

Phase II Final Status Survey Report Mallinckrodt Columbium-Tantalum Plant

St. Louis, Missouri

Chapter 7

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	Prepared by: Energy Solutions, LLC Commercial Projects 1009 Commerce Park Drive, Suite 100 Oak Ridge, TN 37830	
Authored By:	Timothy 1 Bauer, Health Physicist	8/22/2013 Date
Reviewed By:	Michael A. Carr, CHP, Radiological Engineer/Radiation Safety Officer	8- 2?- 2318 Date
Reviewed By:	Mark Cambra, P.E., Project Manager	08-22-13 Date
Approved By:	Arthur J. Palmer, CHP, PMP, Director, Health Physics & Radiological Engineering	8/22/13 Date
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ABBREVIATIONS AND ACRONYMS

% percent

σ sigma; standard deviationAECOM Technical Services

bgs below grade surface
C-T columbium-tantalum

CFR Code of Federal Regulations

cm centimeters

DCGL derived concentration guideline level

DP decommissioning plan DQO data quality objectives

EMC elevated measurement comparison

EnergySolutions EnergySolutions, LLC

F exposure-weighted fraction of the DCGL_W

FSS Final Status Survey

FSSR Final Status Survey Report

ft feet

GWS gamma walk-over survey

HVAC heating, ventilation, and air conditioning

m meters

m² square meters

MARSSIM Multi-Agency Radiation and Site Investigation Manual (NUREG-1575)

mm milli-meters

MDC minimum detectable concentration

NE northeast

NIST National Institute of Standards and Technology

NRC U.S. Nuclear Regulatory Commission

NW northwest

ORISE Oak Ridge Institute for Science and Education

pCi/g picoCuries per gram

Ra radium

SOF sum of fractions

Th thorium U uranium

WRS Wilcoxon Rank Sum

RESULTS SUMMARY FOR PLANT 5 SUBSURFACE SU01 7.0

This chapter of the Final Status Survey Report (FSSR) presents the results of the final status survey (FSS) and data assessment for Plant 5 subsurface survey unit SU01 in accordance with Columbium-Tantalum (C-T) Phase II Decommissioning Plan (DP) Section 14.5. The FSS for this Class 1 survey unit was completed by AECOM Technical Services (AECOM) in May and June of 2011. The SU01 data assessment was performed based on the assumptions, methods, and performance criteria established to satisfy the data quality objectives (DQOs) in accordance with the C-T Phase II DP Section 14.4.3.8. The summary statistics provide numerical values for measures of central tendency (i.e., mean, median), variation (i.e., standard deviation), and spread (i.e., minimum, maximum). Data evaluation and statistical analyses were performed and a separate decision was made for each survey unit of the C-T Plant as to its suitability for release for unrestricted use based upon the industrial use scenario release criterion as established in C-T Phase II DP Chapter 5.

7.1 **OVERVIEW**

SU01 is a Class 1 survey unit located in the northwest portion of C-T Plant 5. The survey unit is approximately 353 square meters (m²) in size, which is less than the size limit of 3,000 m² for Class 1 survey units for subsurface material (per C-T Phase II DP, Table 14-4). Class 1 was the appropriate classification because the survey unit contained residual radioactivity that exceeded the derived concentration guideline value (DCGL_w) prior to remediation. Figure 7-1 shows the location of SU01 within the Plant 5 area.

Figure 7-2 and Figure 7-3 are photographs of SU01 that were taken during the FSS, following remediation. In Figure 7-2, as viewed from the northeast corner of the survey unit looking west, shows the east wall at the north end of Building 250 in the background. SU02 (partially backfilled) appears next to Building 250 in the upper left of the figure. Evident are the large remnants of legacy concrete that extend along the entire length of the north and south sides of the survey unit. Present near the center of Figure 7-2 is a concrete monolith poured during remediation to plug water flow from legacy sewer lines. Portions of brick foundations also remain on the west side of the survey unit adjacent to Building 250. Excavated depths ranged from 4 to 11 feet (ft).

There is a small area in the northwest corner of SU01 next to Building 250 (see Figure 7-2 upper right) beneath a mezzanine which supports heating, ventilation, and air conditioning (HVAC) equipment that was not excavated because of its proximity to Building 250 and the necessity to keep the HVAC systems operational.

Based on historical research, Mallinckrodt Inc. concluded that the legacy concrete was a foundation for the Sheet Mills from the National Enameling and Rolling Mills Company that occupied the site in the late 1800's.

