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REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9508040061 DOC.DATE: 95/07/31 NOTARIZED: NO DOCKET #
FACIL:50-261 H.B. Robinson Plant, Unit 2, Carolina Power & Light C 05000261 P
AUTH.NAME AUTHOR AFFILIATION
GARROU,A.L. Carolina Power & Light Co. R
YOUNG,D.E. Carolina Power & Light Co. I
RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 95-004-00:on 950630,reactor tripped due to MSIV closure.
Caused by loose fuse block fuse clip.Walkdown of other O
circuits on 125 Vdc auxiliary panel conducted.W/950731 ltr.

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**Carolina Power & Light Company**

Robinson Nuclear Plant
3581 West Entrance Road
Hartsville SC 29550

Robinson File No.: 13510C
Serial: RNP-RA/95-0145

JUL 31 1995

United States Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261/LICENSE NO. DPR-23
LICENSEE EVENT REPORT NO. 95-004-00

Gentlemen:

The enclosed Licensee Event Report (LER), is submitted in accordance with 10 CFR 50.73. This report is required to be submitted to the NRC by July 31, 1995.

Very truly yours,

Dale E. Young
Plant General Manager

DTG:dtg
Enclosure

c: Mr. S. D. Ebner, Regional Administrator, USNRC, Region II
Ms. B. L. Mozafari, USNRC Project Manager, HBRSEP
Mr. W. T. Orders, USNRC Senior Resident Inspector, HBRSEP

9508040061 950731
PDR ADCK 05000261
S PDR
0-10038

Highway 151 and SC 23 Hartsville SC

IE22
11

NRC FORM 366
(5-92)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 5/31/95

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH
THIS INFORMATION COLLECTION REQUEST: 50.0 HRS.
FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO
THE INFORMATION AND RECORDS MANAGEMENT BRANCH
(MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION,
WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK
REDUCTION PROJECT (3150-0104), OFFICE OF
MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NUMBER (2)

050-261

PAGE (3)

1 OF 3

TITLE (4)

REACTOR TRIP DUE TO MAIN STEAM ISOLATION VALVE CLOSURE

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER	
06	30	95	95	-- 004 --	00	07	31	95	FACILITY NAME	DOCKET NUMBER 05000	
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)								
N			20.402(b)		20.405(c)		X		50.73(a)(2)(iv)		73.71(b)
POWER LEVEL (10)			20.405(a)(1)(i)		50.36(c)(1)				50.73(a)(2)(v)		73.71(c)
100			20.405(a)(1)(ii)		50.36(c)(2)				50.73(a)(2)(vii)		OTHER
			20.405(a)(1)(iii)		50.73(a)(2)(i)				50.73(a)(2)(viii)(A)		(Specify in Abstract below and in Text, NRC Form 366A)
			20.405(a)(1)(iv)		50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)		
			20.405(a)(1)(v)		50.73(a)(2)(iii)				50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME

A. L. Garrou: Acting Manager - Licensing/Regulatory Programs

TELEPHONE NUMBER (Include Area Code)

(803) 383-1544

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	SB	FUB		Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES
(If yes, complete EXPECTED SUBMISSION DATE).

X NO

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On June 30, 1995, H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2 was operating at 100% power. At 1426 hours, a reactor trip occurred as the result of an inadvertent closure of "B" Main Steam Isolation Valve (MSIV) MS-V1-3B. Upon closure of MS-V1-3B, the Reactor Protection System (RPS) reactor trip signal was received from Low - Low "B" Steam Generator level. Following the reactor trip, Operations personnel placed the plant in hot shutdown condition in accordance with plant procedures.

This event was caused by component failure. Subsequent investigation found that the MSIV closure was caused by a loose fuse block fuse clip for the fuse that supplies control power to the MSIVs actuator "open" air supply solenoid valve. The loss of power occurred while Operations personnel were reinstalling a fuse on a nearby circuit of the same fuse block on the same 125 VDC Auxiliary Panel for an unrelated piece of equipment. Following relocation of the two affected circuits that were identified and post maintenance testing, the plant was returned to 100 percent power.

