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FACIL:50-261 H.B. Robinson Plant, Unit 2, Carolina Power & Light C 05000261
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ERNSTES,M.E. Region 2 (Post 820201)
RECIP.NAME RECIPIENT AFFILIATION
YOUNG,D.E. Carolina Power & Light Co.

SUBJECT: Discusses results of semiannual plant performance review of Robinson NPP completed on 980508.PPR involved participation of technical divs in evaluating insp results & safety performance for period of 971001-980331.

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TITLE: Systematic Assessment of Licensee Performance (SALP) Report

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June 4, 1998

Carolina Power & Light Company
ATTN: Mr. D. E. Young
Vice President
H. B. Robinson Nuclear Plant
3581 West Entrance Road
Hartsville, SC 29550

SUBJECT: PLANT PERFORMANCE REVIEW (PPR) - ROBINSON

Dear Mr. Young:

On May 8, 1998, the NRC staff completed the semiannual Plant Performance Review (PPR) of Robinson Nuclear Power Plant. The staff conducts these reviews for all operating nuclear power plants to develop an integrated understanding of safety performance. The results are used by NRC management to facilitate planning and allocation of inspection resources. The PPR for Robinson involved the participation of all technical divisions in evaluating inspection results and safety performance information for the period October 1, 1997, through March 31, 1998. PPRs provide NRC management with a current summary of licensee performance and serve as inputs to the NRC Systematic Assessment of Licensee Performance (SALP) and senior management meeting (SMM) reviews.

Enclosure 1 contains a historical listing of plant issues, referred to as the Plant Issues Matrix (PIM), that were considered during this PPR process to arrive at an integrated view of licensee performance trends. The PIM includes only items from inspection reports or other docketed correspondence between the NRC and Carolina Power & Light. The PPR may also have considered some predecisional and draft material that does not appear in the attached PIM, including observations from events and inspections that had occurred since the last NRC inspection report was issued, but had not yet received full review and consideration. This material will be placed in the PDR as part of the normal issuance of NRC inspection reports and other correspondence.

This letter advises you of our planned inspection effort resulting from the Robinson PPR review. It is provided to minimize the resource impact on your staff and to allow for scheduling conflicts and personnel availability to be resolved in advance of inspector arrival onsite. Enclosure 2 details our inspection plan for the next 8 months. The rationale or basis for each inspection outside the core inspection program is provided so that you are aware of the reason for emphasis in these program areas. Resident inspections are not listed due to their ongoing and continuous nature.

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During this scheduling cycle, we will be focusing some of our discretionary inspection effort on the resolution of open inspection items. Therefore, additional inspections may be conducted, which are not listed on Enclosure 2, to close open inspection items that are ready to be resolved. You will be notified at least 3 weeks prior to the start of these inspections.

The NRC's general policy for reactor inspections is that each inspection shall be announced, except when announcing the inspection could reasonably compromise the objectives of the inspectors. Therefore, some specific inspections, such as in the security and radiological protection areas, are not included on Enclosure 2 and may not be announced.

We will inform you of any changes to the enclosed inspection plan. If you have any questions, please contact me at (404)562-4540.

Sincerely,

(Original signed by M. Ernstes)

Michael E. Ernstes, Acting Chief
Reactor Projects Branch 4
Division of Reactor Projects

Docket Nos. 50-261
License Nos. DPR-23

Enclosures:
1. Plant Issues Matrix
2. Inspection Plan

cc w/encls:
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United States Nuclear Regulatory Commission
PLANT ISSUES MATRIX
by SALP Functional Area