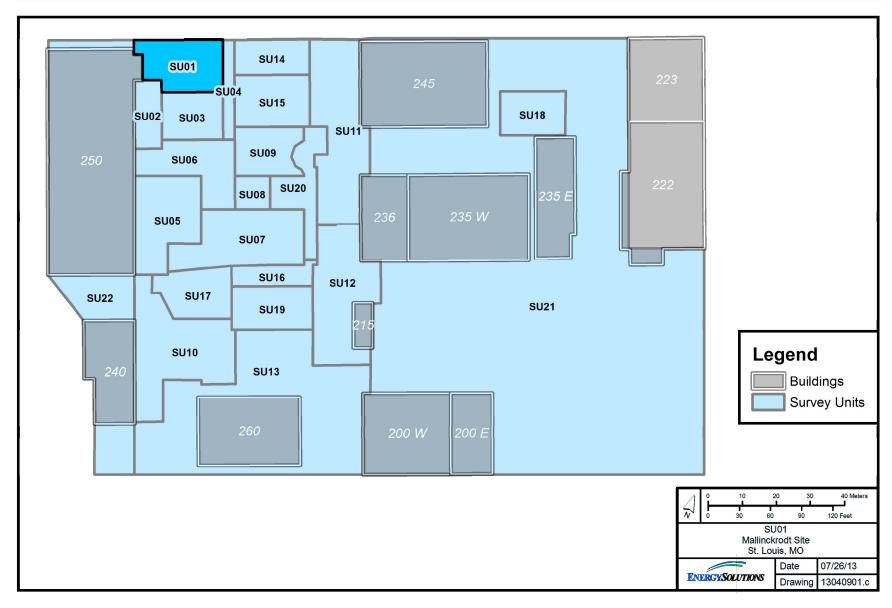


Figure 7-1 Location of Subsurface SU01 in C-T Plant 5



Figure 7-2 Photograph from NE Corner Looking West



Figure 7-3 Photograph from NW Corner Looking East

7.2 DATA COLLECTION

Data collection was performed based on the assumptions, methods, and performance criteria established to satisfy the DQOs in accordance with the C-T Phase II DP, Sections 14.4.1 and 14.4.3. Details regarding FSS design and quality assurance and quality control applicable to all survey units are discussed in Chapters 4 and 5, respectively, of this FSSR.

7.2.1 Gamma Scans

A gamma walk-over survey (GWS) was performed over 100% of the excavated area to locate radiation anomalies that might indicate areas with elevated residual radioactivity where further data collection (i.e., biased soil sampling) was warranted. Due to the relatively large amount of non-soil surface area in SU01, a GWS of non-soil surfaces was performed and evaluated separately so as not to bias the soil GWS results as well as to ensure no non-soil anomalies were left undetected.

7.2.2 Soil Sampling

Soil samples to be used for the statistical test were collected at a frequency and at representative locations throughout SU01 such that a statistically sound conclusion regarding the radiological condition of the survey unit could be developed. Where sample locations were positioned on concrete/brick surfaces, the sample was collected from the nearest suitable location. Additional biased soil samples were also collected at locations of elevated residual radioactivity identified by GWS. Figure 7-4 provides the GWS results and soil sampling locations. A total of 28 soil samples were collected throughout SU01, 19 over the areal footprint of SU01 (15 systematic and 4 GWS biased) and 9 along the excavated west wall along the Building 250 HVAC support.

All soil samples were analyzed on site via gamma spectroscopy analysis. Table 7-1 provides the sample results and summary statistics for the 15 systematic samples. Table 7-2 provides the sample results for the 9 west wall samples and 4 GWS biased samples.

Any remaining sieved material from each sample was analyzed separately to verify residual radioactivity was consistent with sample results. The radiological screening process did not identify any significant levels of radioactivity in the sieved materials removed from samples.

The unexcavated area in the northwest corner of SU01 was identified during remediation as containing elevated levels of residual radioactivity (i.e., sum of fractions [SOF] > 1.0). The nine biased soil samples (0831 through 0839) were taken from the exposed face of the area. The locations of the samples are shown in Figure 7-5.

The C-T Phase II DP, Table 4-17, provided mean background activity levels of 1.3, 2.5, and 4.4 picoCuries per gram (pCi/g) for thorium-232 (²³²Th), radium-226 (²²⁶Ra), and uranium-238 (²³⁸U), respectively. These values were used to calculate net SOF values—note that when measured activity concentration levels were less than the background mean resulting in a negative value, the net activity concentration was set equal to zero for the net SOF calculation.

