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REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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 FACIL: 50-261 H. B. Robinson Plant, Unit 2, Carolina Power and Light 05000261  
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SUBJECT: Forwards corrected Page 10 to Safety & Relief Valve Adequacy  
 Rept submitted 821104, in response to NUREG-0737,  
 Item II.D.1.2. PORVs opened to prevent exceeding NDT curve.

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Carolina Power & Light Company

JAN 20 1983

Director of Nuclear Reactor Regulation  
Attention: Mr. Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing  
United States Nuclear Regulatory Commission  
Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261  
LICENSE NO. DPR-23  
NUREG-0737, ITEM II.D.1.2 - CORRECTION TO  
SAFETY AND RELIEF VALVE ADEQUACY REPORT

Dear Mr. Varga:

In a letter dated November 4, 1982, Carolina Power & Light Company (CP&L) submitted the Safety and Relief Valve Adequacy Report for the H. B. Robinson Steam Electric Plant Unit 2 (HBR2) as required by NUREG-0737, Item II.D.1.2.

There is an error on page 10 of the report which states that the PORVs are opened if the NDT curve is violated. In actuality the PORVs are opened to prevent exceeding the NDT curve. The page has been corrected and is enclosed for your use.

If you have any questions regarding this matter, please contact a member of my staff.

Yours very truly,

S. R. Zimmerman  
Manager  
Licensing & Permits

DCW/kjr (6061C12T1)

cc: Mr. J. P. O'Reilly (NRC-RII)  
Mr. G. Requa (NRC)  
Mr. Steve Weise (NRC-HBR)

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### 3.0 VALVE INLET FLUID CONDITIONS

Justification for inlet fluid conditions used in the EPRI Safety and Relief Valve tests are summarized in References 2 and 3. These conditions were determined based on consideration of FSAR, extended High Pressure Injection, and Cold Overpressurization events, where applicable.

For plants of which Westinghouse is the NSSS supplier, a methodology was used such that a reference plant was selected for each grouping of plant considered.<sup>(3)</sup> Valve fluid conditions resulting from limiting FSAR events, which result in steam discharge and an Extended High Pressure Injection event which may result in liquid discharge, are presented for each reference plant. Use of reference plants results in fluid conditions enveloping those expected for H. B. Robinson Unit 2.

Table 3-1 presents the results of loss of load and locked rotor analysis for three loop plants in which H. B. Robinson Unit 2 was included. The inlet fluid conditions expected at the safety valve and PORV inlets are identified. As can be seen, the Locked Rotor event is considered as the limiting overpressure transient for three loop plants.

The limiting Extended High Pressure Injection event was the spurious activation of the safety injection system at power. For three-loop plants Table 3-2 provides the fluid conditions for safety and PORV's. As the high head Injection Pump cutoff head is 1500 psi, no liquid discharge is expected through the safeties.

Fluid conditions for cold overpressure protection are provided in Table 3-3. Cold overpressure is not a design basis for the Safety Valves but is for the PORV's.

The H. B. Robinson Unit No. 2 cold overpressurization system utilizes the pressurizer power operated relief valves to relieve excess pressure occurring when the RCS temperature is below 350°F. There are two PORV's which are controlled independently by a combination of temperature and pressure instrumentation, and the PORV's are opened to prevent exceeding the NDT curve. The system is activated only when the RCS temperature is below 350°F.