

**ENCLOSURE 5**

**H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
NRC DOCKET NO. 50-261/OPERATING LICENSE NO. DPR-23  
REQUEST FOR LICENSE AMENDMENT  
PERFORMANCE BASED NUCLEAR ASSESSMENT PROGRAM**

**MARKED-UP TECHNICAL SPECIFICATION PAGES**

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assure that the reviewers collectively possess the background and qualifications in the disciplines necessary and important to the specific review. The list will include the disciplines for which each person is qualified.

6.5.1.1.5 Temporary changes to procedures, tests, or experiments may be approved by two members of the plant staff, at least one of whom holds a Senior Reactor Operator License if such change does not change the intent of the original procedure, test, or experiment. Temporary changes shall be documented and, within 21 days of receiving temporary approval, be reviewed in accordance with Specifications 6.5.1.1.2, 6.5.1.1.3, and 6.5.1.1.4 and incorporated as a permanent change or deleted.

6.5.1.1.6 Those procedures, tests, or experiments and changes thereto that constitute an unreviewed safety question, or involve a change to Technical Specifications shall be reviewed by the Plant Nuclear Safety Committee and submitted to the NRC for approval prior to implementation. All such procedures, tests, or experiments and changes shall be reviewed by the Nuclear Assessment ~~Department~~ prior to implementation.

SECTION

6.5.1.1.7 Procedures, tests, or experiments, which constitute a change to the <sup>facility as described in the</sup> FSAR shall also be reviewed by the Nuclear Assessment <sup>U</sup> ~~Department~~ SECTION. These reviews may be conducted after plant Management approval, and implementation may proceed prior to completion of review as provided for by 10 CFR 50.59(a)(1).

6.5.1.2 Modifications

6.5.1.2.1 A safety analysis shall be prepared for all modifications that affect nuclear safety. The analysis shall include a written determination of whether or not the modification is a change in the facility as described in the FSAR, involves a change to the Technical Specifications, or <sup>U</sup> constitutes an unreviewed safety question as defined in 10 CFR 50.59(a)(2).

6.5.1.2.4 Modifications that are determined to either constitute an unreviewed safety question, as defined in 10 CFR 50.59(a)(2), or a change to the Technical Specifications, shall be reviewed by the Plant Nuclear Safety Committee and submitted to the NRC for approval prior to implementation. All such modifications shall be approved by the Nuclear Assessment ~~Department~~ prior to implementation.

Section

6.5.1.2.5 Modifications which constitute changes to the facility as described in the FSAR shall also be reviewed by the Nuclear Assessment ~~Department~~. This review may be conducted after plant management approval, and implementation may proceed prior to completion of review.

6.5.1.3 Technical Specifications and License Changes

6.5.1.3.1 Each proposed Technical Specification or Operating License change shall be reviewed by the Plant Nuclear Safety Committee and submitted to the NRC for approval.

6.5.1.4 Review of Technical Specification Violations

6.5.1.4.1 All violations of Technical Specifications shall be investigated and a report prepared that evaluates the event and that provides recommendations to prevent recurrence. Such reports shall be reviewed by the Plant Nuclear Safety Committee and approved by the Plant General Manager or his designee and submitted to the Vice President - Robinson Nuclear ~~Project~~ and to the Manager - Nuclear Assessment ~~Department~~.

Section

6.5.1.5 Nuclear Safety Review Qualification

6.5.1.5.1 Individuals shall be designated by the Vice President - Robinson Nuclear Project for the safety reviews of Specifications 6.5.1.1.2, 6.5.1.1.3, 6.5.1.2.1, and 6.5.1.2.2. These reviewers shall have a Bachelor of Science in engineering or related field or equivalent and two years related experience.

6.5.1.6 Plant Nuclear Safety Committee (PNSC)

- 6.5.1.6.1 a. As an effective means for the regular overview, evaluation, and maintenance of plant operational safety, a Plant Nuclear Safety Committee (PNSC) is established.
- b. The committee shall function, through the utilization of subcommittees, audits, investigations, reports, and/or performance of reviews as a group, to advise the General Manager on all matters related to nuclear safety.