To mitigate the risk of backfilling, the on-site laboratory analytical results were reviewed to determine the likelihood of the survey unit failing to meet the criteria for radiological release. The on-site laboratory, by design, reported conservative sample results.

AECOM did not send all biased samples to the off-site laboratory for analysis. Specifically, the biased samples collected from the unexcavated area in the northwest corner of SU01 were not sent. After Energy*Solutions*, LLC (Energy*Solutions*) arrived on site, the remaining samples were sent off site with the exception of sample 0834 because it could not be located. Based on the conservative reporting of the on-site laboratory, the results for sample 0834 were considered to be usable for FSS purposes.

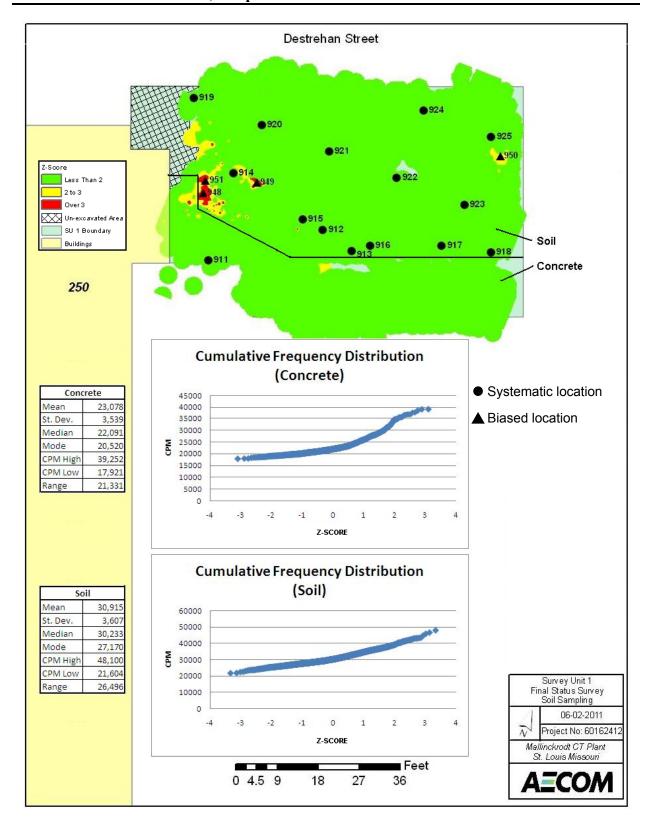


Figure 7-4 GWS and Soil Sampling Locations



Figure 7-5 Sampling Locations in Elevated Area

Table 7-1 Gamma Spectroscopy Systematic Sample Analytical Results

		On-Site Results										Off-Site Results ^a								On-Site/				
Comple	Donth			1	Activity Co		on (pCi/g)	b			SC	NE	Activity Concentration (pCi/g) b										F	Off-Site
Sample ID	Depth (ft bgs)		²³² Th			²²⁶ Ra			²³⁸ U		SC	<i>)</i>		²³² Th			²²⁶ Ra			²³⁸ U		30)T	Gross
ID	(it bgs)	Result	Uncert. (2σ)	MDC	Result	Uncert. (2σ)	MDC	Result	Uncert. (2σ)	MDC	Gross	Net ^c	Result	Uncert. (2σ)	MDC	Result	Uncert. (2σ)	MDC	Result	Uncert. (2σ)	MDC	Gross	Net ^c	SOF Ratio
0911	3.5	1.42	0.23	0.07	3.53	0.99	0.74	9.32	1.11	0.56	0.19	0.05	1.38	0.36	0.37	2.30	0.33	0.11	2.71	0.37	0.10	0.14	0.00	1.38
0912	10	0.54	0.11	0.06	2.24	0.63	0.47	5.25	0.73	0.39	0.11	0.00	1.09	0.40	0.33	2.31	0.33	0.10	2.46	0.34	0.10	0.13	0.00	0.83
0913	10	0.82	0.11	0.05	3.15	0.72	0.53	6.47	0.61	0.33	0.15	0.02	1.27	0.44	0.37	2.90	0.42	0.10	3.08	0.41	0.11	0.16	0.01	0.96
0914	10	0.96	0.15	0.05	1.39	0.45	0.33	1.94	0.52	0.31	0.09	0.00	1.01	0.23	0.19	1.30	0.18	0.05	1.40	0.19	0.05	0.09	0.00	1.02
0915	10	0.89	0.13	0.07	5.06	0.81	0.55	7.35	0.98	0.51	0.22	0.09	1.05	0.37	0.34	3.83	0.54	0.10	3.94	0.52	0.11	0.18	0.05	1.22
