

JAN 13 1986

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Docket 72-3 (50-261)
Project M-39
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Project M-39
Docket 72-3

MEMORANDUM FOR: Leland C. Rouse, Chief
Advanced Fuel and Spent Fuel Licensing Branch
Division of Fuel Cycle and Material Safety

FROM: John P. Roberts
Advanced Fuel and Spent Fuel Licensing Branch
Division of Fuel Cycle and Material Safety

SUBJECT: MEETING WITH CAROLINA POWER AND LIGHT COMPANY (CP&L)
AND NUTECH, INC.

DATE AND TIME: January 8, 1986; 10:00 a.m.

LOCATION: 5th Floor Conference Room, Willste Building, Silver Spring, MD

ATTENDEES: See enclosure 1

PURPOSE: To discuss NRC review: (1) of the NUTECH Topical Report
(TR, docketed under Project No. M-39) and (2) of CP&L's
application for dry storage of spent fuel at its H.B.
Robinson 2 nuclear power station site (Docket No. 72-3)
using concrete storage modules designed by NUTECH and
described in its TR. (See enclosure 2, Agenda).

DISCUSSION:

The review of the NUTECH TR is nearing completion. Some additional information is to be supplied by NUTECH regarding its quality assurance program (QAP), the thermal stress in one area of the dry shielded canister (DSC), temperature versus time for fuel cladding, the DSC, and the inner face of the concrete module and a reference to ACI-349-80. Also information on the details of revised shielding calculations is to be submitted, and information making clear aspects of dose calculations and monitoring is to be supplied. With the exception of the QAP information, NUTECH plans to supply all information to NRC before the end of January 1986.

Given the progress made in the review of the NUTECH TR, NRC reviewers are now in a position to directly address CP&L's application, which involves modifications to and differences from the basic NUTECH design. To complete its October 30, 1985 responses to NRC comments, CP&L will be providing additional

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information. Also other related information arising out of discussion of CP&L's responses will be supplied. These submittals will be made during January 1986. These will included information on items c) through f) listed in enclosure 2, that is, the Site Plan; Cask, Collar and Lid Drawings; Specifications; and Chapter 8 Analysis. Also information related to the fuel to be stored, shielding calculations, material specification information, and structural analysis information regarding the DSC and the concrete foundation for the module will be submitted.

The information to be supplied by CP&L for review (where an analytical response is appropriate) will either reference existing analyses in the NUTECH TR, as applicable, or in general apply the same analytical methods. Handling of the modified General Electric IF-300 cask (which will contain the DSC) within the reactor confines will be governed under the existing H.B. Robinson 2 Part 50 Operating License and its procedures (see enclosure 3, pictures of cask handling operations with an IF-300 cask). Transit of the loaded cask from the reactor to the location of a concrete horizontal storage module (HSM) and cask docking with and loading of the DSC into the HSM will be reviewed by NRC staff under 10 CFR Part 72 (Information to be submitted, which is referred to in items c) through f) of enclosure 2, the Agenda, is applicable here).

Original Signed By

John P. Roberts
Advanced Fuel and Spent Fuel
Licensing Branch
Division of Fuel Cycle and
Material Safety, NMSS

Enclosures:

- 1) Attendance List
- 2) Agenda
- 3) IF-300 Cask Handling

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NAME: JRoberts	: kd:LCRouse	:	:	:	:	:
DATE: 01/10/86	: 01/10/86	:	:	:	:	:

