

Phase II Final Status Survey Report Mallinckrodt Columbium-Tantalum Plant

St. Louis, Missouri

Chapter 33

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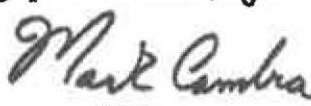
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ABBREVIATIONS AND ACRONYMS

%	percent
β	beta
AECOM	AECOM Technical Services
bgs	below grade surface
C-T	columbium-tantalum
CFR	Code of Federal Regulations
cpm	counts per minute
DCGL	derived concentration guideline level
DP	decommissioning plan
dpm	disintegrations per minute
dpm/100 cm ²	disintegrations per minute per 100 square centimeters
DQO	data quality objectives
EMC	elevated measurement comparison
FSS	Final Status Survey
FSSR	Final Status Survey Report
keV	kiloelectron-volt
m ²	square meters
MARSSIM	Multi-Agency Radiation and Site Investigation Manual (NUREG-1575)
min	minute
NIST	National Institute of Standards and Technology
NRC	U.S. Nuclear Regulatory Commission
SOF	sum of fractions
Tc	technetium
WRS	Wilcoxon Rank Sum

33.0 RESULTS SUMMARY FOR PLANT 7 PAVEMENT SU4

This chapter of the Final Status Survey Report (FSSR) presents the results of the final status survey (FSS) and data assessment for Plant 7 pavement survey unit SU4 in accordance with Columbium-Tantalum (C-T) Phase II Decommissioning Plan (DP) Section 14.5. The FSS for this Class 2 survey unit was completed by AECOM Technical Services (AECOM) in September and October of 2011. The SU4 data assessment was performed based on the assumptions, methods, and performance criteria established to satisfy the data quality objectives (DQOs) in accordance with the C-T Phase II DP Section 14.4.3.8. The summary statistics provide numerical values for measures of central tendency (i.e., mean, median), variation (i.e., standard deviation), and spread (i.e., minimum, maximum). Data evaluation and statistical analyses were performed and a separate decision was made for each survey unit of the C-T Plant as to its suitability for release for unrestricted use based upon the industrial use scenario release criterion as established in C-T Phase II DP Chapter 5.

33.1 OVERVIEW

SU4 is a Class 2 survey unit that consists of three principle areas within Plant 7: The Weir Chamber, The Grit Chamber, and the Pump Vault. All three of these areas dealt with wastewater handling as part of the C-T process. The Weir Chamber is situated north of the east and west Plant 7 basins and immediately east of The Grit Chamber. The Weir Chamber is an entirely underground structure. The grit chamber is an above ground structure north of the west Plant 7 basin, west of the Weir Chamber, and east of the Pump Vault. The Pump Vault is a mostly underground brick-lined structure north of the west Plant 7 basin and immediately west of the Grit Chamber. The C-T Phase II DP calls out sewerage from Plant 5 and extending downstream to the Wastewater Treatment Basins is expected to satisfy an FSS and may remain in service. The Weir Chamber, The Grit Chamber, and the Pump Vault are part of the downstream sewerage.

The survey unit is approximately 234 square meters (m²) in size, which is less than the size limit of 10,000 m² for Class 2 survey units for pavement (per C-T Phase II DP, Table 14-4). Class 2 was the appropriate classification because the survey unit had potential residual radioactivity that was not expected to exceed the DCGL_w. Figure 33-1 shows the location of SU4 within the Plant 7W area.

