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SUBJECT: Submits addl info re Unit 2 individual plant exam as discussed in recent conference calls.Encl 1 Page 33 of 81 of encl to ltr RNP/93-2144,inadvertently omitted from submittal to NRC.

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Carolina Power & Light Company
Robinson Nuclear Plant
PO Box 790
Hartsville SC 29550

November 8, 1993

Robinson File No.: 13510
Serial: RNP/93-2801

United States Nuclear Regulatory Commission
ATTENTION: Document Control Desk
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H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261/LICENSE NO. DPR-23
ADDITIONAL INFORMATION CONCERNING INDIVIDUAL PLANT EXAMINATION

Gentlemen:

The purpose of this letter is to submit additional information concerning the H. B. Robinson Steam Electric Plant, Unit No. 2 Individual Plant Examination as discussed in recent conference calls between Carolina Power & Light Company and the NRC. Enclosure 1 is Page 33 of 81 of the Enclosure to Letter RNP/93-2144, inadvertently omitted from that submittal to the NRC. Enclosure 2 is a tabulation of initiating events and their contribution to overall core damage frequency as requested by the NRC during an October 28, 1993, conference call.

Questions regarding this matter may be referred to Mr. Jan S. Kozyra at (803) 383-1872.

Very truly yours,

David B. Waters
Manager - Regulatory Affairs

JSK:lst
Enclosures
c: Mr. S. D. Ebneter
Ms. B. L. Mozafari
Mr. W. T. Orders

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ENCLOSURE 1

QUESTION FE-6:

While the description of the analyses for loss of Heating, Ventilation, Air Conditioning (HVAC) considers the impact of temperature rise on the pumps in the areas considered, it does not address its impact on instrumentation and controls in these areas, nor its impact on valves. Provide a discussion of your consideration of these items in the criteria used for eliminating loss of HVAC as a possible cause for failure of the systems to be used for shut down or accident mitigation.

RESPONSE TO FE-6:

The loss of HVAC was considered as a possible cause of failure for systems required to safely shutdown the plant. Since the IPE submittal, several heatup analyses were performed to calculate the maximum room temperatures in areas requiring further assessment. The heatup analyses are referenced below. The impact of temperature rise on equipment needed for safe shutdown was analyzed and is also referenced below. The analyses referenced for equipment survivability indicate that all critical equipment will remain operable at the maximum room temperature.

Area Requiring Heatup Analysis	Reference Heatup Analysis	Maximum Room Temperature (°F)	Reference For Vital Equipment Survivability
AFW Pump Room	RNP-M/HVAC-1029[4]	114.2	EE 89-018
RHR Pump Room	88-023[4]	172.5	EQDP[2] 8.1
SI Pump Room	89-006[4]	213.4	EQDP[2] 8.1
CCW Pump Room	NF-1693.001[5]	141.0	SBCS[1]
Charging Pump Room	NF-1693.001[5]	124.0	SBCS[1]
E1/E2 Area	NF-907.01[5]	122.0	E1/E2 Eval.[3]
Battery Room	NF-1693.002[5]	143.0	SBCS[1]
Hagan Rack Room	92-004[4]	110.0	SBCS[1]
Cable Spread Room	SBCS[1]	123.0	SBCS[1]

1. Station Blackout Coping Study (Calc. 8S43-M-03)
2. Environmental Qualification Documentation Package
3. E1/E2 50 °C Equipment Evaluation Report
4. Engineering Evaluations performed by the Nuclear Engineering Department
5. Heatup Analysis using GOTHIC performed by the Nuclear Fuels Section

ENCLOSURE 2

List of initiating events and their contribution to the overall core damage frequecy

Initiator	Core Damage Frequency	Description
T5	5.73E-05	Loss of offsite power
M	5.23E-05	Medium break LOCA
FL-2	3.59E-05	Level 226 in hallwary 15,000 GPM service water flood
T9	2.90E-05	Loss of service water
T12A	1.98E-05	Loss of emergency AC bus E1
T11	1.63E-05	Loss of component cooling water
A	1.59E-05	Large break LOCA
FL-7	1.37E-05	CCW pump room 44,000 GPM service water flood
T12B	1.09E-05	Loss of emergency AC bus E2
T1	8.63E-06	Reactor trip
T4	7.17E-06	Loss of main feedwater
S	7.00E-06	Small break LOCA
FL-4	6.64E-06	Level 226 in hallway 4,000 GPM service water flood
R	5.73E-06	Steam generator tube rupture
T3	5.52E-06	Turbine trip
FL-8	5.21E-06	CCW pump room 15,000 GPM sevice water flood
T10	4.28E-06	Common service water header pipe rupture
V	4.00E-06	Interfacing system LOCA (RHR suction line)
T7	3.82E-06	Reactor trip withe safety injection
FBAT	2.40E-06	Battery room eyewash station spray event
T13A	2.37E-06	Loss of DC bus A
T2	1.70E-06	Reactor trip with RCS pressure challenge
T15	1.32E-06	Loss of CVCS
T13B	1.29E-06	Loss of DC bus B
FREL	1.10E-06	Relay room water spray event
FL-6	9.91E-07	Level 226 in hallway 4,000 GPM fire water flood
FL-9	9.30E-07	Electric equipment room fire hose rack spary
FL-1	9.04E-07	Level 226 in hallway 44,000 GPM service water flood
Exloca	5.00E-07	Excessive LOCA
T6	1.99E-07	Secondary line break
T14	1.54E-07	Loss of instrument air
T8	1.17E-07	Inadvertent containment phase B isolaion
Total Freq:	3.23E-04	