

NRR-PMDAPEm Resource

From: Miernicki, Michael
Sent: Monday, July 07, 2014 9:28 AM
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Subject: Watts Bar 2 - RAIs - Developmental Tech Specs Rev I - Steam Generators
Attachments: RAI WBN2 Stm Gen T.S. 7-7-14docx..docx

Attached please find RAIs from the Steam Generator Branch regarding Watts Bar 2 Developmental Technical Specifications Revision I. Please provide your responses within 30 days.

Mike

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REQUEST FOR ADDITIONAL INFORMATION
REVIEW OF DEVELOPMENTAL TECHNICAL SPECIFICATIONS REV. I
WATTS BAR NUCLEAR PLANT UNIT 2
DOCKET NUMBER 50-391

BACKGROUND

By letter dated June 16, 2014 (Agencywide Documents Access and Management System Accession No. ML14169A525), Tennessee Valley Authority (the licensee) submitted developmental Revision I of the Watts Bar Unit 2 Technical Specifications, for review and licensing by the U.S. Nuclear Regulatory Commission (NRC).

The NRC staff has reviewed sections 3.4.17 "Steam Generator (SG) Tube Integrity," B 3.4.17 "Steam Generator (SG) Tube Integrity Bases," 5.7.2.12 "Steam Generator (SG) Program," and 5.9.9 "Steam Generator Tube Inspection Report," which the licensee provided, and determined that additional information is required in order to complete the review. In order for the NRC staff to complete its review of the document listed above, please provide the following additional information:

QUESTION 1

On page B 3.4-90, the "Applicable Safety Analyses" section of B 3.4.17, "Steam Generator (SG) Tube Integrity Bases" does not adequately identify the leakage rates associated with the faulted and non-faulted SGs. The described leakage rates do not appear to be consistent with Section 15 of the Final Safety Analysis Report (Amendment 106). Please clarify which leakage rates are associated with the appropriate SGs.

QUESTION 2

On page B 3.4-95, the "Surveillance Requirements" section of B 3.4.17, "Steam Generator (SG) Tube Integrity Bases" contains an extraneous paragraph. The second paragraph on this page is an optional paragraph from Technical Specification Task Force 510 (TSTF-510) that is only used when approved repair methods are contained in the Technical Specification 5.7.2.12, "Steam Generator (SG) Program." Since Watts Bar Unit 2 has no approved repair methods, please confirm that this paragraph will be removed.

QUESTION 3

On pages 5.0-14 and 5.0-15, in Technical Specification 5.7.2.12, "Steam Generator (SG) Program," the incorporated wording is inconsistent with the wording in TSTF-510. In TSTF-510, at the end of the opening paragraph, the word "provisions"

was deleted because it is repeated at the beginning of paragraph 5.7.2.12.a. The closing parenthesis in paragraph 5.7.2.12.b.1 was moved, because not all of the anticipated transients in the design specification are considered part of normal operating conditions. Finally, in paragraph 5.7.2.12.d, the phrase, "An assessment of degradation" was replaced with, "A degradation assessment" so that the wording would be consistent with that used in industry SG program documents. Please revise these pages to be consistent with TSTF-510.

QUESTION 4

In Technical Specification 5.7.2.12, "Steam Generator (SG) Program," the most restrictive SG inspection periods were chosen from TSTF-510 (i.e., inspect 100% of the tubes at sequential periods of 60 effective full power months). Please confirm that the SGs at Watts Bar Unit 2 contain mill annealed Alloy 600 tubes. Also, please provide the model designation of the Watts Bar Unit 2 SGs.