6.5.1.6.2 The PNSC shall be composed of the following:

Chairman - General Manager or designated alternate  
Member - Manager - Operations or designated alternate  
Member - Manager - Maintenance or designated alternate  
Member - Manager - Technical Support or designated alternate  
Member - Director - Regulatory Compliance or designated alternate  
Member - Manager - Environmental & Radiation Control or designated alternate  
Member - Manager - QA/QC or designated alternate  
Member - Manager - Outages and Modifications or designated alternate  
Member - Manager - Plant Support or designated alternate

6.5.1.6.3 Alternates shall be appointed in writing by the General Manager to serve on a temporary basis. All alternates shall, as a minimum, meet qualification criteria specified in Section 4.4 of ANSI N18.1-1971 for professional-technical personnel, or for those disciplines not listed in Section 4.4, the equivalent of the Section 4.4 requirement.

6.5.1.6.4 The PNSC shall meet at least once per calendar month and as convened by the PNSC Chairman or his designated alternate.

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~~6.5.1.6.5 A quorum of the PNSC shall consist of the Chairman, and four members, of which two may be alternates.~~

6.5.1.6.6 The PNSC activities shall include the following:

- a) Perform an overview of Specifications 6.5.1.1 and 6.5.1.2 to assure that processes are effectively maintained.
- b) Performance of special reviews, investigations, and reports thereon requested by the Manager - Nuclear Assessment ~~Department.~~ Section
- c) Annual review of the Security Plan and Emergency Plan.
- d) Perform reviews of Specifications 6.5.1.1.6, 6.5.1.2.4, 6.5.1.3.1, and 6.5.1.4.1.
- e) Perform review of all reportable events.
- f) Review of facility operations to detect potential nuclear safety hazards.
- g) Review of every unplanned on site release of radioactive material to the environs including the preparation and forwarding of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrences to the Vice President - Robinson Nuclear ~~Project,~~ Plant  
~~Manager - Nuclear Assessment Department.~~
- h) Review of changes to the Process Control Program and the Offsite Dose Calculation Manual.
- i) Review of major changes to radioactive liquid, gaseous, and solid waste treatment systems.
- j) Review of changes to the CORE OPERATING LIMITS REPORT.
- k) Annual review of the Fire Protection Program, including Program changes.

**INSERT TO PAGE 6.5-6 and 7.**

6.5.1.6.2 The PNSC shall be composed of a Chairman and seven to nine members. The members shall be from the following functional areas:

Operations  
Maintenance  
Engineering  
Health Physics/Chemistry  
Regulatory Affairs  
Nuclear Assessment

6.5.1.6.3 The PNSC Chairman, alternate Chairmen, members, and alternate members shall be designated in writing by the plant General Manager. Members shall be individuals who are unit manager level or above from the site management organization. Alternate members shall, as a minimum, meet equivalent qualification criteria as specified in Section 4.4 of ANSI N18.1-1971 for professional-technical personnel.

6.5.1.6.4.a The quorum of the PNSC necessary for the performance of the activities of these Technical Specifications shall consist of the Chairman (or his designated alternate) and four members (including alternates).

6.5.1.6.4.b No more than two alternates shall be counted toward meeting the quorum requirement or participate as voting members of the PNSC at any one time.

6.5.1.6.5 The PNSC shall meet at least once per calendar month and as convened by the PNSC Chairman or his designated alternate.

6.5.1.6.7 In the event of disagreement between the recommendations of the Plant Nuclear Safety Committee and the actions contemplated by the General Manager, the course determined by the General Manager to be more conservative will be followed. The Vice President - Robinson Nuclear ~~Project and the Manager - Nuclear Assessment Department~~ will be notified <sup>PLANT</sup> within 24 hours of the disagreement and subsequent actions.

6.5.1.6.8 The PNSC shall maintain written minutes of each meeting that, at a minimum, document the results of all PNSC activities performed under the provisions of these Technical Specifications; and copies shall be provided to the Vice President - Robinson Nuclear ~~Project~~ and to the <sup>PLANT</sup> Manager - Nuclear Assessment ~~Department~~.  
SECTION

Replace entire Section 6.5.2 with attached

6.5.2 Nuclear Assessment Department - Independent Review

The Nuclear Assessment Department shall provide independent review of significant plant changes, tests, and procedures; verify that reportable events are investigated in a timely manner and corrected in a manner that reduces the probability of recurrence of such events; and detect trends that may not be apparent to a day-to-day observer. Specific review subjects are defined in Specification 6.5.2.1.d.