0916	10	1.00	0.12	0.08	3.82	0.82	0.59	6.46	0.80	0.44	0.18	0.05	1.74	0.38	0.33	3.93	0.53	0.09	4.53	0.58	0.10	0.21	0.07	0.85
0917	10	1.06	0.12	0.05	3.71	0.75	0.54	6.96	0.68	0.38	0.18	0.04	1.49	0.37	0.34	2.68	0.36	0.08	2.91	0.37	0.10	0.16	0.01	1.14
0918	10	0.00	902.22	0.18	3.63	0.90	0.67	7.61	0.89	0.47	0.13	0.04	1.69	0.39	0.36	2.89	0.40	0.10	3.11	0.42	0.11	0.17	0.03	0.77
0919	2.5	0.82	0.14	0.05	1.01	0.39	0.29	1.34	0.43	0.28	0.07	0.00	1.22	0.36	0.29	0.96	0.17	0.08	1.04	0.17	0.09	0.09	0.00	0.83
0920	10	1.15	0.18	0.06	1.76	0.51	0.37	1.67	0.44	0.31	0.11	0.00	1.31	0.31	0.20	1.38	0.20	0.06	1.70	0.23	0.06	0.10	0.00	1.06
0921	10	0.42	0.01	0.14	2.07	0.56	0.41	3.32	0.59	0.36	0.09	0.00	1.01	0.37	0.26	1.67	0.25	0.07	1.86	0.26	0.07	0.10	0.00	0.91
0922	10.5	0.79	0.13	0.06	2.44	0.64	0.47	5.44	0.58	0.32	0.12	0.00	1.39	0.37	0.25	1.21	0.19	0.08	1.39	0.21	0.08	0.10	0.00	1.22
0923	10	1.13	0.21	0.08	4.00	0.91	0.67	9.64	1.03	0.53	0.20	0.06	1.22	0.38	0.37	3.39	0.45	0.10	3.92	0.50	0.10	0.17	0.03	1.15
0924	10	0.72	0.13	0.06	2.71	0.62	0.45	5.59	0.68	0.37	0.13	0.01	1.04	0.30	0.29	1.88	0.28	0.08	2.13	0.30	0.09	0.11	0.00	1.18
0925	10.5	1.57	0.23	0.08	5.83	1.34	0.99	14.44	1.54	0.77	0.28	0.14	1.77	0.54	0.48	4.02	0.54	0.13	4.42	0.63	0.14	0.22	0.07	1.31
Summar	y Statistics																							
Count:		15			15			15			15	15	15			15			15			15	15	15
Averag	e:	0.89			3.09			6.19			0.15	0.03	1.31			2.44			2.71			0.14	0.02	1.06
Mediar	1:	0.89			3.15			6.46			0.13	0.02	1.27			2.31			2.71			0.14	0.00	1.06
Standa		0.39			1.33			3.44			0.06	0.04	0.26			1.04			1.13			0.04	0.02	0.19
Minim	ım:	0.00			1.01			1.34			0.07	0.00	1.01			0.96			1.04			0.09	0.00	0.77
Maxim	um:	1.57			5.83			14.44			0.28	0.14	1.77			4.02			4.53			0.22	0.07	1.38
Range:		1.57			4.82			13.10			0.21	0.14	0.76			3.06			3.49			0.13	0.07	0.60

a Off-site laboratory results as reported by TestAmerica after sufficient in-growth time to reach ²²⁶Ra progeny equilibrium. Italicized results indicate <MDC.
Calculated as discussed in Section 7.2.2.

