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FACIL:50-261 H.B. Robinson Plant, Unit 2, Carolina Power & Light C 05000261
AUTH.NAME AUTHOR AFFILIATION
BAUR,D.H. Carolina Power & Light Co.
CHAMBERS,R.H. Carolina Power & Light Co.
RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 92-002-00:on 920127,failure to test circuitry associated w/auxiliary feedwater auto start noted.Caused by procedural deficiencies.Revising MST-202 to ensure that required portions of sys are appropriately tested.W/920212 ltr.

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United States Nuclear Regulatory Commission
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H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261:
LICENSE NO. DPR-23
LICENSEE EVENT REPORT 92-002

Gentlemen:

The enclosed Licensee Event Report (LER) is submitted in accordance with 10 CFR 50.73 and NUREG-1022 including Supplements No. 1 and 2.

Very truly yours,

Ray H. Chambers

R. H. Chambers
Plant General Manager
H. B. Robinson S. E. Plant

DHB:sgk

Enclosure

cc: Mr. S. D. Ebneter
Mr. L. W. Garner
INPO

9202190120 920212
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Enclosure to Serial: RNPD/92-0344

NRC FORM 366 (6-89) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104
EXPIRES: 4/30/92

ESTIMATED BURDEN PER RESPONSE TO COMPLY WTH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2	0 5 0 0 0 2 6 1	1 OF 0 5

TITLE (4)

FAILURE TO TEST ALL CIRCUITRY ASSOCIATED WITH AUXILIARY FEEDWATER AUTO START

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 1	2 7	9 2	9 2	— 0 0 2	— 0 0	0 2	1 2	9 2		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: *(Check one or more of the following)* (11)					
POWER LEVEL (10)	1 0 0	20.402(b)		20.405(c)		50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)	OTHER *(Specify in Abstract below and in Text, NRC Form 366A)*
		20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
DAVID H. BAUR - REGULATORY COMPLIANCE	AREA CODE 8 0 3 — 3 8 3 — 1 2 9 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES *(If yes, complete EXPECTED SUBMISSION DATE)*	X NO				

ABSTRACT *(Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines)* (16)

At about 1045 hours on January 27, 1992, with H. B. Robinson Unit No. 2 operating at 100% power, it was determined that the condition identified in ACR 92-013, dated January 15, 1992, represented a condition that was reportable, but did not involve inoperable equipment. The ACR identified that a set of normally closed contacts, TX, and interconnecting wiring, shown on Control Wiring Diagram, B-190628 Sheets 651 and 655, could not be shown to be tested during the performance of MST-202, "4KV Main Feedwater Breakers Open-Auto Start Test Of Motor Driven Auxiliary Feedwater (MDAFW) System". MST-202 is conducted to satisfy the requirements of Technical Specifications Table 4.8-1 Item e, Trip of Main Feedwater Pumps. The ACR also stated that the operability of the Motor Driven Auxiliary Feedwater Pumps was not a concern because both pumps auto started on a Steam Generator Low Level signal during a reactor trip on August 30, 1991. This MDAFW auto start on Steam Generator Low Level signal utilized the same TX contacts that were identified in the ACR, and has thereby verified that the untested TX contacts and interconnecting wiring are intact and functional.

NRC Form 366A
(9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2	0 5 0 0 0 2 6 1	9 2	— 0 0 2	— 0 0	0 2 OF 0 5

TEXT (If more space is required, use additional NRC Form 366A's) (17)

I DESCRIPTION OF EVENT

At about 1045 hours on January 27, 1992, with H. B. Robinson Unit No. 2[1] operating at 100% power, it was determined that the condition identified in ACR 92-013, dated January 15, 1992, represented a condition that was reportable, but did not involve inoperable equipment. The ACR identified that a set of normally closed contacts, TX, and interconnecting wiring, shown on Control Wiring Diagram, B-190628 Sheets 651 and 655[2], could not be shown to have been tested during the performance of MST-202, "4KV Main Feedwater Breakers Open-Auto Start Test Of Motor Driven Auxiliary Feedwater (MDAFW) System". MST-202 is conducted to satisfy the requirements of Technical Specifications Table 4.8-1 Item e, Trip of Main Feedwater Pumps. The ACR also stated that the operability of the Motor Driven Auxiliary Feedwater Pumps was not a concern because both pumps auto started on a Steam Generator Low Level signal during a reactor trip on August 30, 1991. This MDAFW auto start on Steam Generator Low Level signal utilized the same TX contacts that were identified in the ACR, and has thereby verified that the untested TX contacts and interconnecting wiring are intact and functional. The time period between January 15 and January 27, 1992 was consumed by ACR administrative time, and evaluation of procedures, Control Wiring Diagrams, and interpretation of Technical Specification requirements for Channel Functional Testing that was necessary to determine reportability.

II CAUSE OF EVENT

The cause of this condition is attributed to procedural deficiencies. This deficiency was identified by extensive technical review of the procedure during the on-going Maintenance Procedure Upgrade Program. Also contributing to the identification of this deficiency is the added emphasis that has been placed on procedure technical reviews, and increased awareness of changing industry standards and expectations.

