### REGULATURY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSIUN NBR:8102180493 DUC.DATE: 81/02/12 NOTARIZED: NO DOCKET # FACIL:50-261 H. B. Robinson Plant, Unit 2, Carolina Power and Ligh 05000261 AUTH.NAME AUTHUR AFFILIATION STARKEY,R.B. Carolina Power & Light Co. RECIP.NAME RECIPIENT AFFILIATION Region 2, Atlanta, Office of the Director

SUBJECT: LER 81=005/01T=0:on 810129, spurious safeguards actuation & reactor trip resulted from high steam flow spike.Probable cause was valve CVCS=200E vibrating open & pipe cap failing to hold following safety injection.Valve secured.

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NOTES:

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ACTION:	VARGA,S. (	04	3	3.	-		•
INTERNAL:	A/D COMP&STRU	06	1	1	A/D ENV TECH 07	1	1
	A/D MATL & QUO	08	1	1	A/D OP REACTOU9	1	1
	A/D PLANT SYS:	10	1	1	A/D RAD PROT 11	1	1
	A/D SFTY ASSES	12	1	1	ACC EVAL BR 14	1	1
	AEŬD		3	3	ASLBP/J.HARD	1	1
	AUX SYS BR 1	15	1	1	CHEM ENG BR 16	1	1
	CONT SYS BR 1	17	1	1	CORE PERF BR 18	1	1
	DIR, DIV OF LIC	2	1	1	DIR, ENGINEERI20	1	1
	DIR, HUM FAC Sa	21	1	1	DIR, SYS INTEG22	1	1
	EFF TR SYS BRA	23	1	1	EQUIP QUAL BR25	1	1
	GEUSCIENCES 2	26	1	1	I&C SYS BR 29	1	1
	18Ë (	)5	2	2	JORDAN, E./IE	1	1
	LIC GUID BR	30	1	1	MATL ENG BR 32	1	1
	MECH ENG BR	33	1	1	MPA	3	3
	NRC PDR (	2	1	1	OP EX EVAL BR34	3	3
	OR ASSESS BR 3	35	1	1	POWER SYS BR 36	1	1
	RAD ASSESS BR	39	1	1	REACT SYS BR 40	1	1
	REG FILE) (	) 1	1	1	REL & RISK A 41	1	1
	SFTY PROG EVAL	ιŽ	1	1	STRUCT ENG BR44	1	1
	SYS INTERAC B4	45	1	1			
EXTERNAL:	ACKS 4	16	16	16	LPDR 03	1	1
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#### SUPPLEMENTAL INFORMATION

#### FOR

### LICENSEE EVENT REPORT 81-005

# 1. Cause Description and Analysis:

On January 29, 1981, at 0541 hours, a load reduction was initiated to reach a hot standby condition to allow repair of a turbine E-H system oil leak. At 0624 hours, immediately following opening of the generator output breakers, a governor valve spiked open, apparently due to the E-H problems, causing the high steam flow bi-stables to momentarily operate. This momentary high steam flow signal with the low Tavg signal (<543543oF) caused "B" train of safeguards to initiate. It has been concluded that the high steam flow signal spike was of insufficient duration to fully latch the "A" train of safeguards. "A" train was manually initiated at 0625 hours. With conditions appearing normal, the SI was reset at 0627 hours. The low pressure letdown was restored at 0635 hours and the reactor coolant system pressure began decreasing with the containment pressure and dew point increasing. The letdown was secured at 0650 hours. At this time, the containment sump level monitor indicated approximately 3000 gallons in the containment sump. Pressure continued to decrease until at 0705 hours, a "Low Pressurizer Pressure" safety injection was received with both trains operating normally. The RCS pressure decrease was determined to be caused by a leaking pressurizer spray valve and was corrected by stopping the reactor coolant pumps associated with the spray. The pressurizer pressure immediately started to increase and normal control of reactor coolant pressure was me-established.

Upon containment entry, valve CVCS-200E, a letdown line drain valve, size 3/4 inch, was found to be leaking slightly (approximately 5-7 GPM). Letdown was isolated at this time but apparently some leakage occurred through the air operated letdown isolation valves. The leak was later stopped by shutting the valve. The leak rate could not be accurately determined but it was estimated to have been approximately 100 GPM while letdown was unisolated which exceeds the limit of Technical Specification 3.1.5.2. A total of approximately 4500 to 6000 gallons of water was leaked to the containment sump during the event.

The valve CVCS-200E is a normally closed valve with a pipe cap on the outlet. It appears the valve had vibrated open during previous operation and the pipe cap which was serving as a pressure boundary, blew off sometime after the initial SI. The leakage was confined to containment so there was no threat to the plant or public health and safety.

#### 2. Corrective Action:

The valve CVCS-200E was closed to stop the leakage of water. The last threads of the valve outlet pipe (damaged when the cap blew off) were dressed and a new pipe cap was installed.

# 3. Corrective Action To Prevent Further Occurrence:

The valve CVCS-200E, and several other valve/pipe cap arrangements which could be exposed to the same conditions were inspected and physically locked or verified secured in the closed position.

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<b>ب</b>	CONTROL BLOCK:
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	REPORT L 6 0 5 0 0 2 6 1 7 0 1 2 9 8 1 6 0 2 1 2 8 1 9 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80
0 2	On January 29, 1981, at 0624 hours, a spurious safeguards actuation and reactor trip
03	were received resulting from a high steam flow spike caused by turbine governor valve/
04	E-H oil problems. Shortly thereafter letdown flow was restored and RCS pressure began
05 06 07 08	decreasing with containment pressure and dew point increasing. Upon containment entry, following letdown isolation a letdown line drain valve (CVCS-200E) was found partially opened and leaking through with its pipe end-cap missing. The leak rate with letdown unisolated was estimated at approximately 100 GPM which exceeds the limits of Technical Specification 3.1.5.2. No uncontrolled release to the environment occurred and there was no adverse impact to the general public. This item is reportable pursuant to Paragraph 6.9.2.a(2) of the Unit Technical Specifications.
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	Probable cause is attributed to the valve vibrating open and the pipe cap, which was serving as a pressure boundary, failing to hold at sometime following the first SI. Approximately 4500-6000 gallons of water leaked to the containment sump during the event. As immediate corrective action, letdown was secured. As further corrective action the valve was secured in the closed position and a new cap was installed. Additionally, other similar valves were locked or verified secured in the closed posi- tion to prevent recurrence.
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$\begin{bmatrix} 1 & 7 \\ 7 & 8 \end{bmatrix}$	NUMBER TYPE DESCRIPTION N/A B0   9 11 12 13 80   9 11 12 13 80   NUMBER DESCRIPTION 41 0
7 8 20 7 8	9 10 PUBLICITY 8102180493 NRC USE ONLY   ISSUED DESCRIPTION (45) NRC USE ONLY 56   9 10 68 69 80   9 10 68 69 80   NAME OF PREPARER R. B. Starkey, Jr. PHONE: (803) 383-4524 68