Table 7-2 Gamma Spectroscopy Biased Sample Analytical Results

		On-Site Results								Off-Site Results ^a									On-Site/					
Ca1a	Donath			I	Activity Co	oncentratio	on (pCi/g)	b			so	ar c			1	Activity Co		n (pCi/g)	b			so	II. ¢	Off-Site
Sample ID	Depth (ft bgs)		²³² Th			²²⁶ Ra			²³⁸ U		50	'F		²³² Th			²²⁶ Ra			²³⁸ U		50	Г	Gross
ID	(it bgs)	Result	Uncert. (2σ)	MDC	Result	Uncert. (2σ)	MDC	Result	Uncert. (2σ)	MDC	Gross	Net d	Result	Uncert. (2σ)	MDC	Result	Uncert. (2σ)	MDC	Result	Uncert. (2σ)	MDC	Gross	Net d	SOF Ratio
West Wa	ll Samples	}																						
0831	5.5	1.77	0.40	0.16	14.89	2.08	1.38	15.32	1.90	1.06	0.60	0.46	1.72	0.47	0.59	11.70	1.42	0.40	14.20	4.88	5.61	0.49	0.34	1.23
0832	1	0.89	0.21	0.14	26.54	2.43	1.67	51.16	2.98	1.32	1.01	0.88	1.39	0.59	0.56	21.10	2.37	0.45	67.30	10.20	8.40	0.87	0.72	1.16
0833	3	0.92	0.15	0.05	1.84	0.56	0.41	3.42	0.57	0.33	0.11	0.00	1.14	0.29	0.14	1.57	0.27	0.14	3.35	1.31	1.76	0.11	0.00	1.00
0834	1	1.05	0.23	0.15	38.90	3.81	2.71	165.18	7.25	1.76	1.60	1.46						Sampl	e Lost					
0835	3	0.51	0.11	0.05	1.50	0.60	0.47	6.85	0.74	0.38	0.08	0.00	0.45	0.19	0.24	0.96	0.21	0.14	8.00	2.36	2.34	0.06	0.00	1.31
0836	1	0.38	0.09	0.03	1.45	0.43	0.32	3.10	0.50	0.28	0.07	0.00	0.32	0.17	0.24	1.02	0.21	0.15	5.67	1.86	2.01	0.06	0.00	1.24
0837	4	1.04	0.11	0.04	1.86	0.51	0.37	2.91	0.60	0.36	0.11	0.00	1.33	0.28	0.18	1.23	0.25	0.19	6.05	2.21	2.55	0.11	0.00	1.04
0838	1	1.25	0.16	0.09	9.60	1.76	1.34	38.79	2.04	0.76	0.43	0.29	1.86	0.43	0.37	8.89	1.07	0.28	53.40	7.23	5.10	0.45	0.31	0.95
0839	4	0.92	0.13	0.06	1.25	0.39	0.28	1.50	0.48	0.31	0.08	0.00	0.83	0.31	0.50	1.30	0.30	0.21	2.08	0.98	3.16	0.08	0.00	1.02
GWS Bia	sed Samp																							
0948	10	1.23	0.18	0.06	4.27	0.60	0.38	3.45	0.53	0.35	0.20	0.06	1.58	0.32	0.22	2.52	0.33	0.06	2.92	0.37	0.06	0.16	0.01	1.29
0949	4	0.87	0.15	0.07	2.87	0.63	0.44	2.63	0.68	0.41	0.14	0.01	0.98	0.31	0.30	2.50	0.36	0.08	2.72	0.37	0.09	0.13	0.00	1.06
0950	10	1.41	0.17	0.07	5.15	0.78	0.53	4.72	0.84	0.49	0.24	0.10	1.53	0.44	0.33	4.56	0.59	0.09	4.94	0.78	0.10	0.23	0.08	1.07
0951	4	0.34	0.01	0.11	2.16	0.39	0.25	1.49	0.42	0.25	0.09	0.00	0.43	0.23	0.24	1.18	0.18	0.07	1.30	0.19	0.07	0.06	0.00	1.50

a Off-site laboratory results as reported by TestAmerica after sufficient in-growth time to reach ²²⁶Ra progeny equilibrium.
b Italicized results indicate <MDC.
c Bolded orange SOF values indicate a result >0.5 but ≤1 and bolded red SOF values indicate a result >1.
d Calculated as discussed in Section 7.2.2.

7.2.3 Core Boring

The C-T Phase II DP, Table 4-7, provided characterization borehole results. Of the locations provided in the table, three were collected within the extent of SU01: BH-022, BH-023, and BH-045. Table 7-3 provides the data for these three locations. The results indicate that beyond the excavation extent, additional subsurface contamination is not reasonably expected. Therefore, in accordance with Page 14-22 of the C-T Phase II DP, FSS core sampling or measurements were not performed.