6.5.2.1 The Manager - Nuclear Assessment Department is charged with the overall responsibility for administering the independent review function as follows:

a) Approves selection of the individuals to conduct safety reviews under Specification 6.5.2.

b) Has access to plant records and operating personnel in performing independent reviews.

c) Prepares and retains written records of reviews.

d) Assures independent reviews are conducted on the following subjects:

1) Written safety evaluations of changes in the facility as described in the Safety Analysis Report, changes in procedures as described in the Safety Analysis Report, and tests or experiments not described in the Safety Analysis Report that are completed without prior NRC approval under the provisions of 10 CFR 50.59(a)(1). This review is to verify that such changes, tests, or experiments did not involve a change in the Technical Specifications or an unreviewed safety question as



Replace entire section 6.5.2 with attached

defined in 10CFR50.59(a)(2). These reviews may be conducted after appropriate management approval, and implementation may proceed prior to completion of the review.

- (2) Proposed changes in procedures, proposed changes in the facility, or proposed tests or experiments, any of which involves a change in the Technical Specifications or an unreviewed safety question pursuant to 10CFR50.59(c). Matters of this kind shall be referred to the Nuclear Assessment Department by the Plant General Manager or by other functional organizational units within Carolina Power & Light Company prior to implementation.
- (3) Proposed changes to the Technical Specifications or this operating license, prior to implementation.
- (4) All reportable events.
- (5) Any other matter involving safe operation of the nuclear power plant that the Manager - Nuclear Assessment Department, deems appropriate for consideration of which is referred to the Manager - Nuclear Assessment Department, by the on-site operating organization or by other functional organizational units within Carolina Power & Light Company.
- (6) Reports and minutes of the PNSC.

6.5.2.2 Results of Nuclear Assessment Department reviews, including recommendations and concerns, shall be documented.

- a. Copies of documented reviews shall be retained in the NAD files.

RePlace ~~entire~~ Section 6.5.2 with attached

~~b) Recommendations and concerns shall be submitted to the Plant General Manager and Vice President - Robinson Nuclear <sup>Project</sup> ~~Plant~~ within 14 days of determination.~~

~~c) A summation of Nuclear Assessment Department recommendations and concerns shall be submitted to the Chairman/President; Executive Vice President - Power Supply; Senior Vice President - Nuclear Generation; Vice President - Robinson Nuclear <sup>Project</sup> ~~Plant~~; Plant General Manager; and others, as appropriate on at least a bimonthly frequency.~~

~~d) The Nuclear Assessment Department Independent Safety Review Program shall be conducted in accordance with written, approved procedures.~~

Replace entire Section 6.5.2 with attached

6.5.2.3 Personnel

- a. Personnel assigned responsibility for independent reviews shall be specified in technical disciplines and shall collectively have the experience and competence required to review problems in the following areas:
- (1) Nuclear power plant operations
  - (2) Nuclear engineering
  - (3) Chemistry and radiochemistry
  - (4) Metallurgy
  - (5) Instrumentation and control
  - (6) Radiological safety
  - (7) Mechanical and electrical engineering
  - (8) Administration controls
  - (9) Seismic and environmental
  - (10) Quality assurance practices
  - (11) Nondestructive Testing
- b. The following minimum experience requirements shall be established for those persons involved in the independent safety review program:
- (1) Manager of NAD - Bachelor of Science in engineering or related field and ten (10) years' related experience, including five (5) years' involvement with operation and/or design of nuclear power plants.
  - (2) Reviewers - Bachelor of Science in engineering or related field or equivalent and five (5) years' related experience.
- c. An individual may possess competence in more than one specialty area. If sufficient expertise is not available

Replace entire Section 6.5.2 with attached

~~within the Nuclear Assessment Department, competent individuals from other Carolina Power & Light Company organizations or outside consultants shall be utilized in performing independent reviews and investigations.~~

