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SUBJECT: Responds to Generic Ltr 89-21 re request for info on status of implementation of USI requirements.

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United States Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261/LICENSE NO. DPR-23

RESPONSE TO NRC GENERIC LETTER 89-21: REQUEST FOR INFORMATION CONCERNING STATUS OF IMPLEMENTATION OF UNRESOLVED SAFETY ISSUE (USI) REQUIREMENTS

Gentlemen:

The information attached is provided in response to your Generic Letter 89-21 dated October 19, 1989 regarding the status of implementation of Unresolved Safety Issue (USI) requirements. Please note that the time provided to determine the status of implementation of all USIs as required by your request has been short. Therefore, the attached notations regarding the status of individual items are necessarily brief and cannot address all of the details of the more complex issues. The attached status has been compiled based upon a review of correspondence between CP&L and the NRC such as Safety Evaluation Reports (SER) and Inspection Reports, as well as other plant-specific documents.

If you have any questions concerning this information, please contact Mr. R. W. Prunty at (919)546-7318.

Yours very truly,

L. I. Loflin
Manager
Nuclear Licensing Section

LIL/LSR/lbf (537CRS)
Attachment

cc: Mr. S. D. Ebneter
Mr. L. Garner (NRC - HBR)
Mr. R. Lo

A01
1/1

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ENCLOSURE 1

## UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

| USI/MPA NUMBER | TITLE | REF. DOCUMENT | APPLICABILITY | STATUS/DATE* | REMARKS |
|---|---|---|---|---|---|
| A-1 | Water Hammer | SECY 84-119<br>NUREG-0927, Rev. 1<br>NUREG-0993, Rev. 1<br>NUREG-0737 Item I.A.2.3<br>SRP revisions | All | NC | Note 1 |
| A-2/ MPA D-10 | Asymmetric Blowdown Loads on Reactor Primary Coolant Systems | NUREG-0609<br>GL 84-04, GDC-4 | PWR | NC | |
| A-3 | Westinghouse Steam Generator Tube Integrity | NUREG-0844<br>SECY 86-97<br>SECY 88-272<br>GL 85-02<br>(No requirements) | W-PWR | NC | Note 2 |
| A-4 | CE Steam Generator Tube Integrity | NUREG-0844, SECY 86-97<br>SECY 88-272<br>GL 85-02<br>(No requirements) | CE-PWR | NA | |
| A-5 | B&W Steam Generator Tube Integrity | NUREG-0844, SECY 86-97<br>SECY 88-272<br>GL 85-02<br>(No Requirements) | B&W-PWR | NA | |
| E<br>A-6 | Mark I Containment Short-Term Program | NUREG-0408 | Mark I-BWR | NA | |

* C - COMPLETE
NC - NO CHANGES NECESSARY
NA - NOT APPLICABLE
I - INCOMPLETE
E - EVALUATING ACTIONS REQUIRED

| USI/MPA NUMBER | TITLE | REF. DOCUMENT | APPLICABILITY | STATUS/DATE* | REMARKS |
|---|---|---|---|---|---|
| A-7/ D-01 | Mark I Long-Term Program | NUREG-0661<br>NUREG-0661 Suppl. 1<br>GL 79-57 | Mark I-BWR | NA | |
| A-8 | Mark II Containment Pool Dynamic Loads | NUREG-0808<br>NUREG-0487, Suppl. 1/2<br>NUREG-0802<br>SRP 6.2.1.1C<br>GDC 16 | Mark II-BWR | NA | |
| A-9 | Anticipated Transients Without Scram | NUREG-0460, Vol. 4<br>10 CFR 50.62 | All | C(2/89) | Note 3 |
| A-10/ MPA B-25 | BWR Feedwater Nozzle Cracking | NUREG-0619<br>Letter from DG Eisenhut dated 11/13/80<br>GL 81-11 | BWR | NA | |
| A-11 | Reactor Vessel Material Toughness | NUREG-0744, Rev. 1<br>10 CFR 50.60/<br>82-26 | All | NC | |
| A-12 | Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports | NUREG-0577, Rev. 1<br>SRP Revision<br>5.3.4 | PWR | NC | Note 4 |
| A-17 | Systems Interactions | Ltr: DeYoung to licensees - 9/77<br>NUREG-1174, NUREG-1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/CR-4470, GL 89-18<br>(No requirements) | All | NC | Note 5 |
| A-24/ MPA B-60 | Qualification of Class 1E Safety-Related Equipment | NUREG-0588, Rev. 1<br>SRP 3.11<br>10 CFR 50.49<br>GL 82-09, GL 84-24<br>GL 85-15 | All | C (3/85) | Note 6 |

