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REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9207 FACIL:50-261 H.B.	230043 DOC.DATE: 92/07/17 NOTARIZ Robinson Plant, Unit 2, Carolina Pow	(ED: NO ver & Light C	DOCKET # 05000261
AUTH.NAME	AUTHOR AFFILIATION		
CROOK, D.	Carolina Power & Light Co.		
CHAMBERS, R.H.	Carolina Power & Light Co.		
RECIP.NAME	RECIPIENT AFFILIATION		

SUBJECT: LER 92-012-00:on 920619, during surveillance testing reactor trip occurred.Root cause has not yet been identified. Corrective actions will be identified as part of ACR process & will be provided in supplementary rept .W/920717 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED:LTR (ENCL SIZE: 4 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

	RECIPIENT	COPIN	ES	RECIPIENT	COPI	IES
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	PD2-1 LA	1	1	PD2-1 PD	1	1
	MOZAFARI, B	1	1			
INTERNAL:	ACNW	2	2	AEOD/DOA	l	1
	AEOD/DSP/TPAB	1	1	AEOD/ROAB/DSP	2	2
	NRR/DET/EMEB 7E	1	1	NRR/DLPQ/LHFB10	1	1
	NRR/DLPO/LPEB10	1	1	NRR/DOEA/OEAB	1	1
	NRR/DREP/PRPB11	2	2	NRR/DST/SELB 8D	1	1
	NRR/DST/SICB8H3	1	1	✓NRR DST SPLB8D1	1	1
	NRR/DST/SRXB 8E	1	1	REG_FILE 02	1	1
	RES/DSIR/EIB	1	1	RGN2 FILE 01	1	1
EXTERNAL:	EG&G BRYCE, J.H	3	3	L ST LOBBY WARD	1	1
	NRC PDR	.1	1	NSIC MURPHY, G.A	1	1
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Carolina Power & Light Company

ROBINSON NUCLEAR PROJECT DEPARTMENT POST OFFICE BOX 790 HARTSVILLE, SOUTH CAROLINA 29550

JUL 1 7 1992

Robinson File No: 13510C

RNPD/92-1913 (10CFR50.73)

United States Nuclear Regulatory Commission Attn: Document Control Desk Washington, D. C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2 DOCKET NO. 50-261 LICENSE NO. DPR-23 LICENSEE EVENT REPORT NO. 92-012-00

Gentlemen:

The enclosed Licensee Event Report (LER), is submitted in accordance with 10 CFR 50.73 and NUREG 1022, Supplements No. 1 and 2.

Very truly yours,

R. H. Chambers General Manager H. B. Robinson S. E. Plant

RDC:sgk

Enclosure

cc: Mr. S. D. Ebneter Mr. L. W. Garner INPO



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EXPINES: 4/30/92 ESTIMATED BURDEN PER RESPONSE TO COMPLY WTH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REFORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.																			
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ABSTRACT (Limit to 1400 upcer, i.e. eporoximately fifteen single source typewritteen lines) (100) On June 19, 1992, H. B. Robinson Unit No. 2 was in hot shutdown and preparing for startup from a scheduled refueling outage. During surveillance testing which involved collection of cross calibration data for the Reactor Coolant System narrow range Resistance Temperature Detectors (RTD), a reactor trip occurred. At the time of the event, a Reactor Protection System bistable in loop 1 was in the tripped condition due to a RTD time response failure, unrelated to the on-going surveillance testing. The trip occurred when a temperature module voltage spiked high, causing a high delta temperature signal on loop 3. The plant was placed in stable condition, and an event investigation was initiated. The root cause of this event has not been determined. The apparent cause is attributed a combination of the response of the equipment being tested and the configuration of the system during the testing process. Corrective actions will be identified when the investigation is completed. There were no adverse safety consequences because the plant was shutdown at the time of the event. This report is submitted pursuant to 10 CFR 50.73(a)(2)(iv).																			

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LICENSEE EVENT RE	EPORT (LER) TEXT	CONTINUATION
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U.S. NUCLEAR REGULATORY COMMISSION APPROVED ONS NO. 3150-0104

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I. <u>DESCRIPTION OF EVENT</u>

On June 19, 1992, H. B. Robinson Unit No. 2^1 was in hot shutdown condition at 542 degrees F and preparing for startup from a scheduled refueling outage. At 2048 hours, with surveillance test EST-052, "Operational Alignment of Process Temperature Instrumentation" in progress, a reactor trip occurred. The Emergency Operating Procedures were entered, and the plant was placed in a stable condition. At the time of the event, the reactor protection Over Temperature Delta Temperature (OT Δ T) bistable (TC-412C) associated with Reactor Coolant System (RCS) Loop 1 was in the tripped condition due to a Resistance Temperature Detector (RTD) time response failure, unrelated to the ongoing surveillance testing.

The portion of the test being performed involved the collection of cross-calibration data for the RCS Narrow Range RTD's. The data is acquired using an RTD Test Panel external to the Reactor Protection and Control Analog Instrumentation cabinets (Hagan Racks) to switch between individual RTD's in order to measure their outputs at different temperature plateaus during heatup operations. On the test panel, individual switches are manually actuated to energize relays inside the test panel that divert the RTD input from the Hagan Rack terminal strip to the test meter used to measure the RTD resistance. These switches are actuated one at a time and each RTD output is restored to its Hagan Rack input before the next RTD is switched out of service to be measured. Four sets of data are taken in an alternate fashion (two sets in the order of loop 1, then loop two, and lastly loop three, and two sets in reverse order) at each temperature plateau.

Data had been successfully acquired from loops 1 and 2 without incident. However, during data collection for loop 3, the temperature module voltage for TE-432B2 spiked high, due to the rapid removal of the input resistance when the test switch was actuated, causing the OTAT trip setpoint to be exceeded. This, when combined with the pre-existing OTAT bistable tripped for loop one, resulted in a reactor trip via the satisfaction of the required two out of three logic.

The NRC was notified of this event at 1221 hours via the ENS pursuant to 10 CFR 50.72(b)(2)(ii).

¹ H. B. Robinson Steam Electric Plant, Unit No. 2 is a Westinghouse Pressurized Water Reactor in commercial operation since March, 1971.

LICENSEE EV	VENT REPORT	(LER) TEXT	CONTINUATION
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U.S. NUCLEAR REGULATORY COMMISSIO

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II. CAUSE OF EVENT

An Adverse Condition Report² was initiated so that this event could be investigated and the root cause be determined. Because the investigation process has not yet been completed, the root cause of this event has not yet been identified. The apparent cause is a combination of the response of the equipment being tested and the configuration of the system during the testing process.

III. ANALYSIS OF EVENT

There were no adverse safety consequences as a result of this event. The portion of the test being conducted at the time is not performed during power operations. The plant was in hot shutdown condition at the time of the event, safeguards systems performed as designed, and a significant transient did not occur. Plant operations personnel maintained plant safety in accordance with established procedures.

This report is submitted pursuant to 10 CFR 50.73(a)(2)(iv).

IV. CORRECTIVE_ACTIONS

Corrective actions will be identified as part of the ACR process and will be provided in a supplement to this report.

V. ADDITIONAL INFORMATION

A. Previous Similar Events

None

B. Component Failures

None

² Adverse Condition Report 92-222.