- ~~d. At least three persons, qualified as discussed in Specification 6.5.2.3.b, shall review each item submitted under the requirements of Section 6.5.2.1.d.~~
- ~~e. Independent safety reviews shall be performed by personnel not directly involved with the activity or responsible for the activity.~~

## INSERT TO PAGES 6.5-8 THROUGH 6.5-12.

### 6.5.2 Nuclear Assessment Section Independent Review Program

#### 6.5.2.1 FUNCTION

The Nuclear Assessment Section shall function to provide independent review of plant changes, tests, and procedures. In addition, the independent review function will verify that reportable events are investigated in a timely manner and corrected in a manner that reduces the probability of recurrence of such events and detect trends that may not be apparent to a day-to-day observer.

#### 6.5.2.2 ORGANIZATION

6.5.2.2.1 The individuals assigned responsibility for independent reviews shall be qualified in specific disciplines. These individuals shall collectively have the experience and competence required to review activities in the following areas:

- a) nuclear power plant operations
- b) nuclear engineering
- c) chemistry and radiochemistry
- d) metallurgy
- e) nondestructive testing
- f) instrumentation and control
- g) radiological safety
- h) mechanical and electrical engineering
- i) administrative controls
- j) seismic and environmental
- k) quality assurance practices
- l) other appropriate fields

6.5.2.2.2 The Manager - Nuclear Assessment Section shall have a bachelor's degree in an engineering or related field and, in addition, shall have a minimum of ten years' related experience, of which five years shall be in the operation and/or design of nuclear power plants.

6.5.2.2.3 The Independent Safety Review Program reviewers shall have a bachelor's degree in an engineering or related field or equivalent and, in addition, shall have a minimum of five years' related experience.

**INSERT TO PAGES 6.5-8 THROUGH 6.5-12 (CONTINUED).**

- 6.5.2.2.4 An individual may possess competence in more than one specialty area. If sufficient expertise is not available within the Nuclear Assessment Section, competent individuals from other CP&L organizations or outside consultants shall be utilized in performing independent reviews and investigations.
- 6.5.2.2.5 The documents submitted under 6.5.2.3 shall be reviewed by individuals meeting the requirements of 6.5.2.2.1 and 6.5.2.2.3 to ensure all applicable disciplines are encompassed. Multiple reviews will be conducted on documents where required to meet applicable disciplines of 6.5.2.2.1.
- 6.5.2.2.6 Independent safety reviews shall be performed by individuals not directly involved with the activity under review or responsible for the activity under review.
- 6.5.2.2.7 The Nuclear Assessment Section Independent Safety Review Program shall be conducted in accordance with written, approved procedures.

**6.5.2.3 REVIEW**

Nuclear Assessment Section shall perform reviews of the following:

- a. Written safety evaluations of changes in the facility as described in the UFSAR, changes in procedures as described in the UFSAR, and tests or experiments not described in the UFSAR which are completed without prior NRC approval under the provisions of 10 CFR 50.59(a)(1). These reviews are to verify that such changes, tests, or experiments do not involve a change in the Technical Specifications or an unreviewed safety question as defined in 10 CFR 50.59(a)(2). These reviews may be conducted after appropriate management approval, and implementation may proceed prior to completion of the review.
- b. Proposed changes in procedures required by these Technical Specifications, proposed changes in the facility, or proposed tests or experiments, any of which involve a change in the Technical Specifications or an unreviewed safety question pursuant to 10 CFR 50.59(a)(2) prior to implementation.
- c. Proposed changes to the Technical Specifications or this Operating License prior to implementation.

**INSERT TO PAGES 6.5-8 THROUGH 6.5-12 (CONTINUED).**

- d. Violations, deviations, and reportable events, which require reporting to the NRC in writing, such as:
  - 1. violations of applicable codes, regulations, orders, Technical Specifications, license requirements, or internal procedures or instructions have safety significance;
  - 2. significant operating abnormalities or deviations from normal or expected performance of plant safety-related structures, systems, or components; and
  - 3. reportable events, as specified in 10 CFR 50.73.
  