III ANALYSIS OF EVENT

This condition is being reported under 10CFR50.73 (a) (2) (i) B as an operation or condition prohibited by the plant's Technical Specifications. The violation of Technical Specifications occurred when it was determined that the contacts and associated wiring could not be shown to have been tested as required by Table 4.8-1 Item e. At the time this condition was identified, it was not considered to be a significant safety concern since the contacts and wiring were functional and did operate during an August 30, 1991 reactor trip.

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[1]H. B. Robinson Unit No. 2 is a Pressurized Water Reactor in commercial operation since March, 1971.

[2]See "clouded" area of Figure 1 and 2

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
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H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2	0 5 0 0 0 2 6 1	9 2	— 0 0 2	— 0 0	0 3 OF 0 5

TEXT (If more space is required, use additional NRC Form 366A's) (17)

IV CORRECTIVE ACTIONS

Corrective actions include revising MST-202 to ensure that the required portions of the system are appropriately tested. Additionally, the Maintenance Procedure Upgrade Program, committed to in our response to NRC Inspection Report No. 50-261/89-02, identified this condition and will continue to identify other like conditions, if they exist. Carolina Power and Light Company also has procedural verification of Technical Specification Table 4.1-1 surveillance requirements as committed to in our response to NRC Inspection Report No. 50-261/90-11. The purpose of this program is to identify other conditions, if they exist, similar to that being reported in this LER.

The revision of procedure MST-202 is scheduled to be complete by March 15, 1992. MST-202 is a Refueling Interval test, and this revision date will ensure the changes are complete prior to the next test performance. The other actions listed above are committed to as identified.

V ADDITIONAL INFORMATION

1. Component failures.

   None

2. Previous similar events.

   LER 84-005
   LER 86-008
   LER 88-011
   LER 90-005
   LER 91-012

NRC Form 366A (9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2	0 5 0 0 0 2 6 1	9 2	— 0 0 2	— 0 0	0 4 OF 0 5

TEXT (If more space is required, use additional NRC Form 366A's) (17)