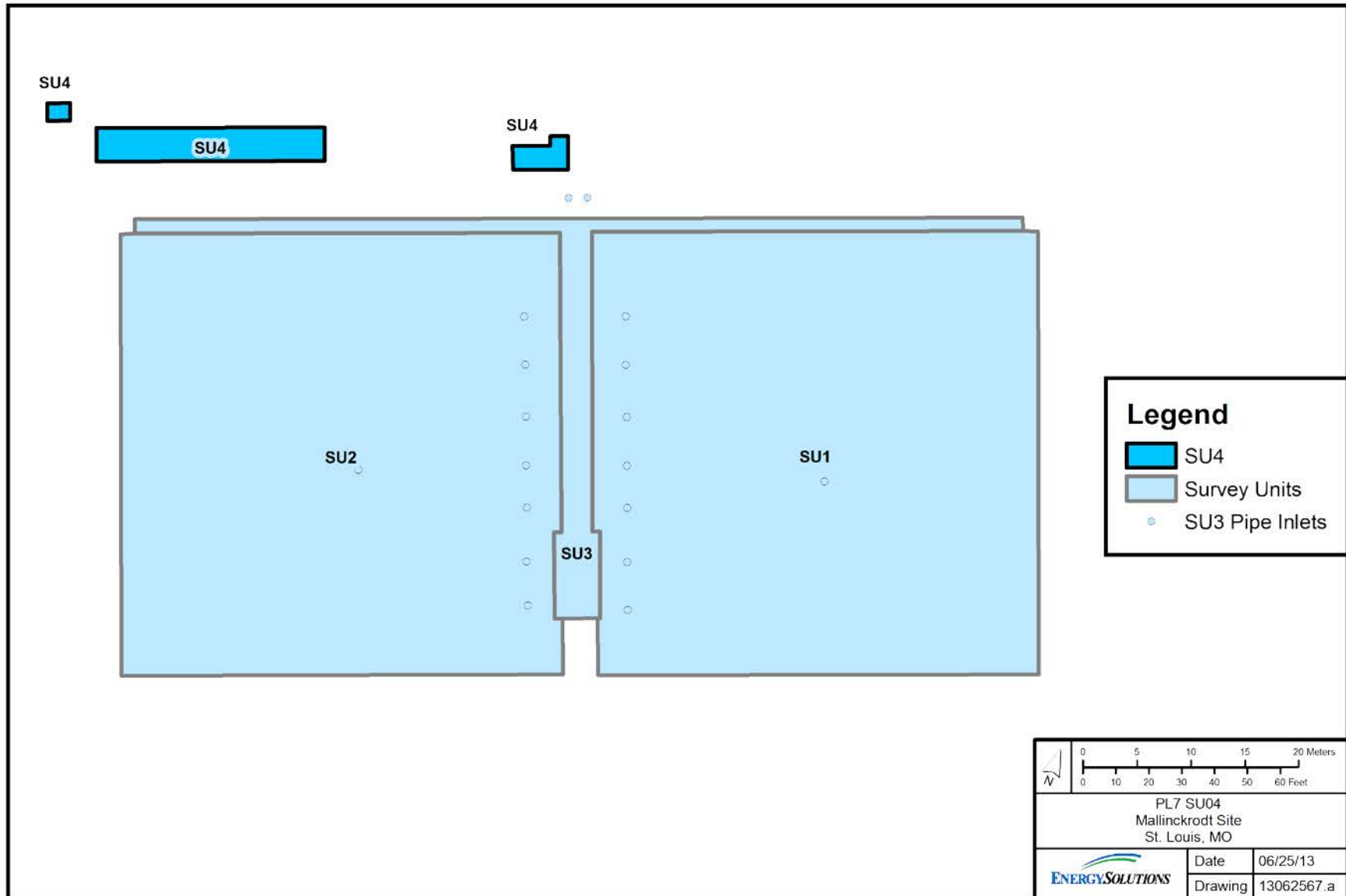


Figure 33-1 Location of Pavement SU4 in C-T Plant 7W

33.1.1 Weir Chamber

The Weir Chamber is a 5-foot (ft) deep underground “L” shaped concrete chamber approximately 18 ft long and 15 ft wide. It features a 12-inch pipe on the south end of the west wall. It is accessed by two 3-ft 7-inch square openings. When not pumped out, water collects several inches deep in the chamber before emptying farther into the storm sewer system. Figure 33-2 through Figure 33-6 are photographs of the Weir Chamber and features.



Figure 33-2 Photograph of Access to the Weir Chamber Looking North



Figure 33-3 Photograph of Access to the Weir Chamber Looking West



Figure 33-4 Photograph of 12-inch Pipe at Bottom of the Weir Chamber



Figure 33-5 Photograph of Main Section of Weir Chamber Looking East



Figure 33-6 Photograph of “L” Shaped Portion of Weir Chamber Looking East

33.1.2 Grit Chamber

The Grit Chamber is an above ground concrete structure approximately 68 ft long, 9 ft wide, and 4 ft tall. It is separated into two sections and has a single drain in the southeast corner of the chamber. It was fed by a series of above ground pipes that came from the Pump Vault. Figure 33-7 through Figure 33-9 show the Grit Chamber and Figure 33-10 shows the drain in the southeast corner of the chamber.