ROBINSON

03-Jun-98

DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)	SMM CODES
OPERATIONS						
12/22/97	Negative		IR 97-12	NRC	The failure to properly implement Regulatory Guide commitments as descibed in the Updated Final Safety Analysis Report (UFSAR) associated with the Loose Parts Monitoring Program was considered similar to deficiencies and inconsistencies in the UFSAR and other licensing documents previously identified during the NRC's Architectural Engineering (A/E) Team Inspection. 4B	1 2 3 4 5 A ☐☐☐☒☐ B ☐☐☐☒☐ C ☐ ☐☒☐
12/22/97	Positive		IR 97-12	NRC	The onsite review functions of the Plant Nuclear Safety Committee (PNSC) were conducted in accordance with TS. The PNSC meetings were well coordinated and meeting topics were thoroughly discussed and evaluated. NAS continued to provide strong oversight of licensee activities. A NAS assessment of the Robinson Moter-Operated Valve program was thorough and resulted in important findings. 1C	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☐☐ C ☐ ☐☐☐
12/22/97	Positive		IR 97-12	NRC	The licensee's implementation and transition to the Improved Technical Specifications was good. 1C	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☐☐ C ☐ ☐☐☐
12/22/97	Positive		IR 97-12	NRC	Overall, licensee implementation of cold weather protection activities was satisfactory and the system engineer was familiar with plant cold weather protection equipment. Additionally, a Nuclear Assesment Section (NAS) audit of the plant cold weather protection activities was thorough, and condition reports were appropriately initiated to address several dificiencies that were identified by the NAS audit. 1A	1 2 3 4 5 A ☐☐☐☐☐ B ☒☐☐☐☐ C ☒ ☐☐☐
12/22/97	NCV		IR 97-12	NRC	Two Non-Cited Violations (NCVs) were identified involving operator failure to follow procedures while implementing Technical Specification (TS) surveillance requirements. While individually, each of these items had low safety consequence, they indicated weaknesses in operator log keeping accuracy and operator attention to detail. 1A/3A	1 2 3 4 5 A ☒☐☐☐☐ B ☐☐☐☐☐ C ☒ ☐☐☐
12/22/97	Positive		IR 97-12	NRC	Overall, operator response to the failure of a condensate pump shaft and subsequent automatic reactor trip was appropriate. Plant shutdown and startup activities were satisfactorily conducted. Ineffective corrective actions from a similar condensate pump shaft failure that occurred in 1991 resulted in the subsequent failure on November 16, 1997. 1B/5C	1 2 3 4 5 A ☐☒☐☐☐ B ☒☐☐☐☐ C ☐ ☐☐☐

DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)	SMM CODES
12/7/97	Positive	MAINT	IR 97-11	NRC	The overall approach to assessing the risk-impact of maintenance activities was considered adequate. 1C	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☐☐
12/7/97	Positive		IR 97-11	NRC	Licensed operators had a good understanding of the Maintenance Rule, understood how to use the matrix for taking equipment out-of-service, and understood their responsibilities for implementing the Maintenance Rule. 3B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☒☐☐ C ☐ ☐☐●
10/11/97	VIO		EEI 97-10-02, -03, Sect. O8.1, pg 6-9	NRC	Two apparent violations were identified for the failure to identify and correct a mispositioned EDG output breaker control switch prior to NRC identification, as well as the failure to assure adequate configuration controls of the switch position. The licensee determined that the switch was most likely mispositioned as a result of someone bumping the switch. Controls for verifing the switch position were weak. Enforcement Conference with the licensee was scheduled for 11/25/97.	1 2 3 4 5 A ☒☐☒☐☐ B ☐☐☐☐☐ C ☐ ☐☐☐
10/11/97	Negative	SAQV	IR 97-10, Sect. O7.1, pg 5-6	NRC	PNSC failed to review a 1996 NRC notification event (10CFR50.72) as required by TS. The event involved an offsite notification to SCDEHC related to a diesel fuel oil storage tank leak. An error was made in the classification of the Condition Report related to the issue. Instead of Level 1, the CR was classified as Level 3, which does not get automatic review by PNSC. The inspector determined this was an isolated occurrence.	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☒☐☐ C ☐ ☐☐☐
10/11/97	Positive		IR 97-10, Sect. O2.1and O2.2, pg 3-5	NRC	Walkdowns of plant electrical distribution and control room ventilation systems revealed that they were being operated in accordance with UFSAR and licensee lineup procedures. The systems were well maintained and TS surveillances were being met. Good licensee control and maintenance of systems important to safety.	1 2 3 4 5 A ☐☒☐☐☐ B ☐☐☐☐☐ C ☐ ☐☐●
10/11/97	Positive	ENG	IR 97-10, Sect. O1.3, pg 2-3	NRC	Licensee actions following the receipt of Loose Parts Monitoring alarms on SG A were responsive, thorough, and indicative of strong management attention. While the source of the noise was not identified, action plans were detailed and exhaustive. Good management attention and sensitivity on potential for loose part.	1 2 3 4 5 A ☐☐☐☐☒ B ☐☐☐☒☐ C ☐ ☐☐☐
10/11/97	Weakness	ENG	URI 97-10-01, Sect. O1.3, pg 2-3	NRC	Potential discrepancies were identified in the licensee's implementation of Loose Parts Monitoring System implementation and FSAR commitments to RG 1.133, with regards to testing and maintenance of LPMS. Failure to implement regulatory guide commitments as described in the UFSAR.	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☒☐☒ C ☐ ☐☐☐

