	1.49E-12	
3. Percent of ODCM limit (1)	%	4.05E-03	2.91E-03	5.05E-03	5.99E-03	
4. Percent of ODCM limit (2)	%	2.55E-03	3.19E-03	4.87E-03	6.78E-03	
5. Percent of ODCM limit (3)	%	1.05E-02	5.60E-03	1.14E-02	1.39E-02	
6. Percent of ODCM limit (4)	%	6.09E-03	7.52E-03	1.31E-02	1.99E-02	
B. TRITIUM						
1. Total Release	Ci	1.90E+02	1.33E+02	3.05E+02	3.70E+02	±1.03E+01
2. Average diluted concentration during period	µCi/ml	8.88E-08	5.43E-08	1.24E-07	1.50E-07	
3. Percent of applicable limit (5)	%	2.96E-03	1.81E-03	4.13E-03	5.00E-03	
C. DISSOLVED AND ENTRAINED GASES						
1. Total Release	Ci	1.00E-02	1.51E-06	7.29E-04	1.27E-03	±1.03E+01
2. Average diluted concentration during period	µCi/ml	4.70E-12	6.18E-16	2.96E-13	5.18E-13	
D. GROSS ALPHA RADIOACTIVITY						
1. Total Release	Ci	(6)	(6)	(6)	(6)	N/A
E. VOLUME OF WASTE RELEASED (prior to dilution)						
1. Volume processed through radwaste system	liters	9.79E+07	1.01E+08	9.09E+07	1.23E+08	±1.30E+00
F. VOLUME OF DILUTION WATER USED DURING PERIOD (7)						
	liters	2.14E+12	2.45E+12	2.46E+12	2.46E+12	±1.64E+01

NOTES TO TABLE 2A

- (1) Percent of I.C.3 Quarterly Organ Dose Limit (10 mrem) to maximum exposed organ
- (2) Percent of I.C.3 Yearly Organ Dose Limit (20 mrem) to maximum exposed organ
- (3) Percent of I.C.3 Quarterly Whole Body Dose Limit (3 mrem)
- (4) Percent of I.C.3 Yearly Whole Body Dose Limit (6 mrem)

APPENDIX A

ANNUAL EFFLUENT AND WASTE DISPOSAL REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

- (5) Limit used is $3 \times 10^{-3} \mu\text{Ci/ml}$
- (6) Less than minimum detectable activity which meets the LLD requirements of ODCM Surveillance Requirement 4.11.1.1.1.
- (7) Includes dilution water used during continuous discharges.
- (8) Liquid releases are higher than normal during refueling outages due to noble gases released from failed fuel

APPENDIX A

**ANNUAL EFFLUENT AND WASTE DISPOSAL REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT
SPENT FUEL STORAGE INSTALLATION - 2013**

**TABLE 2B - REG GUIDE 1.21
LIQUID EFFLUENTS**

NUCLIDES RELEASED	Units	CONTINUOUS MODE				BATCH MODE			
		1ST QUARTER	2ND QUARTER	3RD QUARTER	4TH QUARTER	1ST QUARTER	2ND QUARTER	3RD QUARTER	4TH QUARTER
Beryllium - 7	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Sodium - 24	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Chromium - 51	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Manganese - 54	Ci	(1)	(1)	(1)	(1)	3.72E-06	(1)	(1)	(1)
Iron - 55	Ci	(2)	(2)	(2)	(2)	(1)	(1)	(1)	1.52E-03
Cobalt - 57	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Cobalt - 58	Ci	(1)	(1)	(1)	(1)	9.02E-04	2.08E-03	5.07E-04	(1)
Iron - 59	Ci	(1)	(1)	(1)	(1)	1.46E-06	(1)	(1)	(1)
Cobalt - 60	Ci	(1)	(1)	(1)	(1)	1.61E-04	1.08E-04	3.91E-05	3.65E-04
Nickel-63	Ci	(1)	(1)	(1)	(1)	1.50E-03	(1)	8.09E-04	1.51E-03
Zinc - 65	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Strontium - 89	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Strontium - 90	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Strontium - 91	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Strontium - 92	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Niobium - 95	Ci	(1)	(1)	(1)	(1)	4.77E-06	1.98E-06	(1)	(1)
Zirconium - 95	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Niobium - 97	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Zirconium - 97	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Molybdenum - 99	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Technetium - 99m	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Ruthenium - 103	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Rhodium - 105	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Ruthenium - 105	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Silver - 110m	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Tin - 113	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Tin - 117m	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Antimony - 122	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Antimony - 124	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Antimony - 125	Ci	(1)	(1)	(1)	(1)	1.22E-04	2.80E-04	1.33E-03	(1)