Figure 33-7 Photograph of Grit Chamber Looking West



Figure 33-8 Photograph of Grit Chamber Looking East



Figure 33-9 Photograph of East End of Grit Chamber Looking North



Figure 33-10 Photograph of Southeast Drain of Grit Chamber

33.1.3 Pump Vault

The Pump Vault is a brick-lined underground chamber in which several storm sewer pipes from Plant 5 and elsewhere on the site terminate. It is approximately 17 ft deep, 6 ft long, and 5 ft wide. The access to the vault is poured concrete extending approximately 3 ft above the surface. The presence of water in the bottom of this vault was a constant factor, despite plugging the northern 24-inch line coming from Plant 5. Figure 33-11 through Figure 33-13 are photographs of the Pump Vault.



Figure 33-11 Photograph of Pump Vault Looking East



Figure 33-12 Photograph of Pump Vault Looking South

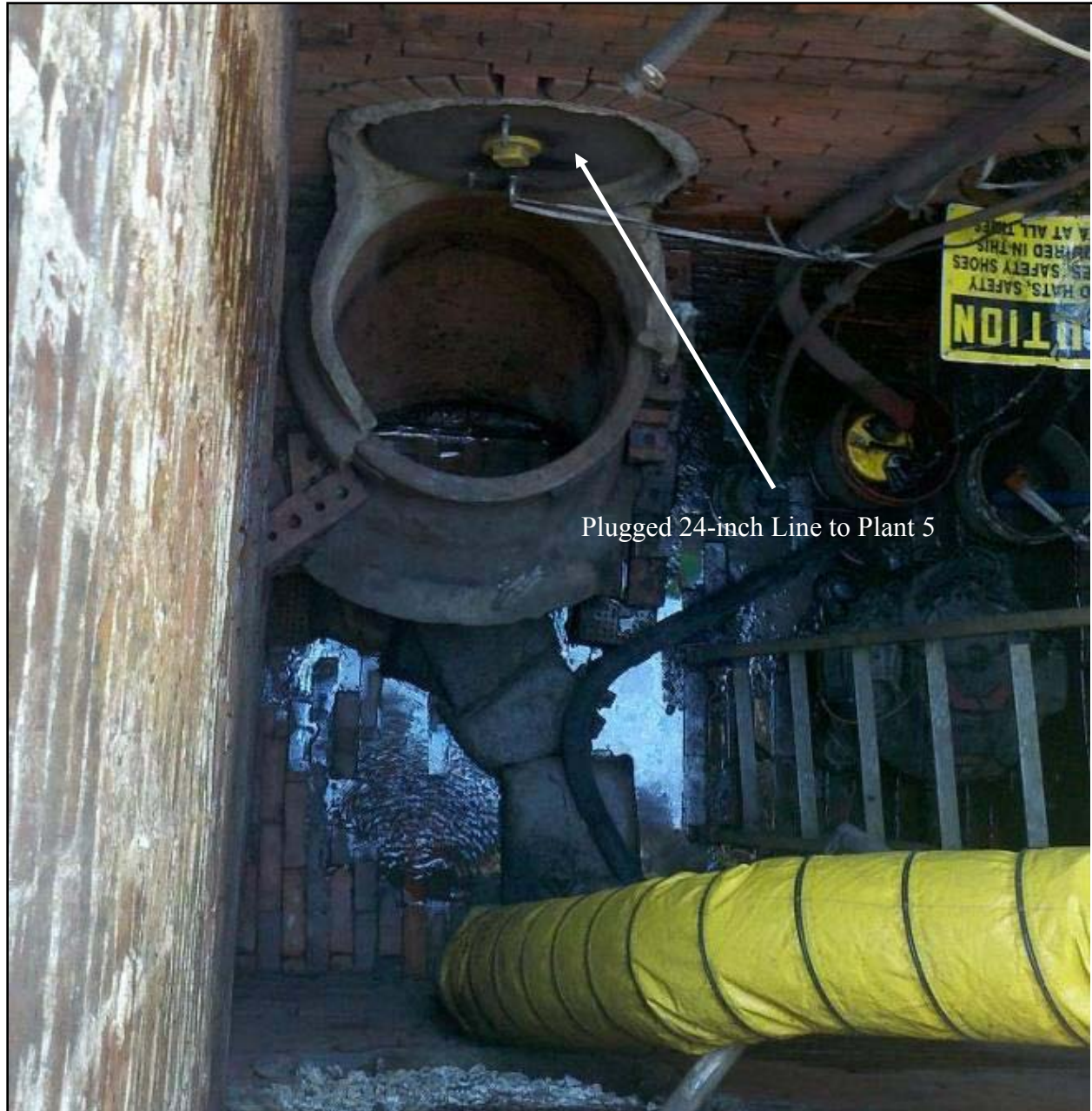


Figure 33-13 Photograph of Pump Vault Looking West

33.2 REMEDIAL ACTION AND RADIOLOGICAL SUMMARY

33.2.1 Weir Chamber

Rubber inserts were removed from the walls of the Weir Chamber to expose all wall surfaces and the walls and floor were power-washed. Radiological analysis via gamma spectroscopy indicated modest activity in the sediment (gross sum of fractions [SOF] = 0.25, Sample 3061). The walls and floor were then scanned with beta-gamma scintillation detectors and 16 static measurements were taken along with 100 cm² smears.

33.2.2 Grit Chamber

Debris was removed from the Grit Chamber and the entire structure was power washed. Radiological analysis via gamma spectroscopy indicated modest activity in the sediment (gross SOF < 0.29, Samples 3059 and 3060). The walls and floor were then scanned with beta-gamma scintillation detectors and 16 static measurements were taken along with 100 cm² smears.

33.2.3 Pump Vault

Two functional lift station pumps were taken off-line and removed from the Pump Vault prior to initial entry into the Pump Vault. Considerable material had accumulated at the bottom of the vault, so a vacuum truck was initially employed to remove water and debris. After the vacuum truck had removed all loose material, project personnel removed the remainder of the material with hand methods and/or wet/dry vacuums. Initial hand removal had to be stopped due to the presence of volatile chemical components in the