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10/11/97	Negative		IR 97-10, Sect. O1.1, pg 1	NRC	Overall, control room operator logs were appropriately maintained; however, on instance was noted where the status of a seismic monitor was not appropriately logged. Operator inattention to detail	1 2 3 4 5 A ☐☐☒☐☐ B ☐☐☐☐☐ C ☐ ☐☐☐
MAINTENANCE						
3/12/98	Positive	OPS	IR 9801	NRC	Changes made to maintenance and operation procedures which implemented the more restrictive requirements delineated in the new Improved Technical Specifications were found to be effective and thorough.	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☒ ☐☐☐
3/12/98	Positive		IR 98-01	NRC	The process followed by the licensee in handling a pin hole leak in Component Cooling Water Line 10-AC-41 until a code repair could be performed was correct and thorough.	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☒ ☐☒☐
12/22/97	Positive		IR 97-12	NRC	Emergency diesel generator (EDG) maintenance management indicated advanced planning and careful attention to detail. EDG post-mainenance testing was performed in a thorough and professional manner. The licensee thoroughly researched potential sources of vibration experienced on the "B" EDG and planned further evaluations during an upcoming planned diesel maintenance outage. The "B" EDG vibration data exceeded the in-service limit but was well below the vendor recommended shutdown limit. 2B	1 2 3 4 5 A ☐☒☐☐☐ B ☐☐☐☒☐ C ☐ ☐☐☐
12/7/97	Positive		IR 97-11	NRC	Assessments of the Maintenance Rule were very good and provided effective monitoring of implementation of the Maintenance Rule. Audit personnel demonstrated good knowledge of Maintenance Rule requirements. 5A/5B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☐☐
12/7/97	Strength		IR 97-11	NRC	In general, plant material condition and housekeeping observed during walkdowns was excellent. Preservation of equipment by painting was considered to be good. The licensee initiated corrective actions for the minor discrepant conditions noted in the structural area. The overall excellent housekeeping and material condition was considered a strength. 2A	1 2 3 4 5 A ☐☒☐☐☐ B ☐☒☐☐☐ C ☐ ☐☐☐

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12/7/97	Weakness		IR 97-11	NRC	Industry-wide operating experience had been considered, where practical, and operating data had been properly captured. However, a weakness was identified in the licensee's program in the area of collecting and tracking unavailability for safety significant SSCs. 4B/2B/3A	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☒☐
12/7/97	Strength		IR 97-11	NRC	For (a)(2) SSCs, in general, detailed performance criteria had been properly established, suitable trending had been performed, and corrective actions were taken when SSCs failed to meet performance criteria or experienced failures. The overall detailed program, specifically monitoring at the function level, was considered a strength. 4B/2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☐●
12/7/97	VIO		IR 97-11	NRC	A violation was identified for failure to identify an unavailability period for two (a)(1) Maintenance Rule components. 4B/3A	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☒☐☐ C ☐ ☐☐☐
12/7/97	Positive		IR 97-11	NRC	In general, operating data was being properly captured, and industry-wide operating experience was considered, as appropriate. 4B/2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☐☐
12/7/97	Positive		IR 97-11	NRC	Corrective actions, goals, and monitoring were very detailed and comprehensive for all (a)(1) SSCs reviewed. 4B/5A	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☐●
12/7/97	Positive		IR 97-11	NRC	The licensee considered safety in establishment of goals and monitoring for the (a)(1) systems and components reviewed. 4B/2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☐☐
12/7/97	Negative		IR 97-11	NRC	Procedural instructions to the system engineers for how to collect and track unavailability time were considered weak. 4B/2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☒☐☐ C ☐ ☐☐☐

