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CHAMBERS,R.H. Carolina Power & Light Co.
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SUBJECT: Advises that containment structural test will be conducted during refueling outage 14,currently scheduled to commence on 920328.

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ROBINSON NUCLEAR PROJECT DEPARTMENT
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CONTAINMENT STRUCTURAL TEST

Gentlemen:

In accordance with the reporting requirements provided within Technical Specification (TS) 4.4.4.3.c for H. B. Robinson Unit No. 2 (HBR2), this letter provides notification of the pending containment structural test. This test will be conducted during the refueling outage number 14 which is currently scheduled to commence March 28, 1992. Actual performance of the Structural Integrity Test (SIT) is currently scheduled to begin on April 7, 1992. The submittal of this letter satisfies the requirement to notify the NRC at least two months prior to the actual test, and also provides the required detailed acceptance criteria.

Section 4.4.4.3 (Acceptance Criteria) of the TS states:

b. Observation of the structural test at design pressure indicating no significant differences in containment growth and crack pattern spacing and width from that during the proof test shall be considered as demonstrating the continual integrity of the structure. It is realized that the deflections, in the prestressed direction particularly, will be small, that the significance of differences in these small deflections will be difficult to evaluate, and therefore that only a gross difference in the structure, such as a large loss of prestress force, would be apparent from the measurements. The difference in measurements, if any, will be examined considering the predictable range of variation of time dependent changes in material properties, the thermal conditions at the time of the test, instrument error and other pertinent factors.

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As quantification of the above acceptance criteria, the following are the maximum acceptable displacements as previously identified in letter NG-74-223, dated February 20, 1974.

a) Radial Displacements
   1) Containment Diameter 1.62 inch
   2) Equipment Hatch 0.69 inch

b) Vertical Displacement
   1) Containment Wall 0.175 inch

The measurements listed above include the 20 percent tolerance for acceptability without additional evaluation.

The acceptable crack patterns without additional engineering evaluation are as follows:

1) Average Crack Spacing (Vertical) 17 inches

2) Width of Crack shall be compared to the 1970 Test Measurement of 0.027 inch

The exceeding of these acceptance values will be preliminary evaluated prior to plant start-up, with complete evaluation and documentation included in the final report.

In accordance with the requirements of TS 4.4.4.3.c, the final report and evaluation will be submitted to the NRC within six months of conducting the test.

Should you have any questions regarding this matter, please contact Mr. C. T. Baucom at (803) 383-1253.

Very truly yours,

R. H. Chambers
Plant General Manager
Robinson Nuclear Project Department

CTB:dwm

cc: Mr. S. D. Ebneter
Mr. L. W. Garner
Mr. R. Lo