remaining sediment. Upon analyzing the sediment and donning appropriate respiratory protection, the remainder of the material was successfully removed. Radiological analysis via gamma spectroscopy indicated modest activity in the sediment (gross SOF = 0.19, Sample 3155). The walls were then scanned with beta-gamma scintillation detectors and 16 static measurements were taken along with 100 cm² smears. The presence of water at the bottom of the vault prevented initial survey of the floor of the Pump Vault.

33.2.4 Summary

Survey results for the Weir Chamber, Grit Chamber, and Pump Vault are summarized in Table 33-1.

Table 33-1 Post-Remediation Residual Radioactivity Summary

Type of Residual Radioactivity	Range
Removable	< 10 dpm/100 cm ² alpha < 100 dpm/100 cm ² beta-gamma
Total	5 to 1,960 dpm/100 cm ² alpha 20 to 10,300 dpm/100 cm ² beta-gamma

Note: cpm to dpm conversion based on instrument response to average beta energy of 86.4 keV (Tc-99)

33.3 DATA COLLECTION

Data collection was performed based on the assumptions, methods, and performance criteria established to satisfy the DQOs in accordance with the C-T Phase II DP, Sections 14.4.1 and 14.4.3. Details regarding FSS design and quality assurance and quality control applicable to all survey units were discussed in Chapters 4 and 5, respectively, of this FSSR.

33.3.1 Beta Surface Activity Scans

A beta surface scan was performed over the Weir Chamber, Grit Chamber, and Pump Vault surfaces to locate radiation anomalies that might indicate areas with elevated residual radioactivity where further data collection (i.e., biased measurement) was warranted.

33.3.2 Surface Activity Measurements

Surface activity measurements to be used for the statistical test were collected at a frequency and at representative locations throughout SU4 such that a statistically sound conclusion regarding the radiological condition of the survey unit could be developed. Additional biased surface activity measurements were also collected at locations of elevated residual radioactivity identified by the beta surface activity scans. Figure 33-14 provides the beta surface activity scan results and surface activity measurement locations. A total of 26 (15 systematic and 11 beta surface activity scan biased locations) surface activity measurements were performed on the surfaces of SU4.