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12/7/97	Positive	ENG	IR 97-11	NRC	The licensee's method for balancing reliability and unavailabilty was satisfactory and met the intent of paragraph (a)(3) of the Maintenance Rule. 4B/2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☒☐☐ C ☐ ☐☒☐
12/7/97	Positive		IR 97-11	NRC	The licensee's plans for performing periodic evaluations and assessments met the requirements of the Maintenance Rule. 4B/2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☐☐
12/7/97	VIO	ENG	IR 97-11	NRC	A violation was identified for failure to include the switchyard relay building and the turbine exaust hood spray system within the scope of the Rule. 4B/2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☒☐☐ C ☐ ☐☐☐
12/7/97	Positive		IR 97-11	NRC	In general, required structures, systems, and components (SSCs) were included within the scope of the Rule. 4B/2B	1 2 3 4 5 A ☐☒☐☐☐ B ☐☐☐☐☐ C ☐ ☐☐☐
12/7/97	Positive		IR 97-11 (Maintenance Rule Team Inspection)	NRC	Overall, the inspection team concluded that the licensee had a comprehensive Maintenance Rule program that met the requirements of 10 CFR 50.65, and the program was being effectively implemented. 4B/2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☐☐
10/11/97	Strength		IR 97-10, Sect. M1.2, pg 10-11	NRC	The licensee's process for incorporating PSA/PRA information in the planning of online maintenance activities was considered a strength. Management has a strong commitment to utilizing PSA/PRA information in the planning and implementation of maintenance.	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☐☐ C ☐ ☐☒☐
ENGINEERING						
12/22/97	VIO		IR 97-12	NRC	A Violation was identified for the failure to adequately control post modification testing related to ESR 9500783. The post-modification testing failed to confirm that the cooling performance of the Containment Air Recirculation Cooling system was not adversely impacted, and that the system was capable of meeting its intended safety function as described in Section 9.4.3.1 of the licensee's UFSAR. 2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☒☐ C ☐ ☐☐☐

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12/22/97	MISC		IR 97-12	NRC	The licensee had not met its original commitment date for correcting adverse findings regarding its implementation of Generic Letter (GL) 89-10 identified during NRC Inspection 50-261/96-12. Progress toward correcting the findings by the new date proposed by the licensee was generally satisfactory. Some concerns were identified. These concerns and the outstanding findings from rpt 97-12 will be re-examined in a future insp which will assess the licensee's completion of GL8910. (2B)	1 2 3 4 5 A ☐☐☐☒☐ B ☐☐☐☒☐ C ☐ ☐☒☐
12/7/97	Strength	MAINT	IR 97-11	NRC	System engineer's knowledge of their systems and the Maintenance Rule was excellent. They understood how to apply the Maintenance Rule to their systems and were proactive in taking corrective actions. The system engineering area was considered a strength. 3B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☒☐ C ☐ ☐☐☐
12/7/97	Positive	OPS	IR 97-11	NRC	The licensee's process for ensuring that critical safety functions were available during planned outages was adequate. The use of Equipment Out-Of-Service (EOOS) Monitor to evaluate plant configurations was good. 1C	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☐☐ C ☐ ☒☐☐
12/7/97	Strength	OPS	IR 97-11	NRC	The approach, under paragraph (a)(3) of the Rule, to assessing the risk-impact of maintenance activities was good. The assignment and use of licensed operators in the planning process and to perform evaluations of planned configurations was a strength. 4B/2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☒☐ C ☐ ☐☐☐
12/7/97	Positive	MAINT	IR 97-11	NRC	The approach to risk-ranking for the Maintenance Rule was adequate. 4B/2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☒☐
12/7/97	Positive	MAINT	IR 97-11	NRC	The expert panel committee meeting discussions on covered topics were good, and the expert panel meeting minutes were well documented. 3A	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☒☐
12/7/97	Positive	OPS	IR 97-11	NRC	The method of assuring the assumptions for reliability and availability in the PRA are conserved was good. 4B/2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☐☐ C ☒ ☐☒☐