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Enclosure 1

ATTENDEES

John P. Roberts
Jack Spraul
Fritz Sturz

NRC
NRC
NRC

Jan Kozyra
David Koss
John McLean

CP&L
CP&L
CP&L

John V. Massey
Jerry Rosa

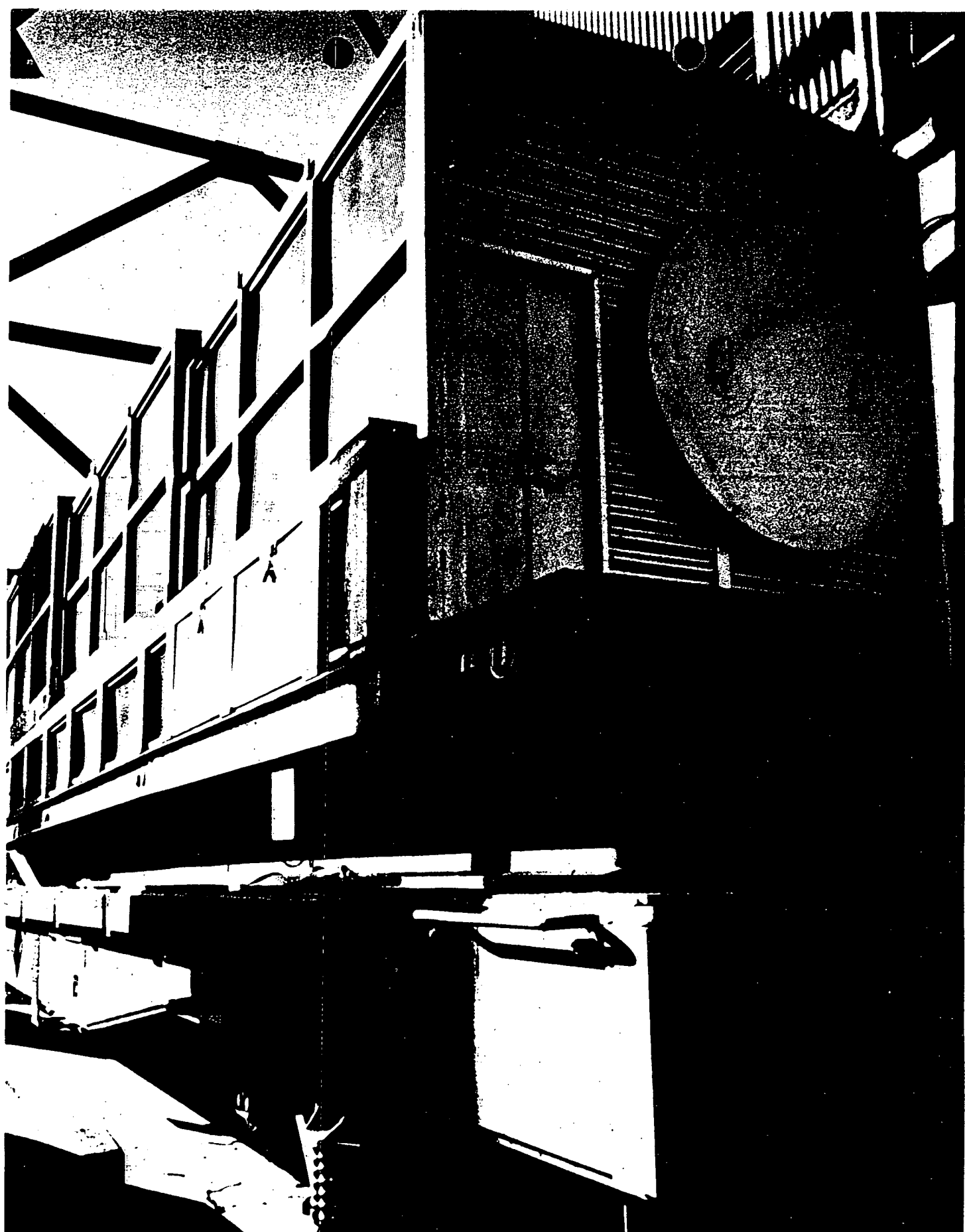
NUTECH
NUTECH

T.E. Albert
Richard Belanger
Jim Hammelman
Neil E. Johnson
Bill MacNabb
John R. Stokley