Table 33-2 provides the surface activity measurement results and summary statistics for the 15 systematic measurements. Table 33-3 provides the surface activity measurements results for the 11 beta surface activity scan biased measurement locations. In evaluating the data, background contributions were not accounted for in calculating surface activity. Alpha surface activity measurements were performed by AECOM, presumably for informational purposes, and are provided in the tables.

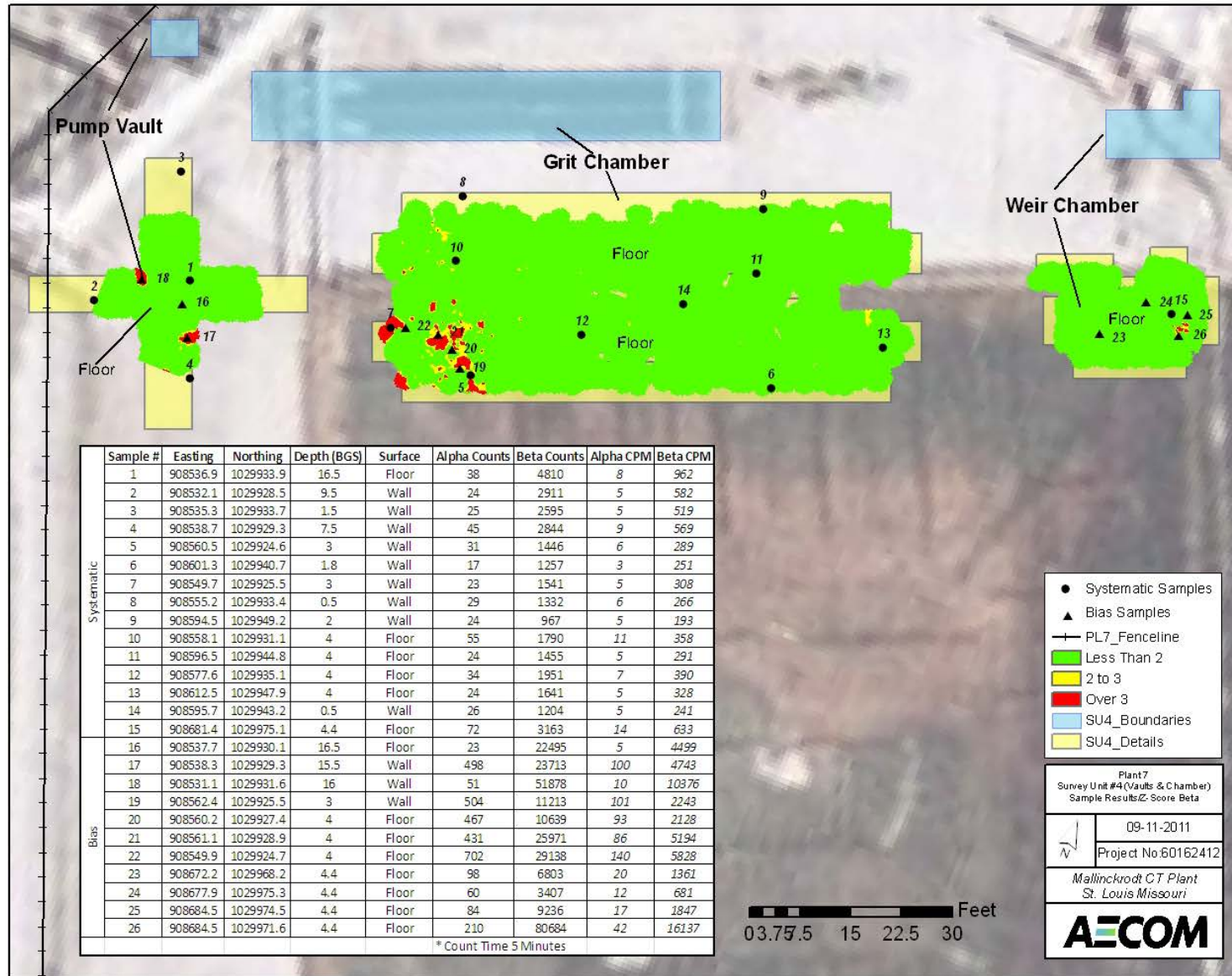


Figure 33-14 Beta Surface Activity Scans and Surface Activity Measurement Locations