Lasation ID	Sample	Activity	Concentration	SOF				
Location ID	Depth (ft)	²³² Th	²²⁶ Ra	²³⁸ U	Gross	Net ^b		
	0.75 - 2	1.60	0.43	10.30	0.10	0.02		
	5 - 6	1.40	9.30	34.80	0.42	0.28		
BH-022	6 - 7	0.07	4.86	31.00	0.21	0.12		
	7 - 8	0.29	4.22	17.00	0.18	0.08		
	9 - 10	1.60	4.70	13.00	0.24	0.10		
DH 022	9 - 10	0.61	0.13	6.60	0.04	0.00		
BH-023	14 - 15	1.10	0.24	2.50	0.06	0.00		
	1.5 - 2	0.89	1.80	3.10	0.10	0.00		
BH-045	9 - 10	0.81	1.10	1.50	0.07	0.00		
	13 - 14	0.71	1.20	2.10	0.07	0.00		

Table 7-3 Characterization Borehole Results

7.3 DATA ANALYSIS

The data analysis was performed based on the assumptions, methods, and performance criteria established to satisfy the DQOs in accordance with the C-T Phase II DP, Sections 14.4.1 and 14.4.3. Details regarding FSS design and quality assurance and quality control applicable to all survey units are discussed in Chapters 4 and 5, respectively, of this FSSR.

7.3.1 Elevated Area Evaluation

Equation 9 from the C-T Phase II DP, Section 5.8.7 provides for the calculation of an *Index* value that represents the fraction or multiple of the $DCGL_{EMC}$. If the *Index* value is greater than one, then the $DCGL_{EMC}$ is exceeded. Parameters necessary to calculate the *Index* value for the unexcavated area in the northwest corner were:

- The levels of elevated area activity, represented conservatively by sample 0834, were 1.05, 38.90, and 165.18 pCi/g for ²³²Th, ²²⁶Ra, and ²³⁸U, respectively (refer to Table 7-2);
- Mean background activity levels were 1.3, 2.5, and 4.4 pCi/g for ²³²Th, ²²⁶Ra, and ²³⁸U, respectively (as provided in C-T Phase II DP Table 4-17);
- The size of the elevated area was determined to be approximately 21 m² (3 meters [m] by 7 m) with the elevated radioactivity extending to a depth less than 1 m; and,

^a Italicized results indicate <MDC.

^b Calculated as discussed in Section 7.2.2.

• The area factors from C-T Phase II DP Figure 5-3 for the elevated area were 1.8, 1.9, and 2.7 for ²³²Th, ²²⁶Ra, and ²³⁸U, respectively.

The calculation of the *Index* value is shown below. Because the ²³²Th elevated area activity concentration was less than the background mean, the thorium series term was set equal to zero. Because the *Index* value as calculated in accordance with the DP was less than one, this elevated area is compliant with the C-T Phase II DP for elevated measurements in soil.

$$Index = (0)_{Th \ series} + \frac{(38.90 - 2.5) \ pCi/g}{(1.9 \times 29.4 \ pCi/g)_{Ra226}} + \frac{(165.18 - 4.4) \ pCi/g}{(2.7 \times 721 \ pCi/g)_U} = 0.73$$

7.3.2 Data Set Screening Analysis

Table 7-4 summarizes the results of the screening tests performed in accordance with Pages 14-27 through 14-29 of the C-T Phase II DP. All applicable tests demonstrating compliance passed.

Screening Test	Test Value	Conclusion
Min/Max	0.20	PASS
Low Level	N/A	Not applicable; Class 1 survey unit
$\mathrm{DCGL}_{\mathrm{W}}$	N/A	Not applicable; Min/Max < 1
EMC Limit	0.18	PASS

Table 7-4 Screening Tests Results

7.3.2.1 Min/Max

In accordance with Page 14-27 of the C-T Phase II DP, the Min/Max screening test value was calculated by subtracting the minimum reference area result from the maximum survey unit systematic result. Sample 0925 with a gross SOF of 0.22 (from Table 7-1) was the maximum survey unit systematic result. Sample BH-Z-08 with a calculated gross SOF of 0.02 (from C-T Phase II DP Table B-1) was the minimum reference area result. The Min/Max screening test value was calculated to be 0.20. Because the test value was less than one, no further computations are required, i.e., DCGL_W screening and Wilcoxon Rank Sum (WRS) tests.

7.3.2.2 Low Level

In accordance with Page 14-27 of the C-T Phase II DP, the Low Level screening test is not applicable to Class 1 survey units.

7.3.2.3 DCGL_w

In accordance with Page 14-28 of the C-T Phase II DP and because the Min/Max test value was less than one, the $DCGL_W$ screening test was not applicable to this survey unit.