APPENDIX A

ANNUAL EFFLUENT AND WASTE DISPOSAL REPORT FOR CALVERT CLIFFS NUCLEAR POWER PLANT AND INDEPENDENT SPENT FUEL STORAGE INSTALLATION - 2013

**TABLE 2B - REG GUIDE 1.21
LIQUID EFFLUENTS**

NUCLIDES RELEASED	Units	CONTINUOUS MODE				BATCH MODE			
		1ST QUARTER	2ND QUARTER	3RD QUARTER	4TH QUARTER	1ST QUARTER	2ND QUARTER	3RD QUARTER	4TH QUARTER
Tellurium - 125m	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Tellurium - 132	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Iodine - 131	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Iodine - 132	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Iodine - 133	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Iodine - 135	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Cesium - 134	Ci	(1)	(1)	(1)	(1)	8.68E-05	1.14E-04	1.66E-04	(1)
Cesium - 136	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Cesium - 137	Ci	(1)	(1)	(1)	(1)	6.78E-04	7.15E-04	1.45E-03	2.62E-04
Barium - 140	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Lanthanum - 140	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Cerium - 144	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Europium - 154	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Europium - 155	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Tungsten - 187	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Total For Period	Ci	(1)	(1)	(1)	(1)	3.46E-03	3.30E-03	4.29E-03	3.66E-03
Krypton - 85	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Xenon - 131m	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Xenon - 133	Ci	(1)	(1)	(1)	(1)	1.00E-02	1.51E-06	7.29E-04	1.27E-03
Xenon - 133m	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Xenon - 135	Ci	(1)	(1)	(1)	(1)	(1)	(1)	(1)	(1)
Total For Period	Ci	(1)	(1)	(1)	(1)	1.00E-02	1.51E-06	7.29E-04	1.27E-03

NOTES TO TABLE 2B

- (1) Less than minimum detectable activity which meets the LLD requirements of ODCM Surveillance Requirement 4.11.1.1.1.
- (2) Continuous mode effluents are not analyzed for Fe-55.

APPENDIX A
CALVERT CLIFFS NUCLEAR POWER PLANT AND
INDEPENDENT SPENT FUEL STORAGE INSTALLATION
EFFLUENT AND WASTE DISPOSAL 2013 ANNUAL REPORT

TABLE 3A
SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

A. SOLID WASTE SHIPPED OFFSITE FOR BURIAL OR DISPOSAL (NOT IRRADIATED FUEL)

1. Type of Waste	Units	12-Month Period	Est. Total Error %
a) Spent resins, Filters	m ³ Ci	3.13+01 1.10E+03	25%
b) Dry compressible waste, contaminated equipment, etc.	m ³ Ci	1.91E+03 1.75E+00	25%
c) Irradiated components, control rods, etc.	m ³ Ci	0.00E+00 0.00E+00	N/A
d) Other (cartridge filters, misc. dry compressible, Oil)	m ³ Ci	7.00E+00 2.00E+00	25%
e) Solidification agent or absorbent	m ³	N/A	N/A

Volume shipped represents waste generated prior to offsite volume reduction.

2. Estimate of Major Nuclides (By Type of Waste - Only nuclides >1 % are reported)

a)	Mn-54	1.2%
	Fe-55	18.4%
	Co-58	1.13%
	Co-60	8.80%
	Ni-63	54.5%
	Cs-134	4.20%
	Cs-137	11.8%
b)	Cr-51	5.0%
	Fe-55	21.6%
	Co-58	15.5%
	Co-60	15.2%
	Ni-63	17.2%
	Ag-110m	1.4%
	Cs-134	7.8%
	Cs-137	16.2%
c)	N/A	
d)	Cr-51	3.3%
	Fe-55	22.1%
	Co-58	17.7%
	Co-60	16.5%
	Ni-63	25.6%
	Cs-134	4.4%
	Cs-137	9.8%
e)	N/A	

APPENDIX A
CALVERT CLIFFS NUCLEAR POWER PLANT AND
INDEPENDENT SPENT FUEL STORAGE INSTALLATION
EFFLUENT AND WASTE DISPOSAL 2013 ANNUAL REPORT

3. Solid Waste Disposition

<u>Number of Shipments</u>	<u>Mode of Transportation</u>	<u>Destination</u>
47	Motor Surface Transit	Studsvik Processing Facility Memphis, TN
11	Motor Surface Transit	Studsvik Processing Facility Erwin, TN
2	Motor Surface Transit	Energy Solutions (Duratek) Oak Ridge, TN

B. IRRADIATED FUEL SHIPMENTS (DISPOSITION) N/A