Table 33-2 Systematic Surface Activity Results

Location	Depth (ft bgs)	Surface	Alpha Counts ^a	Beta Counts ^a	Alpha CPM	Beta CPM	Alpha (dpm/100 cm ²)	Beta (dpm/100 cm ²)
1	16.5	Floor	38	4,810	8	962	35	6,411
2	9.5	Wall	24	2,911	5	582	22	3,880
3	1.5	Wall	25	2,595	5	519	23	3,458
4	7.5	Wall	45	2,844	9	569	41	3,790
5	3	Wall	31	1,446	6	289	28	1,927
6	1.8	Wall	17	1,257	3	251	16	1,675
7	3	Wall	23	1,541	5	308	21	2,054
8	0.5	Wall	29	1,332	6	266	26	1,775
9	2	Wall	24	967	5	193	22	1,289
10	4	Floor	55	1,790	11	358	50	2,386
11	4	Floor	24	1,455	5	291	22	1,939
12	4	Floor	34	1,951	7	390	31	2,600
13	4	Floor	24	1,641	5	328	22	2,187
14	0.5	Wall	26	1,204	5	241	24	1,605
15	4.4	Floor	72	3,163	14	633	66	4,215
Average:							30	2,746
Median:							24	2,187
Standard Deviation:							13	1,369
Minimum:							16	1,289
Maximum:							66	6,411

^a Total count time of five minutes.

Table 33-3 Biased Surface Activity Results

Location	Depth (ft bgs)	Surface	Alpha Counts ^a	Beta Counts ^a	Alpha CPM	Beta CPM	Alpha (dpm/100 cm ²)	Beta (dpm/100 cm ²)
16	16.5	Floor	23	22,495	5	4,499	21	29,980
17	15.5	Wall	498	23,713	100	4,743	455	31,603
18	16	Wall	51	51,878	10	10,376	47	69,140
19	3	Wall	504	11,213	101	2,243	460	14,944
20	4	Floor	467	10,639	93	2,128	427	14,179
21	4	Floor	431	25,971	86	5,194	394	34,613
22	4	Floor	702	29,138	140	5,828	641	38,834
23	4.4	Floor	98	6,803	20	1,361	90	9,067
24	4.4	Floor	60	3,407	12	681	55	4,541
25	4.4	Floor	84	9,236	17	1,847	77	12,309
26	4.4	Floor	210	80,684	42	16,137	192	107,531

^a Total count time of five minutes.

33.4 DATA ANALYSIS

The data analysis was performed based on the assumptions, methods, and performance criteria established to satisfy the DQOs in accordance with the C-T Phase II DP, Sections 14.4.1 and 14.4.3. Details regarding FSS design and quality assurance and quality control applicable to all survey units were discussed in Chapters 4 and 5, respectively, of this FSSR.

33.4.1 Elevated Area Evaluation

There were no elevated areas identified in SU4.

33.4.2 Data Set Screening Analysis

Table 33-4 summarizes the results of the screening tests performed in accordance with Pages 14-27 through 14-29 of the C-T Phase II DP. All applicable tests demonstrating compliance passed.

Table 33-4 Screening Tests Results

Screening Test	Test Value	Conclusion
Min/Max	6,411	PASS
Low Level	N/A	Not applicable; Class 2 survey unit
DCGL _W	N/A	Not applicable; Min/Max < DCGL _W
EMC Limit	N/A	Not applicable; No elevated areas

33.4.2.1 Min/Max

Page 14-27 of the C-T Phase II DP describes calculating the Min/Max screening test value by subtracting the minimum reference area result from the maximum survey unit systematic result. Background was not accounted for in the surface activity measurements; therefore, the Min/Max screening test value was equal to the maximum survey unit result. Location 1 with a beta surface activity result of 6,411 dpm/100 cm² (from Table 33-2) was the maximum survey unit systematic result, which is the Min/Max screening test value. Because the test value was less than 180,000 β/min/100 cm², no further computations are required, i.e., DCGL_W screening and Wilcoxon Rank Sum (WRS) tests.