7.3.2.4 EMC Limit

In accordance with Page 14-28 of the C-T Phase II DP, the EMC limit screening test was applied due to the elevated area in the northwest corner (samples 0831 through 0839). Parameters necessary to calculate the exposure-weighted fraction of the DCGL_W, *F*, were:

- The size of the elevated area was determined to be approximately 21 m² (3 m by 7 m) with the elevated radioactivity extending to a depth less than 1 m,
- The area factor from C-T Phase II DP Figure 5-3 for the elevated area was conservatively set to 1.8 (based on thorium series only),
- The elevated area activity level was conservatively represented by sample 0834 with a gross SOF = 1.60, and
- The survey unit average was a gross SOF = 0.14 (refer to Table 7-1).

The calculation of the EMC screening test result is shown below, using C-T Phase II DP Equation 14-7.

$$F = \left[\frac{20 \ m^2}{353 \ m^2} \times \frac{1.60}{1.8 \times 1} \right] + \left[\frac{(353 - 20) \ m^2}{353 \ m^2} \times \frac{0.14}{1} \right] = 0.18$$

In accordance with the C-T Phase II DP and because the result was less than one, the total radioactivity concentration in the survey unit is within the release criterion.

7.3.3 WRS Test

In accordance with Page 14-29 of the C-T Phase II DP and because the Min/Max test value was less than one, the WRS Test was not required to demonstrate compliance.

7.3.4 Retrospective Analysis

A retrospective analysis was performed of the FSS results to determine whether the results met the survey design objectives, in accordance with Page 14-30 of the C-T Phase II DP. Table 7-5 provides the results of the retrospective analysis. Because the actual sample size exceeded the retrospective value sample size, the conclusion is that the survey design objectives were met.

Parameter	A Priori Value	Retrospective Value Based on FSS Results (Gross SOF)
Upper Bound of Gray Region	DCGL = 1	1
Lower Bound of Gray Region	0.5 x DCGL = 0.5	0.14
Spatial Variability (standard deviation)	$1/6 \times DCGL = 0.17$	0.043
Type I Error (false positive)	0.05	0.05
Type II Error (false negative)	0.05	0.05
Relative Shift	3	19.7
Calculated N/2 Sample Size	15 ^a	9
Actual N/2 Sample Size		15

Table 7-5 Retrospective Analysis

7.4 **DEVIATIONS**

In accordance with the second bullet in Section 14.5 of the C-T Phase II DP, the FSSR is required to list changes made in the FSS from what was proposed in the DP. Only one deviation was noted. Page 14-27 of the C-T Phase II DP indicated that the "data set for the survey unit will be processed within a database using screening software developed and verified for the project." This database was not developed; instead, a combination of Microsoft[®] Excel[®] spreadsheets and hand calculations was utilized. This deviation is not significant and does not affect the data collection or assessment.

7.5 ORISE CONFIRMATORY SURVEY

The Oak Ridge Institute for Science and Education (ORISE), by NRC request, performed an independent evaluation of AECOM's FSS methods and results. It made two site visits - on April 28, 2011, and on June 1-2, 2011. During these visits, ORISE observed FSS activities and conducted confirmatory surveys of SU01 and SU03 (ORISE 2011).

Two specific issues were identified by ORISE:

- 1. The contractor (AECOM) sieved the soil samples resulting in the removal of contaminated slag material (greater than the sieve size) and the excess material/slag (potentially containing significant residual radioactivity) was left behind in the surveyed area.
- 2. The contractor (AECOM) technicians were not relying on the audible output of the instrument to pinpoint judgmental locations real time. Instead their process was to post-process the data and go back and investigate the suspect locations.

In response to the issues identified by ORISE, AECOM performed an assessment of its GWS and soil sampling methodology. The assessment is found in Appendix H of AECOM's Preliminary FSSR (AECOM 2012), and is summarized below.

^a The *a priori* value of 15 for the N/2 sample size was determined to be a conservative value that would allow application of either the Sign or WRS test. The *a priori* value for N/2 is 10 based on MARSSIM Table 5.3.

Regarding ORISE's second issue, the GWS process was not modified to rely on the surveyor responding to an audible output of the instrument to pinpoint judgmental locations in real time. There are several reasons. A GWS data set collected uniformly over the surface of a survey unit, based on the rate at which data are logged (at 1-second intervals), is believed to more likely capture indications of an anomalous or subtle trending count rate than a surveyor responding to an audible indication. In addition, an evaluation of the GWS data set using both statistical and graphical methods (see Section 4.4.1.4 of this FSSR) is believed to provide superior information than a subjective response by a surveyor to an audible indication. For example, a cumulative frequency distribution provides information on the general shape of the data distribution, whether the population is normally or non-normally distributed, whether there are multiple populations present or individual outliers that may represent locations for further investigation. Another example is the z-score contouring process, which tends to smooth over single data points with slightly elevated values while accentuating clustered areas or single locations with significantly elevated values. This is the desired effect which aids in the data analysis by focusing attention on those areas most likely to have elevated residual radioactivity. These advantages are among those lost in a data set that is subjectively collected.