This event is reported pursuant to 10 CFR 50.73(a)(2)(iv) as an RPS actuation.

NRC FORM 366A
(5-92)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 5/31/95LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
H. B. ROBINSON, UNIT NO. 2	050-261	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
		95	004	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

I. DESCRIPTION OF EVENT

On June 30, 1995, H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2 was operating at 100% power. Operations personnel were realigning a Local Clearance on the 125 VDC Auxiliary Panel (EIIIS Code: EJ) GC, Circuit 3, Chemical and Volume Control System (CVCS) (EIIIS Code: CB) valve CVC-204B (EIIIS Code: ISV). When the operator installed the CVC-204 control power fuses (EIIIS Code: FU) into the fuse block Circuit 3 fuse holder (EIIIS Code: FUB), sufficient pressure was simultaneously exerted on a nearby Circuit 1 fuse holder for "B" Main Steam Isolation Valve (MSIV) (EIIIS Code: ISV) MS-V1-3B actuator "open" air solenoid valve (EIIIS Code: PSV). The exerted pressure caused a momentary break in electrical contact of the Circuit 1 fuse. As a result, MS-V1-3B "open" air solenoid valve repositioned allowing control air to bleed off the MSIVs actuator. MS-V1-3B began to close due to the force from the closing spring and was driven closed once it entered the steam flow stream. Upon closure of the MSIV, the resulting Low - Low "B" Steam Generator level initiated a Reactor Protection System (RPS) (EIIIS Code: JE) reactor trip signal. Following the reactor trip, Operations personnel placed the plant in hot shutdown condition in accordance with plant procedures.

The NRC was notified of this event at 1511 hours via the Emergency Notification System in accordance with 10 CFR 50.72(b)(2)(ii).

II. CAUSE OF EVENT

This event was caused by a component failure. Subsequent investigation found that the MS-V1-3B inadvertent closure was caused by a fuse block fuse clip that supplies control power to the MSIV's "open" air supply solenoid valve and that had become loose during prior installation/removal for normal maintenance activities. The loss of control power occurred when an operator was realigning a Local Clearance on a nearby circuit of the same fuse block in the 125 VDC Auxiliary Panel GC for an unrelated piece of equipment. As part of the root cause investigation, a second loose fuse clip was found. This fuse clip was for the "C" MSIV "open" air supply solenoid valve. Following relocation of the two affected circuits that were identified and post maintenance testing, the plant was returned to 100 percent power.

NRC FORM 366A
(5-92)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
H. B. ROBINSON, UNIT NO. 2	050-261	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	3 OF 3
		95	004	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

III. ANALYSIS OF EVENT

The MSIVs serve to limit an excessive Reactor Coolant System cooldown rate and the resulting reactivity insertion, as well as the release of radioactive material in the event of a steam generator tube rupture. The Updated Final Safety Analysis Report Chapter 15 safety analysis for an inadvertent MSIV closure states that the consequences of this event are bounded by the results of the safety analysis for the loss of external electrical load event. All protective systems operated as designed.

This event is reported pursuant to 10 CFR 50.73(a)(2)(iv) as an RPS actuation.

IV. CORRECTIVE ACTIONS

A walkdown of the other circuits on the 125 VDC Auxiliary Panel was conducted. One additional circuit (i.e., 125 VDC Auxiliary Panel GC, Circuit 2, MSIV MS-V1-3C) was identified that also appeared to have a loose fuse clip.

The power supplies for Auxiliary Panel GC, Circuits 1 and 2 were relocated, and testing was performed to verify operability of the affected MSIVs.

V. ADDITIONAL INFORMATION

A. Failed Component Information

Fuse Block (EIIS Code: FUB)

B. Previous Similar Events

None