33.4.2.2 Low Level

In accordance with Page 14-27 of the C-T Phase II DP, the Low Level screening test is not applicable to Class 2 survey units.

33.4.2.3 DCGL_W

In accordance with Page 14-28 of the C-T Phase II DP and because the Min/Max test value was less than 180,000 β/min/100 cm², the DCGL_W screening test was not applicable to this survey unit.

33.4.2.4 EMC Limit

In accordance with Page 14-28 of the C-T Phase II DP, the elevated measurement comparison (EMC) Limit screening test was not applicable to this survey unit because no elevated areas were identified.

33.4.3 WRS Test

In accordance with Page 14-29 of the C-T Phase II DP and because the Min/Max test value was less than 180,000 $\beta/\text{min}/100 \text{ cm}^2$, the WRS Test was not required to demonstrate compliance.

33.4.4 Retrospective Analysis

A retrospective analysis was performed of the FSS results to determine whether the results met the survey design objectives, in accordance with Page 14-30 of the C-T Phase II DP. Table 33-5 provides the results of the retrospective analysis. Because the actual sample size exceeded the retrospective value sample size, the conclusion is that the survey design objectives were met.

Table 33-5 Retrospective Analysis

Parameter	<i>A Priori</i> Value	Retrospective Value Based on FSS Results ($\text{dpm}/100 \text{ cm}^2$)
Upper Bound of Gray Region	DCGL = 180,000 $\beta/\text{min}/100 \text{ cm}^2$	180,000
Lower Bound of Gray Region	0.5 x DCGL = 90,000 $\beta/\text{min}/100 \text{ cm}^2$	2,746
Spatial Variability (standard deviation)	1/6 x DCGL = 30,000 $\beta/\text{min}/100 \text{ cm}^2$	1,369
Type I Error (false positive)	0.05	0.05
Type II Error (false negative)	0.05	0.05
Relative Shift	3	129
Calculated N/2 Sample Size	15 ^a	9
Actual N/2 Sample Size	--	15

^aThe *a priori* value of 15 for the N/2 sample size was determined to be a conservative value that would allow application of either the Sign or WRS test. The *a priori* value for N/2 is 8 based on MARSSIM Table 5.3.

33.5 DEVIATIONS

In accordance with the second bullet in Section 14.5 of the C-T Phase II DP, the FSSR is required to list changes made in the FSS from what was proposed in the DP. Only one deviation was noted. Page 14-27 of the C-T Phase II DP indicated that the “data set for the survey unit will be processed within a database using screening software developed and verified for the project.” This database was not developed; instead, a combination of Microsoft[®] Excel[®] spreadsheets and hand calculations was utilized. This deviation is not significant and does not affect the data collection or assessment.

33.6 NRC INSPECTIONS

A summary of U.S. Nuclear Regulatory Commission (NRC) inspections applicable to the FSS are provided in Section 5.8 of this FSSR. None of the inspections applied to the Plant 7 final status surveys.

33.7 CONCLUSION

FSS data were verified to be reliable, appropriately documented, and technically defensible. Specifically, the following conclusions are made:

- The instruments used to collect the data were capable of detecting the radiation type (i.e., gamma) at or below the release criteria (described in Sections 4.4 and 4.5 of this FSSR).
- The calibration of the instruments used to collect the data was current and radioactive sources used for calibration were National Institute of Standards and Technology (NIST) traceable (described in Section 5.4 of this FSSR). Specific records available upon request.
- Instrument response was checked before instrument use each day, at minimum (described in Section 5.4 of this FSSR). Specific records available upon request.
- The survey methods used to collect the data were appropriate for the media and type of radiation being measured (described in Sections 4.4 and 4.5 of this FSSR).
- The survey data consist of qualified measurement results that are representative of the area of interest.

All the applicable screening tests passed, the retrospective analysis found that the survey design objectives were met. SU4 meets the industrial use scenario release criterion as established in the C-T Phase II DP Chapter 5; and therefore, satisfies the unrestricted release provisions of Title 10, Code of Federal Regulations (CFR), Part 20, Subpart E.

33.8 REFERENCES

Mallinckrodt, *Mallinckrodt Columbium-Tantalum Phase II Decommissioning Plan*, Revision 2, August 2008.