



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
101 MARIETTA ST., N.W., SUITE 3100  
ATLANTA, GEORGIA 30303

Report No. 50-261/80-13

Licensee: Carolina Power and Light Company  
411 Fayetteville Street  
Raleigh, NC 27602

Facility Name: H. B. Robinson

Docket No. 50-261

License No. DPR-23

Inspection at Robinson site near Hartsville, South Carolina

Inspector: A. K. Hardin

7/11/80  
Date Signed

Approved by: P. J. Kellogg  
P. J. Kellogg, Section Chief, RONS Branch

7/14/80  
Date Signed

SUMMARY

Inspection on June 16-20, 1980

Areas Inspected

This routine, unannounced inspection involved 31 inspector-hours on site in the areas of Operations, and Bulletins.

Results

Of the two areas inspected, no items of noncompliance or deviations were identified.

## DETAILS

### 1. Persons Contacted

#### Licensee Employees

- \*R. B. Starkey, Jr., General Manager
- \*H. S. Zimmerman, Manager, Technical and Administration
- \*C. Wayne Crawford, Manager, Operations and Maintenance
- J. M. Curley, Engineering Supervisor
- \*F. Lowrey, Operations Supervisor
- C. A. Bethea, Training Supervisor

\*Attended exit interview

### 2. Exit Interview

The inspection scope and findings were summarized on June 20, 1980 with those persons indicated in Paragraph 1 above. The inspection activities presented in the following details were discussed.

### 3. Licensee Action on Previous Inspection Findings

Not inspected

### 4. Unresolved Items

Unresolved items were not identified during this inspection.

### 5. Review of Plant Operations

The inspector reviewed the following areas of plant operation to ascertain whether facility operations were in conformance with applicable regulatory requirements and licensee commitments:

#### a. Operating Logs and Records

The inspector reviewed the following operating logs and records to ascertain whether plant operations were in conformance with technical specifications and license requirements:

- (1) Shift Foremen logbook
- (2) Control Room Operator logbook
- (3) Auxiliary Operator - Outside Data Log
- (4) Auxiliary Operator - Inside Data log
- (5) Equipment Inoperable Reports
- (6) Several Periodic tests

The inspector reviewed operating logs from June 10-17, 1980 to ensure that logged data was recorded in sufficient detail to communicate equipment status and was properly reviewed by supervisory personnel.

In addition, logs were reviewed to insure that logged information did not conflict with technical specification requirements.

Periodic test results which were submitted as complete on June 16 and June 17, 1980 were reviewed for completeness and compliance with acceptance criteria. Operating Data for several periods during the week of June 16, 1980 were reviewed to ascertain status of the plant and condition of the equipment.

No items of deviation or noncompliance were identified.

b. Facility Tour

The inspector toured portions of the Unit 2 reactor auxiliary building, turbine structure and control bays. Inspection activities in this area included observation to ensure that: instrumentation was operating and calibrated as required, radiation controls were established and properly adhered to, fire protection equipment was located as specified and control switches were verified to be properly aligned as required by technical specifications and plant procedures.

No item of deviation or noncompliance were identified.

c. Control Room Operations

Control Room operations were observed and discussed with control room personnel to assure compliance with technical specifications and plant administrative requirements. Inspection activities in the control room included: observation of selected instrumentation to assure plant operation was within required limits and verification that control room manning was in conformance with the requirements of 10 CFR 50.54(k) and (m).

No items of deviation or noncompliance were identified.

6. IE Bulletins (IEB)

a. IEB 79-17 Rev. 1 "Pipe Cracks in Stagnant Borated Water Systems at PWR plants."

Based on review of the licensee's response to the subject bulletin, dated December 6, 1979 and onsite inspection of the licensee's work on the bulletin conducted during the period October 15-18, 1979 and reported in IE Report Number 50-261/79-23, IE bulletin 79-17 Rev 1 is closed.

b. IEB 79-21 "Temperature Effects on Level Measurements"

The inspector reviewed the licensee's response to Bulletin 79-21 dated September 14, 1979. Onsite inspection of the bulletin was conducted during the period June 16-20, 1980. In item 2 of the bulletin the

licensee stated they planned further evaluation of the phenomena of reference leg boiling as it might apply to the Robinson Plant, although containment analyses by Westinghouse for a typical dry containment plant indicates that reference leg boiling would not occur.

The licensee had not completed the evaluation at the close of the inspection. This item will remain open as 261-80-13-01. The licensee had concluded that in cases of reference leg heating that the Technical Specification value for the low-low trip for the steam generators was nonconservative. The licensee stated they had submitted a proposed Technical Specification change to correct the nonconservation, i.e. change the low low trip from "more than 5 percent" to 14 percent. As of the inspection date, the Technical Specification had not been approved.

The licensee stated they have provided for the nonconservatism by procedurally and administratively requiring the trip to be set at 15 percent. The inspector, at the exit interview urged the licensee to expedite correction of the Technical Specification, but did not leave the item open or unresolved.

- c. IEB 80-04 "Analysis of PWR Main Steam Line Break With Continued Feedwater Addition"

The inspector reviewed the licensee's response to IEB 80-04. The inspector verified that the Nuclear Steam System Supplier had provided input to the licensee and that the licensee had responded to each of the questions raised in the bulletin. At the exit interview the inspector stated that a preliminary screening of the bulletin had been done and the response appeared adequate, however the bulletin response would be transmitted to Nuclear Reactor Regulation (NRR) for technical evaluation. IEB 80-04 is closed.