- e. Any other matter involving safe operation of the nuclear power plant that the Manager - Nuclear Assessment Section deems appropriate for consideration or which is referred to by the Manager - Nuclear Assessment Section by the on-site operating organization, PNSC, or by other functional organizational units within CP&L.

6.5.2.4

RECORDS

- (a) Results of Nuclear Assessment Section independent safety reviews shall be documented and retained.

Replace entire Section with 6.5.3 attached

~~6.5.3 Nuclear Assessment Department - Audit Program~~

~~6.5.3.1 The Nuclear Assessment Department shall perform audits of plant activities. Specific audit subjects are defined in Specification 6.5.3.2.d.~~

~~6.5.3.2 The Manager - Nuclear Assessment Department is charged with the overall responsibility for administering the audit program as follows:~~

~~a) Approves selection of the individual(s) to conduct audits.~~

~~b) Has access to the plant operating records and operating personnel in performing the audits.~~

~~c) Prepares and retains written records of audits.~~

~~d) Assures audits are conducted on the following subjects:~~

~~1) The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.~~

~~2) The training and qualifications of the entire facility staff at least once per 12 months.~~

~~3) The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems, or method of operation that affect nuclear safety at least once per 6 months.~~



Replace entire Section with 6.5.3 attached

- (4) The verification of compliance and implementation of the requirements of the Quality Assurance Program to meet the criteria of Appendix B, 10CFR50, at least once per 24 months.
  - (5) The Emergency Plan and implementing procedures at least once per 12 months.
  - (6) The Security Plan and implementing procedures at least once per 12 months.
  - (7) The Facility Fire Protection Program and implementing procedures at least once per 24 months.
  - (8) Any other area of facility operation considered appropriate by the Nuclear Assessment Department; the Executive Vice President - Power Supply; or the Senior Vice President - Nuclear Generation.
  - (9) The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months.
  - (10) The Offsite Dose Calculation Manual and implementing procedure at least once per 24 months.
  - (11) The Process Control Program and implementing procedures for solidification of radioactive wastes at least once per 24 months.
  - (12) The performance of activities required by the Quality Assurance Program to meet the criteria of Regulatory Guide 4.15, December 1977 at least once per 12 months.
- e. Distribute reports and other records to appropriate managers.

Replace entire Section with 6.5.3 attached

6.5.3.3 Audit Personnel

a) Audit personnel shall be independent of the area audited.

Selection for auditing assignments is based on experience or training that establishes that their qualifications are commensurate with the complexity or special nature of the activities to be audited. In selecting auditing personnel, consideration shall be given to special abilities, specialized technical training, prior pertinent experience, personal characteristics, and education.

b) Qualified outside consultants or other individuals independent from those personnel directly involved in plant operation shall be used to augment the audit teams when necessary. Individuals performing the audits may be members of the audited organization; however, they shall not audit activities for which they have immediate responsibility, and while performing the audit, they shall not report to a management representative who has immediate responsibility for the activity audited.

6.5.3.4 Results of plant audits are approved by the Manager - Nuclear Assessment Department and transmitted to the Executive Vice President - Power Supply; the Senior Vice President - Nuclear Generation; Vice President - Robinson Nuclear Project; Plant General Manager; and others, as appropriate within 30 days after the completion of the audit.

6.5.3.5 The Nuclear Assessment Department Audit Program shall be conducted in accordance with written, approved procedures.

**INSERT TO PAGES 6.5-13 THROUGH 6.5-15.**

6.5.3 Nuclear Assessment Section Assessment Program

6.5.3.1 Assessments of facility activities shall be performed by the Nuclear Assessment Section. Assessments will be performance based and will be scheduled based on plant performance and importance to safety but at a frequency not to exceed 24-months. These assessments shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
- b. The performance, training and qualifications of the entire facility staff.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety.
- d. The performance of activities required by the Quality Assurance Program to meet the criteria of Appendix B, 10 CFR 50.
- e. Any other area of facility operation considered appropriate by the Vice President - Robinson Nuclear Plant.
- f. The Fire Protection Program and implementing procedures.
- g. The Radiological Environmental Monitoring Program and the results thereof.
- h. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures.
- i. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes.