| USI/MPA NUMBER | TITLE | REF. DOCUMENT | APPLICABILITY | STATUS/DATE* | REMARKS |
|---|---|---|---|---|---|
| A-26/ MPA B-04 | Reactor Vessel Pressure Transient Protection | DOR Letters to Licensees 8/76<br>NUREG-0224<br>NUREG-0371<br>SRP 5.2<br>GL 88-11 | PWR | C (4/78) | Note 7 |
| A-31 | Residual Heat Removal Shutdown Requirements | NUREG-0606<br>RG 1.113,<br>RG 1.139<br>SRP 5.4.7 | All OLs After 01/79. | NA | |
| A-36/ C-10, C-15 | Control of Heavy Loads Near Spent Fuel | NUREG-0612<br>SRP 9.1.5<br>GL 81-07, GL 83-42, GL 85-11<br>Letter from DG Eisenhut dated 12/22/80 | All | C (6/85) | Note 8 |
| A-39 | Determination of SRV Pool Dynamic Loads and Pressure Transients | NUREG-0802<br>NUREGs-0763,0783,0802<br>NUREG-0661<br>SRP 6.2.1.1.C | BWR | NA | |
| A-40 | Seismic Design Criteria | SRP Revisions, NUREG/CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776<br>NUREG/CR-3805<br>NUREG/CR-5347<br>NUREG/CR-3509 | All | | See A-46 |
| A-42/ MPA B-05 | Pipe Cracks in Boiling Water Reactors | NUREG-0313, Rev. 1<br>NUREG-0313, Rev. 2<br>GL 81-03, GL 88-01 | BWR | NA | |

| USI/MPA NUMBER | TITLE | REF. DOCUMENT | APPLICABILITY | STATUS/DATE* | REMARKS |
|---|---|---|---|---|---|
| A-43 | Containment Emergency Sump Performance | NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.82 (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements | All | NC | Note 9 |
| A-44 | Station Blackout | RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63 | All | I (see note) | Note 10 |
| A-45 | Shutdown Decay Heat Removal Requirements | SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements) | All | I (8/92) | Note 11 |
| A-46 | Seismic Qualification of Equipment in Operating Plants | NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03 | All | I/E (see note) | Note 12 |
| A-47 | Safety Implication of Control Systems | NUREG-1217, NUREG-1218 GL 89-19 | All | E (3/90) | Note 13 |
| A-48 | Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment | 10 CFR 50.44 SECY 89-122 | All, except PWRs with large dry containments | NA | |
| A-49 | Pressurized Thermal Shock | RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11 | PWR | C (6/89) | Note 14 |

NOTES

1. Changes to steam generators circa 1984 incorporate J tubes.
2. Response to Generic Letter 85-02 submitted 6/17/85.
3. AMSAC installed in Refueling Outage 12 ending 2/25/89.
4. Resolution only applied to plants with CP issued after October 1983.
5. Resolution did not require any licensee actions.
6. NRC SER dated 3/19/85.
7. Overpressure protection system installed 2-4/78 documented in SER for TS dated 9/14/79. Response to GL 88-11 submitted 12/22/88.
8. Phase I SER dated 5/29/84; Phase II closed by GL 85-11 dated 6/28/85.
9. Revised guidance resulting from the resolution does not apply to HBR2 per GL 89-21.
10. CP&L letter dated 3/3/89 proposed HBR2 actions to meet rule. Awaiting NRC action.
11. Subsumed into IPE. Currently scheduled for submittal to NRC 8/92.
12. CP&L is part of SQUG. Awaiting NRC approval of Rev. 1 of GIP. However, see CP&L letter dated 10/6/88.
13. Response to GL 89-14 to be provided by 3/19/90.
14. NRC SER on 10 CFR 50.61 dated 6/19/89. Response to GL 88-11 submitted dated 12/22/88. See A-26.

JSK/lbf (541CRS)