

**PRIORITY 1**  
ACCELERATED RIDS PROCESSING

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9501300057      DOC. DATE: 95/01/20      NOTARIZED: NO      DOCKET #  
 FACIL: 50-261 H.B. Robinson Plant, Unit 2, Carolina Power & Light C      05000261  
 AUTH. NAME      AUTHOR AFFILIATION  
 KRICH, R.M.      Carolina Power & Light Co.  
 RECIP. NAME      RECIPIENT AFFILIATION  
                          Document Control Branch (Document Control Desk)

SUBJECT: Submits 30-day rept per 10CFR50.46(a)(3)(ii) for plant, Unit 2, due to changes made in ECCS evaluation models.

DISTRIBUTION CODE: A001D      COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 3  
 TITLE: OR Submittal: General Distribution

NOTES:

	RECIPIENT		COPIES		RECIPIENT		COPIES	
	ID CODE/NAME		LTR	ENCL	ID CODE/NAME	LTR	ENCL	
	PD2-1 LA		1	1	PD2-1 PD	1	1	
	MOZAFARI, B		1	1				
INTERNAL:	<del>ELLE-CENTER</del> 01		1	1	NRR/DRCH/HICB	1	1	
	NRR/DSSA/SPLB		1	1	NRR/DSSA/SRXB	1	1	
	NUDOCS-ABSTRACT		1	1	OGC/HDS3	1	0	
EXTERNAL:	NOAC		1	1	NRC PDR	1	1	

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK, ROOM P1-37 (EXT. 504-2083) TO ELIMINATE YOUR NAME FROM DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTR 11 ENCL 10

P  
R  
I  
O  
R  
I  
T  
Y  
  
1  
  
D  
O  
C  
U  
M  
E  
N  
T

*MAY*



10 CFR 50.46

**Carolina Power & Light Company**  
Robinson Nuclear Plant  
3581 West Entrance Road  
Hartsville SC 29550

File No: 13510  
Serial: RNP-RA/95-004

**JAN 20 1995**

United States Nuclear Regulatory Commission  
ATTENTION: Document Control Desk  
Washington, DC 20555

**H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2**  
**DOCKET NO. 50-261/LICENSE NO. DPR-23**  
**THIRTY-DAY REPORT PURSUANT TO 10 CFR 50.46**

Gentlemen:

The purpose of this letter is to provide a 30-day report in accordance with 10 CFR 50.46(a)(3)(ii) for the H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2, due to changes made in our Emergency Core Cooling System (ECCS) evaluation models.

The ECCS evaluation model for the Small Break Loss-of-Coolant Accident (LOCA) has been reanalyzed. A summary of the new peak clad temperature, hot channel factor ( $F_{\Delta H}$ ) and the total core power peaking factor ( $F_Q^T$ ) results is contained in Enclosure 1.

This Small Break LOCA analysis uses the Siemens Power Corporation (SPC) calculation methodology approved by letter from Gary M. Holahan, NRC Director of Division of Systems Safety and Analysis, to R. A. Copeland of SPC, dated October 3, 1994.

The new plant specific Small Break LOCA analysis report is EMF-94-203(P), "H. B. Robinson Unit 2 Small Break LOCA Analysis," dated October 1994, and is proprietary to SPC. The limiting break size has increased from a 2 inch equivalent diameter area to a new limiting break size of 2.5 inches. A maximum Steam Generator tube plugging of 6% is included in the basis of the calculation.

270048

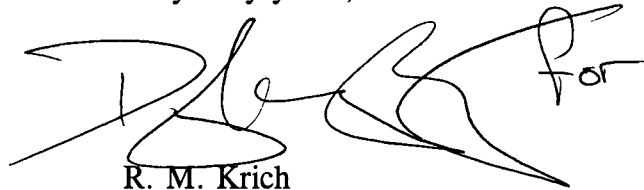
9501300057 950120  
PDR ADDEK 05000261  
R PDR

Highway 151 and SC 23 Hartsville SC

*A001*

Please refer any questions regarding this submittal to Mr. K. R. Jury at (803) 857-1363.

Very truly yours,

A handwritten signature in black ink, appearing to be 'R. M. Krich', with a large flourish at the end. To the right of the signature, the word 'for' is written in a smaller, cursive script.

R. M. Krich  
Manager - Regulatory Affairs

RES:res

Enclosure

c: Mr. S. D. Ebnetter, Regional Administrator, USNRC, Region II  
Ms. B. L. Mozafari, USNRC Project Manager, HBRSEP  
Mr. W. T. Orders, USNRC Senior Resident Inspector, HBRSEP

Enclosure to Serial: RNP-RA/95-004

COMPARISON OF PEAK CLADDING TEMPERATURES ANALYSES

	Peak Cladding Temperature	$F_{\Delta H}$	$F_Q^T$
New Small Break LOCA Analysis	1820°F	1.80	2.50
Previous Small Break LOCA Analysis (first half of cycle)	2033°F	1.70	2.40
Previous Small Break LOCA Analysis (second half of cycle)	2154°F	1.70	2.32