6.5.3.2 Assessments of activities prescribed by the Code of Federal Regulations will be performed at the frequencies prescribed by the applicable regulation. These assessments shall encompass:

- a. Emergency Preparedness (per 10 CFR 50.54(t))
- b. Security (per 10 CFR 50.54(p))

~~6.5.4~~

Outside Agency Inspection and Audit Program

~~6.5.4.1 An independent fire protection and loss prevention inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite personnel or an outside fire protection firm.~~

~~6.5.4.2 An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.~~

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for events requiring immediate notification:

a) The NRC shall be notified pursuant to the requirements of 10 CFR 50.72.

b) Each reportable event shall be reviewed in accordance with Specification 6.5.1.6.6 and submitted to the Manager - Nuclear Assessment Department, and the Vice President - Robinson Nuclear Project.  
Section PLANT

6.6.2 The following actions shall be taken for reportable events requiring a Licensee Event Report:

a) A report shall be submitted to the NRC pursuant to the requirements of 10 CFR 50.73.

b) Each reportable event shall be reviewed in accordance with Specification 6.5.1.6.6 and submitted to the Manager - Nuclear Assessment Department, and the Vice President - Robinson Nuclear Project.  
Section PLANT

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a safety limit is violated:

- a) The provisions of 10 CFR 50.72 shall be complied with.
- b) The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with.
- c) The safety limit violation shall be reported to the NRC Region II within one hour and the Vice President - Robinson Nuclear ~~Project~~ and the ~~Manager - Nuclear Assessment Department~~ within 24 hours. <sup>PLANT</sup>
- d) A Safety Limit Report shall be prepared. The report shall be reviewed in accordance with Specification 6.5.1.6.6. This report shall describe (1) applicable circumstances preceding the violation; (2) effects of the violation upon facility components, systems, or structures; and (3) corrective action taken to prevent recurrence.
- e) The Safety Limit Violation Report shall be submitted to the NRC, Vice President - Robinson Nuclear ~~Project~~, and the Manager - Nuclear ~~Assessment Department~~ within 14 days of the violation. <sup>PLANT</sup>  
<sub>Section</sub>

- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material released to the environs.
- f. Records of transient or operational cycles for those facility components designed for a limited number of transients or cycles.
- g. Records of training and qualification for current members of the plant staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required by the QA program.
- j. Records of review performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10CFR50.59.
- k. Records of (1) meetings of the PNSC, ~~(2) the independent reviews performed by the previous Corporate Nuclear Safety Section, and~~  
~~(3) the independent reviews performed by the Nuclear Assessment Department.~~
- l. Records of data results required by the radiological environmental monitoring program.

*m. Records of Independent Reviews*

February 23, 1995

Mr. C. S. Hinnant, Vice President  
Carolina Power & Light Company  
H. B. Robinson Steam Electric Plant,  
Unit No. 2  
3581 West Entrance Road  
Hartsville, South Carolina 29550

SUBJECT: INDIVIDUAL NOTICE REGARDING DEGRADED VOLTAGE RELAY SETPOINT CHANGE  
FOR H.B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2 (TAC NO. 90314)

Dear Mr. Hinnant:

Enclosed is a copy of the a notice sent to the Office of the Federal Register  
for publication regarding the August 23, 1994, request for a Technical  
Specification amendment for the degraded voltage relay setpoint change.

Sincerely,

(Original Signed By)

Brenda Mozafari, Project Manager  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-261

Enclosure: Individual Notice

cc w/enclosure: See next page

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NAME	PAnder <sup>son</sup>	BMozafari	WBal <sup>eman</sup>		
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COPY	Yes/No	Yes/No	Yes/No		

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Mr. C. S. Hinnant  
Carolina Power & Light Company

H. B. Robinson Steam Electric  
Plant, Unit No. 2

cc:

Mr. R. E. Jones  
General Counsel - Legal Department  
Carolina Power & Light Company  
Post Office Box 1551  
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U.S. Nuclear Regulatory Commission  
Resident Inspector's Office  
H. B. Robinson Steam Electric Plant  
2112 Old Camden Road  
Hartsville, South Carolina 29550