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12/7/97	Positive	MAINT	IR 97-11	NRC	The overall quantitative approach used to perform risk ranking for SSCs in the scope of the Maintenace Rule was good. Performance criteria were established with substantial probabilistic risk assessment (PRA) input. Documentation of PRA input was good. 4B/2B	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☒☐
10/11/97	NCV		NCV 97-10-05, Sect. E8.1, pg 14-15	LICENSEE	An NCV was identified against 10CFR50, App. B, Criterion III, Design Controls, for the lack of adequate design controls related to the licensee's repairs and alterations to the ECCS sump screens that resulted in degraded sump conditions. This issue was originally identified as URI 50-261/96-12-08.	1 2 3 4 5 A ☐☐☐☒☐ B ☐☐☐☐☐ C ☐ ☐☐●
10/11/97	VIO		EEI 97-10-04, Sect. E2.2, pg13-14	LICENSEE	Licensee actions upon discovery of USQ associated with certain SF shipping activities were adequate. An apparent violation was identified for the failure to meet 50.59 requirements for performing a change to shipping procedures involving a USQ Apparent violation was reviewed by NRC management and addressed in 11/7/97 correspondence where the NRC exercised enforcement discretion from issuing a violation	1 2 3 4 5 A ☐☐☐☐☒ B ☐☐☒☐☐ C ☐ ☐☐☐
10/11/97	Strength		IR 97-10, Sect. E2.1, pg 12	NRC	Licensee management asked probing questions during maintenance rule system review meetings that were recently initiated. The reivews sensitized appropriate managers to the problems related to the systems. Maintenance rule system reivew meetings should result in better focus and management of problems related to maintenance rule plant systems.	1 2 3 4 5 A ☐☐☐☐☒ B ☐☐☐☐☐ C ☐ ☐☒☐
10/11/97	Positive	MAINT	IR 97-10, Sect. E1.1, pg 11-12	NRC	The spent fuel pool anti-siphon modification was properly implemented including adequacy of 10CFR50.59, UFSAR updates, and post-mod. testing. This modification further enhances plant safety, in that, it eliminates the potential for a siphon induced draindown of the spent fuel pool.	1 2 3 4 5 A ☐☐☐☒☐ B ☐☐☐☒☐ C ☐ ☐☐●
PLANT SUPPORT						
12/31/97	Strength		IR 97-13	NRC	The controller/evaluator organization conducted an excellent critique process. 1C	1 2 3 4 5 A ☐☐☐☐☒ B ☐☐☐☐☐ C ☐ ☐☐☐
12/31/97	Strength		IR 97-13	NRC	Emergency Response Facilities were organized, equipped, and maintained in a manner that provided for the emergency response. 1C	1 2 3 4 5 A ☐☒☐☐☐ B ☐☐☐☐☐ C ☐ ☐☐☐

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12/31/97	Strength		IR 97-13	NRC	The Joint Information Center and its staff were activated and functioned in a manner that provided for the dissemination of coordinated and accurate information to the public via the news media. 1C	1 2 3 4 5 A ☐☐☐☐☐ B ☐☒☐☐☐ C ☐ ☐☐☐
12/31/97	Strength		IR 97-13	NRC	Provisions existed for the prompt communications among principal response organizaitons to emergency personnel, and they were effectively used during the exercise to provide timely information and coordinate emergency response. 1C	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☐☐ C ☐ ☐☐☐
12/31/97	Strength		IR 97-13	NRC	The licensee demonstrated the ability to make timely and concise initial and follow-up notifications to the States and counties. 1C	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☐☐ C ☐ ☐☐☐
12/31/97	Weakness		IR 97-13	NRC	The licensee had a standard system for emergency classifications and used it to effectively classify the off-normal events. An Exercise Weakness was identified for failure to promptly declare a Notification of Unusual Event. 1C	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☐☐ C ☐ ☐☐☐
12/31/97	Strength		IR 97-13	NRC	Predesignated personnel with well defined responsibilities promplty staffed the Emergency Response Facilities. 1C	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☐☐ C ☐ ☐☐☐
12/31/97	Strength		IR 97-13	NRC	The scenario developed for this exercise was effective for testing the integrated emergency response capability and exercise preparations were well organized. 1C	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☐☐ C ☐ ☐☐☐
10/11/97	Positive		IR 97-10, Sect. S1.2, pg 16-17	NRC	The licensee completed construction upgrades of the security firing range training facility. The licensee's efforts to upgrade the training facility was indicative of good management support to further enhance security personnel performance.	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☒☐ C ☐ ☐☐☐