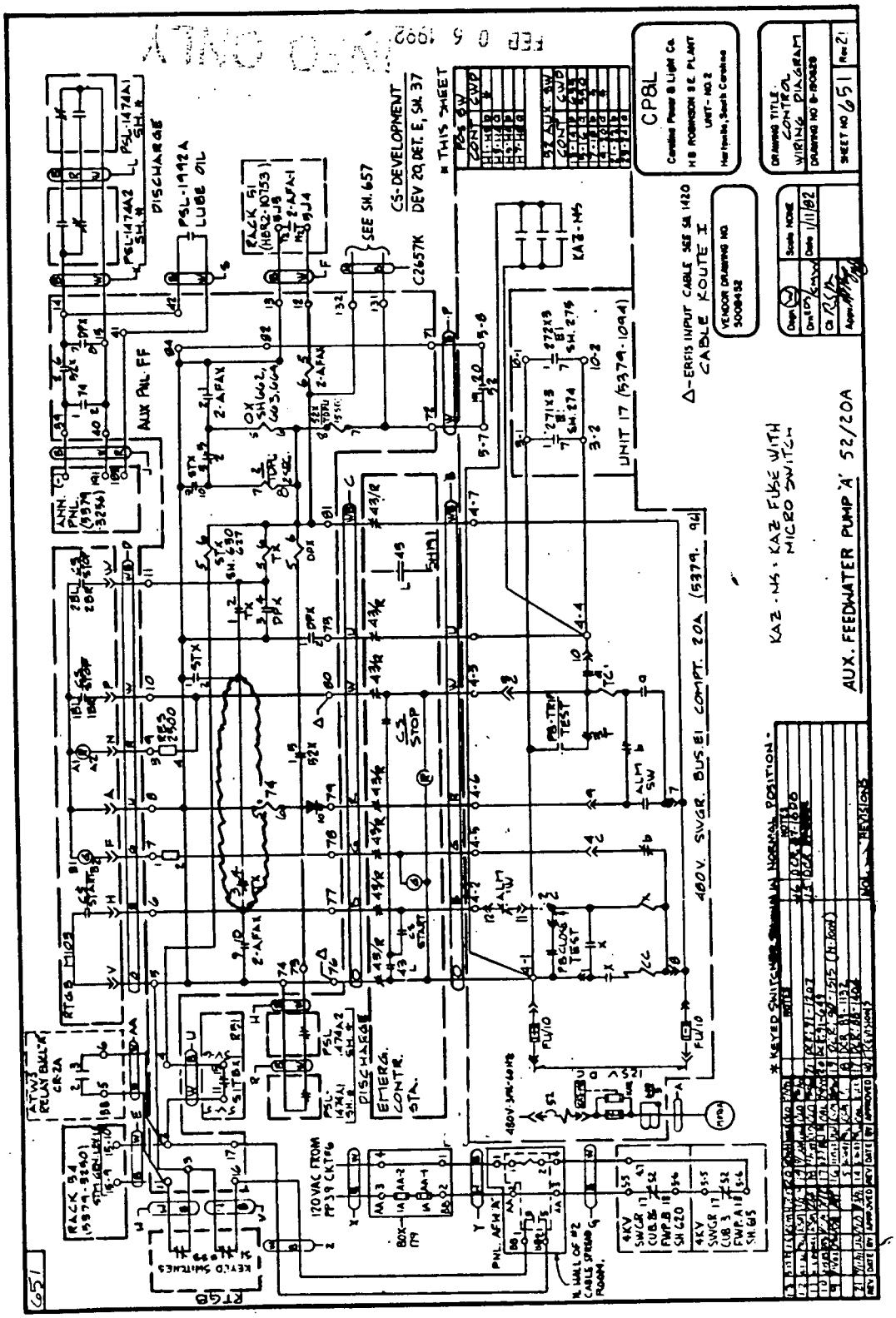

FIGURE 1

• U.S. GPO: 1988-0-624-538/455

NRC Form 366A (9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2	0 5 0 0 0 2 6 1	9 2	— 0 0 2	— 0 0	0 5 OF 0 5

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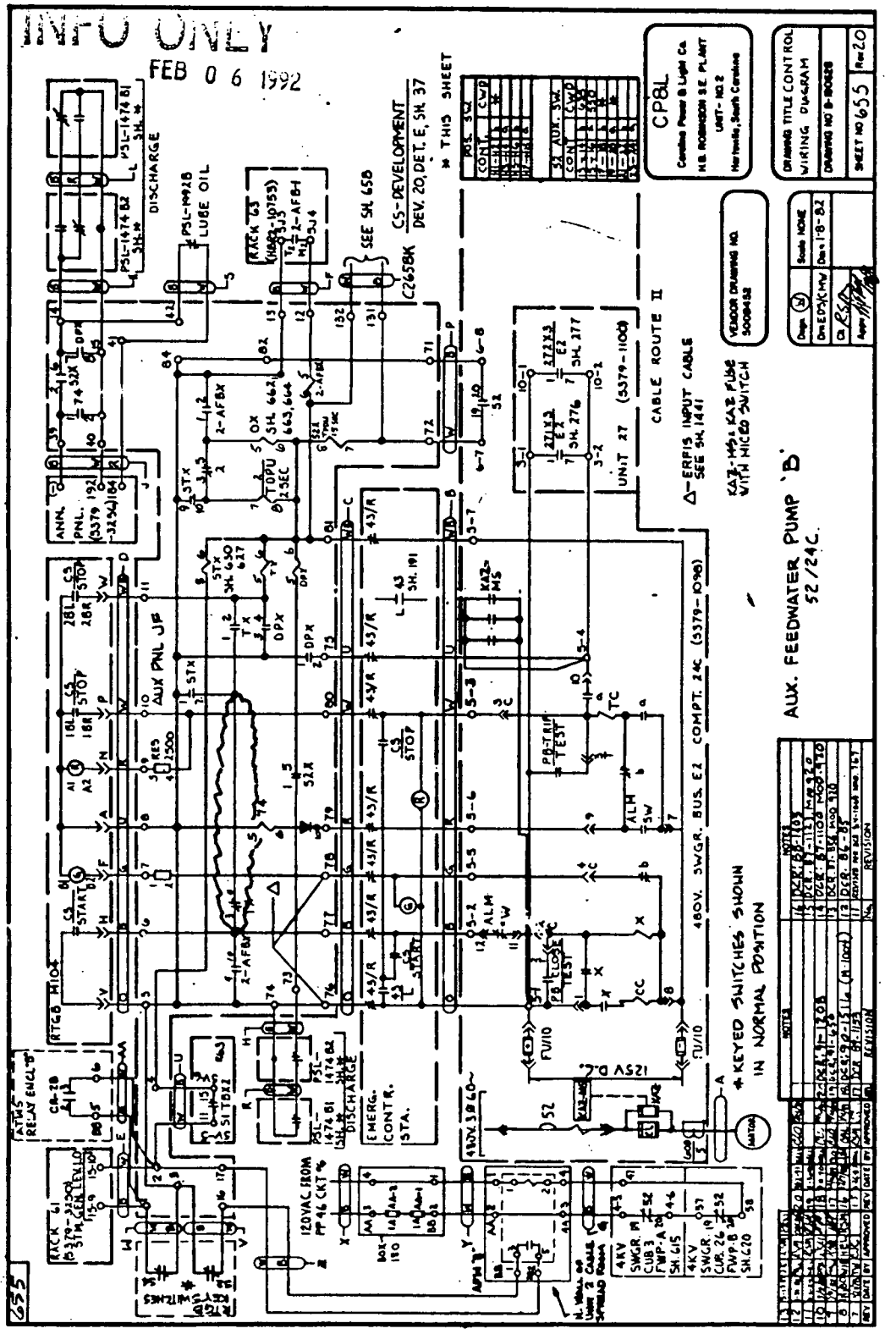

FIGURE 2

* U.S. GPO 1988-0-624-538/435