Table 7-6 lists the changes implemented by AECOM to improve the FSS survey and sampling methods.

Table 7-6 Changes to AECOM Survey and Sampling Methods

FSS Method	Change to Method	Rationale for Change
Perform single pass GWS	Perform double pass GWS with 2 nd pass performed perpendicular to 1 st pass.	A double pass increases data density and improves likelihood of detecting presence of small areas of elevated residual radioactivity.
Collect soil samples to a depth of 15 cm.	Collect soil samples to a depth of 30 cm.	External exposure is primary exposure pathway. The depth of contamination continues to affect external exposure up to 30 cm, beyond which impact is negligible.
Field screen soil samples using a ¹ / ₄ inch sieve; discard plus ¹ / ₄ inch material.	Collect all sample material. Do not field screen sample.	Radioactive material larger than 1/4 inch soil may exist.
Prepare soil samples by drying, screening using a No. 4 sieve, and mixing; discard plus No.4 (4.75 mm) material.	Prepare soil samples by drying, screening using No. 4 sieve, and mixing; retain plus No. 4 material.	Sample preparation preserves counting assumptions (uniformity, density, geometry) and comparability of onsite and offsite results.
Perform gamma spec count of prepared soil sample.	Perform gamma spec count of prepared soil sample; perform screening count of plus No. 4 material; investigate if above 0.5 x DCGL.	Radioactive material larger than No. 4 soil will be monitored with minimal impact on sampling process.

7.6 NRC INSPECTIONS

A summary of NRC inspections applicable to the FSS are provided in Section 5.8 of this FSSR. The scope of the inspections included, but was not limited to: review of project plans, interviewing of project personnel, evaluation of the on-site laboratory, observation of FSS field activities, and independent confirmatory surveys conducted by the NRC prior to and after backfilling. No violations were identified. No findings of significance were identified.

7.7 CONCLUSION

FSS data were verified to be reliable, appropriately documented, and technically defensible. Specifically, the following conclusions are made:

• The instruments used to collect the data were capable of detecting the radiation type (i.e., gamma) at or below the release criteria (described in Sections 4.4 and 4.5 of this FSSR).

- The calibration of the instruments used to collect the data was current and radioactive sources used for calibration were National Institute of Standards and Technology (NIST) traceable (described in Section 5.4 of this FSSR). Specific records are available upon request.
- Instrument response was checked before instrument use each day, at minimum (described in Section 5.4 of this FSSR). Specific records are available upon request.
- The survey methods used to collect the data were appropriate for the media and type of radiation being measured (described in Sections 4.4, 4.5, and 4.6 of this FSSR).
- The custody of samples collected for laboratory analysis was tracked from the point of collection until final results were obtained (described in Section 5.5.2 of this FSSR). Specific records are available upon request.
- The survey data consist of qualified measurement results that are representative of the area of interest.
- Areas identified with elevated residual radioactivity (i.e. SOF > 1.0) were appropriately investigated and the $DCGL_{EMC}$ properly applied.

All the applicable screening tests passed, the retrospective analysis found that the survey design objectives were met, and additional subsurface contamination was not reasonably suspected. SU01 meets the industrial use scenario release criterion as established in the C-T Phase II DP Chapter 5; and therefore, satisfies the unrestricted release provisions of Title 10, Code of Federal Regulations (CFR), Part 20, Subpart E.

7.8 REFERENCES

AECOM, Phase II Decommissioning Mallinckrodt C-T Plant, St. Louis, Missouri, Preliminary Final Status Survey Report: Plant 5 Survey Units 01, 02, and 03, March 2012.

Mallinckrodt, *Mallinckrodt Columbium-Tantalum Phase II Decommissioning Plan*, Revision 2, August 2008.

Oak Ridge Institute for Science and Education (ORISE), Final Report – Independent Confirmatory Survey Results for Survey Units 1 and 3 within the Plant 5 Footprint at the Mallinckrodt Inc. Site, St. Louis, Missouri, September 2011.