



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

January 28, 1994

Docket Nos. 50-325, 50-324
50-261, 50-400

Mr. W. S. Orser, Executive Vice President
Nuclear Generation
Carolina Power & Light Company
Post Office Box 1551
Raleigh, North Carolina 27602

SUBJECT: GENERIC LETTER 89-10, SUPPLEMENT 5, "INACCURACY OF MOTOR-OPERATED VALVE DIAGNOSTIC EQUIPMENT" - BRUNSWICK STEAM ELECTRIC PLANT, UNITS 1 AND 2, H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2, AND SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1 (TAC NOS. M87924, M87925, M87995 and M87954)

On June 28, 1993, the NRC staff issued Supplement 5, "Inaccuracy of Motor-Operated Valve Diagnostic Equipment," to Generic Letter (GL) 89-10, "Safety-Related Motor-Operated Valve Testing and Surveillance," requesting nuclear power plant licensees and construction permit holders (1) to re-examine their motor-operated valve (MOV) programs and to identify measures taken to account for uncertainties in properly setting valve operating thrust to ensure operability, and (2) to evaluate the schedule necessary to consider the new information on MOV diagnostic equipment inaccuracy and to take appropriate action in response to that information. Within 90 days of receipt of Supplement 5 to GL 89-10, the licensees were required (1) to notify the NRC staff of the diagnostic equipment used to confirm the proper size, or to establish settings for safety-related MOVs, and (2) to report whether they had taken actions or planned to take actions (including schedule) to address the new information on the accuracy of MOV diagnostic equipment.

The staff has reviewed the responses and has found that, for the most part, the licensees and permit holders have been actively addressing the uncertainties regarding the accuracy of MOV diagnostic equipment. The increased inaccuracy of MOV diagnostic equipment can raise questions regarding (1) the adequacy of torque switch settings to provide sufficient thrust while not exceeding thrust or torque structural limits and (2) the capability of actuator motors at current settings. In their responses, the licensees and permit holders indicated that many MOVs had the potential for underthrusting or overthrusting as a result of the higher than expected inaccuracy of MOV diagnostic equipment. Consequently, some licensees reported that MOVs have been retested, adjusted, or modified to resolve the concerns regarding the accuracy of MOV diagnostic equipment.

In your September 28, 1993, response to Supplement 5, you stated that for the Brunswick Steam Electric Plant, Units 1 and 2 (Brunswick), H. B. Robinson Steam Electric Plant, Unit No. 2 (Robinson), and Shearon Harris Nuclear Power Plant, Unit 1 (Harris), you use Liberty Technologies' VOTES equipment for MOV diagnostic testing. Your response also stated that all data for these facilities has been evaluated using the VOTES 2.3 software. Based on this

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- 2 -

review, the staff finds your response acceptable and during a future inspection will discuss your resolution of the MOC diagnostic equipment accuracy issue. Particularly, the staff will discuss your evaluation of MOVs set up with previous versions of the VOTES software. This completes the staff's effort on TAC NOS. M87924, M87925, M87995 and M87954.

If you have any questions regarding this issue, please contact Patrick D. Milano at (301) 504-1457 with questions regarding Brunswick, Brenda L. Mozafari at (301) 504-2020 with questions regarding Robinson, or N. B. Le at (301) 504-1458 with questions regarding Shearon Harris.

Sincerely,



S. Singh Bajwa, Director
Project Directorate II-1
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Office of Nuclear Reactor Regulation

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- 2 -

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Sincerely,

Original Signed by:
S. Singh Bajwa, Director
Project Directorate II-1
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