DATE	TYPE(s)	SEC. SFA	SOURCE(s)	ID'd	ISSUE(s)	SMM CODES
10/11/97	Positive		IR 97-10, Sect. R1.2, pg 16	NRC	The licensee's corrective actions for problems related to personnel entering the Radiation Control Area included the implementation of new turnstiles at the Radiation Control Area to provide greater positive controls. The licensee's corrective actions for problems related to personnel entering the RCA without appropriate monitoring was considered aggressive.	1 2 3 4 5 A ☐☐☐☐☐ B ☐☐☐☐☐ C ☐ ☐☐☒

SMM Template Codes:

1A	OPERATION PERFORMANCE - Normal Operations
1B	OPERATION PERFORMANCE - Operations During Transients
1C	OPERATION PERFORMANCE - Programs and Processes
2A	MATERIAL CONDITION - Equipment Condition
2B	MATERIAL CONDITION - Programs and Processes
3A	HUMAN PERFORMANCE - Work Performance
3B	HUMAN PERFORMANCE - KSA
3C	HUMAN PERFORMANCE- Work Environment
4A	ENGINEERING/DESIGN - Design
4B	ENGINEERING/DESIGN - Engineering Support
4C	ENGINEERING/DESIGN - Programs and Processes
5A	PROBLEM IDENTIFICATION & SOLUTION - Identification
5B	PROBLEM IDENTIFICATION & SOLUTION - Analysis

SALP Functional Areas:

ENG	ENGINEERING
MAINT	MAINTENANCE
OPS	OPERATIONS
PLT SUP	PLANT SUPPORT

ID Code:

LICENSEE	LICENSEE
NRC	NRC
SELF	SELF-REVEALED

EEIs are apparent violations of NRC requirements that are being considered for escalated enforcement action in accordance with the "General Statement of Policy and Procedure for NRC Enforcement Action" (Enforcement Policy), NUREG-1600. However, the NRC has not reached its final enforcement decision on the issues identified by the EEIs and the PIM entries may be modified when the final decisions are made. Before the NRC makes its enforcement decision, the licensee will be provided with an opportunity to either (1) respond to the apparent violation or (2) request a predecisional enforcement conference.

URIs are unresolved items about which more information is required to determine whether the issue in question is an acceptable item, a deviation, a nonconformance, or a violation. However, the NRC has not reached its final conclusions on the issues, and the PIM entries may be modified when the final conclusions are made.

ROBINSON
INSPECTION PLAN

INSPECTION PROCEDURE/ TEMPORARY INSTRUCTION	TITLE/PROGRAM AREA	NUMBER OF INSPECTORS	PLANNED INSPECTION DATES	TYPE OF INSPECTION - COMMENTS
IP 81001	SECURITY	2	6/98	REGIONAL INITIATIVE INSPECTION TO REVIEW ACCESS AUTHORIZATION AT ALL CP&L SITES
IP 92902	MAINTENANCE	1	6/98	REGIONAL INITIATIVE INSPECTION TO REVIEW MAINT. RULE PERIODIC ASSESSMENT
IP 84750,IP 86750, IP 60855	RADIATION PROTECTION	1	6/98	CORE INSPECTION
IP 84750, IP 86750	RADIATION PROTECTION	1	7/98	CORE INSPECTION
IP 81700	SECURITY	1	7/98	CORE INSPECTION
IP 92903	ENGINEERING	1	8/98	REGIONAL INITIATIVE - IFI FOLLOWUP (MOVs)
IP 82701	EMERGENCY PREPAREDNESS	1	9/98	CORE INSPECTION
TI 2515/138	OPERATIONS	TBD	TBD	EVALUATION OF THE CUMULATIVE EFFECT OF OPERATOR WORKAROUNDS
IP 40500	CORRECTIVE ACTION	3	7/98	CORE INSPECTION

TBD = To Be Determined

Enclosure 2