Mr. Max Batavia, Chief  
South Carolina Department of Health  
Bureau of Radiological Health  
and Environmental Control  
2600 Bull Street  
Columbia, South Carolina 29201

Regional Administrator, Region II  
U.S. Nuclear Regulatory Commission  
101 Marietta St., N.W., Ste. 2900  
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Mr. H. W. Habermeyer, Jr.  
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Post Office Box 1551  
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Plant General Manager  
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Mr. R. M. Krich  
Manager - Regulatory Affairs  
Carolina Power & Light Company  
H. B. Robinson Steam Electric Plant,  
Unit No. 2  
3581 West Entrance Road  
Hartsville, South Carolina 29550

UNITED STATES NUCLEAR REGULATORY COMMISSIONCAROLINA POWER & LIGHT COMPANYDOCKET NO. 50-261NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO  
FACILITY OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS  
CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR A HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. DPR-23 issued to Carolina Power & Light Company (the licensee) for operation of the H. B. Robinson Steam Electric Plant, Unit No. 2, located in Darlington County, South Carolina.

The proposed amendment would increase the degraded grid voltage relay (DGVR) setpoint to comply with revised voltage criteria established by Carolina Power & Light Company's alternating current auxiliary electrical distribution system voltage/load flow/fault current study. The DGVR setpoint will be changed from 415 plus or minus 4 volts to 430 plus or minus 4 volts. The revised criteria would provide a voltage setting such that continuous duty, safety-related motors will not be allowed to operate at terminal voltages below the voltage required for proper operation for periods of time greater than the time delay setting of the DGVR.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The increase in the DGVR setpoint will prevent motor operation at terminal voltages below which motor overheating and possible life reduction could occur, due to sustained offsite power degradation under the design basis plant operating scenario. The new setting ensures that the emergency buses are transferred to their respective diesel generators at offsite power voltage levels higher than allowed by the existing setting. Analysis has determined that the new DGVR setting will not result in unnecessary offsite power separations, due to motor starting transients, during normal power operation or postulated accident conditions.

The function of the DGVR remains unchanged. The design configuration of the DGVR circuit remains unchanged. The proposed amendment will increase the minimum voltage available at the safety buses and maintain safety related loads within their voltage requirements under degraded conditions.

The change to the DGVR trip setpoint also considered the minimum bus recovery voltage following a transient that would reset the relay to prevent unnecessary transfers to the emergency diesel generators. With the offsite system at the minimum predicted voltage, the DGVRs will reset following a motor starting transient. Therefore, there would be no increase in the probability or consequences of an accident previously evaluated.

2. The proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The higher DGVR setpoint improves the operation of continuous duty, safety-related motors, in that it ensures motor terminal voltages of sufficient value for proper operation. The higher setting has been evaluated against plant system transient voltage conditions under minimum predicted switchyard voltages and determined to result in no risk of spurious relay actuations. The proposed change is in the of DGVR only. The function of the DGVR circuit remains unchanged. Failure of the relays at their new setpoint would not change the failure analysis. The proposed amendment to the DGVR setpoint ensures appropriate automatic action will be taken in the event voltage sufficient to operate required vital electrical loads within acceptable voltage limits is not available. Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed amendment does not involve a significant reduction in a margin of safety.

The proposed amendment will increase the minimum voltage limit at the emergency buses. This increase in DGVR setpoint ensures that the minimum voltage requirements for vital loads will be available including under degraded offsite voltage conditions or automatic action will occur to restore voltage. Calculations have determined that the proposed setpoint meets current design requirements. Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the FEDERAL REGISTER a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By \_\_\_\_\_, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and

who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Hartsville Memorial Library, 147 West College Avenue, Hartsville, South Carolina 29550. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has

filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the

opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to William H. Bateman: petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this FEDERAL REGISTER notice. A copy



of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to R. E. Jones, General Counsel, Carolina Power & Light Company, Post Office Box 1551, Raleigh, North Carolina 27602, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated August 23, 1994, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Hartsville Memorial Library, 147 West College Avenue, Hartsville, South Carolina 29550.

Dated at Rockville, Maryland, this 23rd day of February 1995.

FOR THE NUCLEAR REGULATORY COMMISSION



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