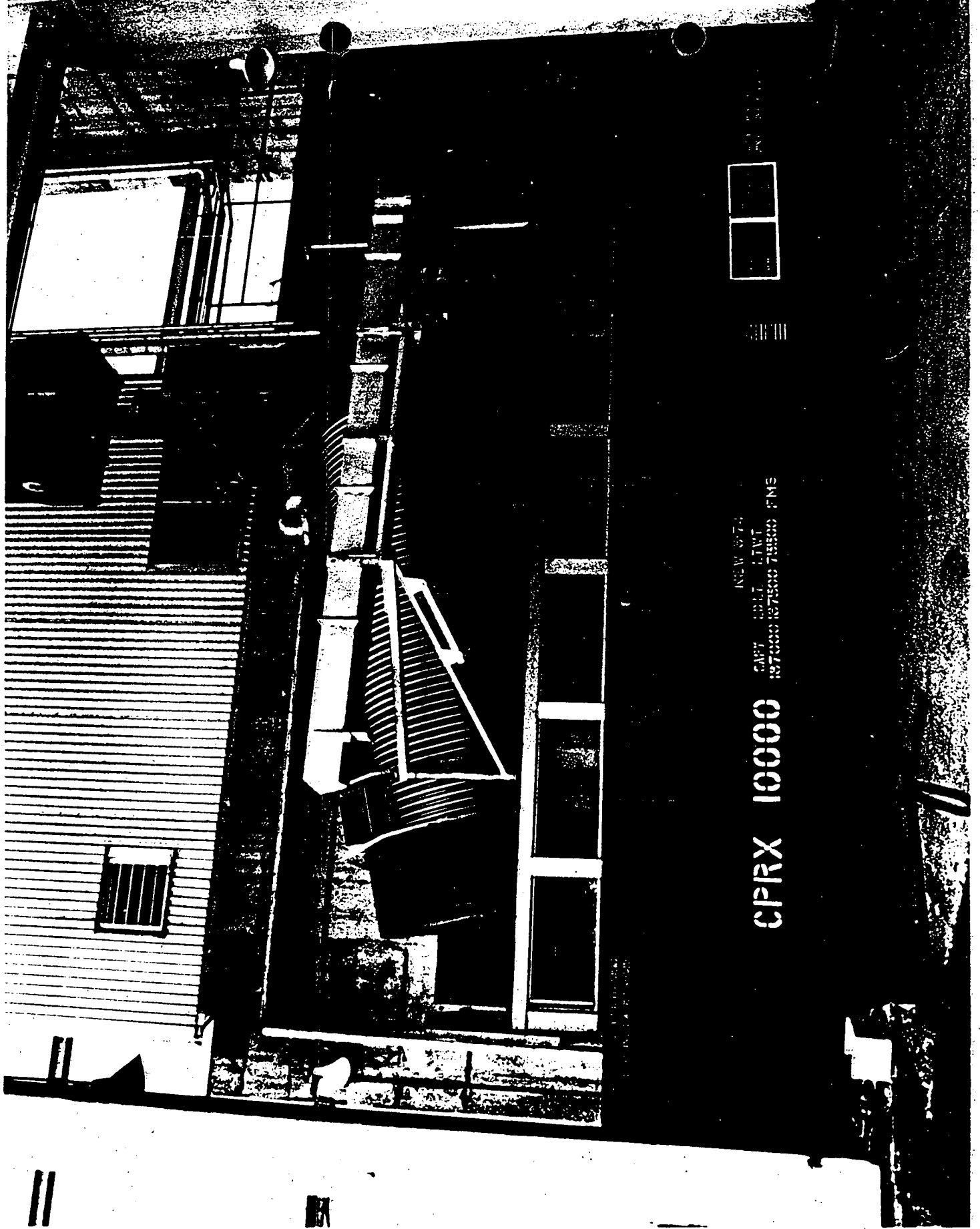
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CAROLINA POWER & LIGHT COMPANY
DRY STORAGE DEMONSTRATION PROGRAM
MEETING WITH NRC JANUARY 8, 1986

- A. STATUS OF NUTECH TOPICAL REPORT NRC
- B. STATUS OF SAI REVIEW OF INSTRUMENTATION NRC/SAI
- C. DISCUSS ADDITIONAL CP&L INFORMATION CP&L/NUTECH
- a) Cask Handling System
 - b) Shielding Analysis
 - c) Site Plan
 - d) Cask, Collar, Lid Drawings
 - e) Specifications
 - f) Chapter 8 Analysis
- D. LICENSING SCHEDULE All



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