UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

Report No. 50-261/79-21

Licensee: Carolina Power and Light Company
411 Fayetteville Street
Raleigh, North Carolina 27602

Facility Name: Robinson

Docket No. 50-261

License No. DPR-23

Inspection at Robinson Site, Hartsville, South Carolina

Inspected by: _____ 10/16/79
T. J. McHenry Date Signed

Approved by: _____ 10/16/79
P. J. Kellogg, Section Chief, RONS Branch Date Signed

SUMMARY

Inspected on September 18-21, 1979.

Areas Inspected

This routine, unannounced inspection involved 30 inspector-hours onsite in the areas of plant operations, periodic and special reports, licensee event reports, IE Bulletins and Circulars, previous enforcement matters, and outstanding items.

Results

Within the six areas inspected, no apparent items of noncompliance or deviations were identified.

DETAILS

1. Persons Contacted

Licensee Employees

R. B. Starkey, Jr., Plant Manager
*B. W. Garrison, Quality Assurance Supervisor
D. S. Crocker, Environmental and Radiation Control Supervisor
C. Wright, Engineering Technician
*M. Page, Engineer
*R. H. Chambers, Senior Engineer
R. S. McGirt, Senior Nuclear General Specialist
*W. T. Traylor, Administrative Supervisor
F. Bishop, Electrical Engineer
F. Lowery, Training Coordinator

Other licensee employees contacted included operators, security force members, and office personnel.

*Attended exit interview.

2. Exit Interview

The inspection scope and findings were summarized on September 21, 1979 with those persons indicated in Paragraph 1 above. The inspection activities presented in these details were discussed.

3. Licensee Action on Previous Inspection Findings

a. (Closed) Infraction (78-17-01): Failure to distribute and use revisions to plant procedures. The inspector reviewed selected copies of procedures in the control room working copy file and verified that only current revisions existed in the file. A similar audit of the control room working file was also performed during inspection 50-261/ 79-03. Based upon these audits it appears that the licensee's corrective actions in regard to this item are satisfactory.

b. (Closed) Infraction (79-11-01): Procedure changes without Plant Nuclear Safety Committee review and plant manager approval. The inspector verified that OP-7A valve checklist had been revised to require diesel day tank drain valves to be locked. The inspector also verified that OP-14 had been revisied to assure proper alignment of the motor-driven auxiliary feedwater pump discharge valves. This item was also reviewed during inspection 50-261/79-17. The inspector determined that appropriate locking devices and procedual controls had been established for the turbine-driven auxiliary feedwater pump discharge valve.

c. (Closed) Deficiency (79-11-03): Failure of maintenance procedures to restore systems to an operable alignment. The inspector reviewed selected Operating Work Procedures (OWP's) and verified that they had been reviewed and revised as required to assure that component restoration to an operable alignment following maintenance.

d. (Closed) Unresolved Item (79-11-08): Missing valve identification tags. This item was identified during the 1979 refueling outage. Subsequent inspection of valve tagging indicated that equipment was tagged and properly identified consistent with plant requirements. The missing tags noted during inspection 50-261/79-11 had apparently resulted from losses caused by the extensive refueling maintenance and design modification activities. Since tags had been replaced, it appears that the licensee's existing tagging program meets the requirements of 10 CFR 50, Appendix B, Criterion XIV.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Review of Plant Operations

The inspector reviewed the following areas of plant operation to ascertain whether facility operation was in conformance with applicable regulatory requirements and licensee commitments:

a. Operating Logs and Records

The inspector reviewed the following operating logs and records to ascertain whether plant operation was in conformance with Technical Specifications and administrative licensee requirements:

-Shift Foremen logbook July 25 - September 9, 1979
-Control Room Operator logbook August 25-September 20, 197
-Control Room Hot Operation Data log August 1-31, 1979
-Auxiliary Operator-Outside Data log August 1-31, 1979
-Auxiliary Operator-Inside Data log August 1-31, 1979
-Equipment Inoperable Report Forms July 1-September 20, 1979

The inspector reviewed operating logs to insure that logged data was recorded in sufficient detail to communicate equipment status and properly reviewed by supervisory personnel. In addition, logs were reviewed to insure that logged information did not conflict with Technical Specification requirements. The inspector reviewed plant Trouble Tickets and Equipment Inoperable Report Forms to insure that applicable Technical Specifications including reporting requiremens were adhered to for inoperable equipment.

No problems were identified in this area.

b. Facility Tours

The inspector conducted several facility tours during normal and backshifts. Inspection activities in this area included observation to insure that; instrumentation was operating and calibrated as required, radiation controls were established and properly adhered to, fire protection equipment was located as specified and appeared operable. In addition, selected valves, breakers and control switches were verified to be properly aligned as required by Technical Specifications and plant procedures.

No problems were identified within this area.

c. Control Room Operations

Control room operations were observed and discussed with control room personnel to assure compliance with Technical Specifications and plant administrative requirements. Inspection activities in the control room included observation of selected instrumentation to assure plant operation within required limits, observation of surveillance testing and shift turnover, verification of control room manning in conformance with the requirements of 10 CFR 50.54(k), IE Bulletin 79-06C and discussions with control room operators pertaining to selected parameters and annunciators.

No problems in this area were identified.

6. Review of Licensee Event Report (LER's)

The inspector reviewed five LER's for consistency with the requirements of Technical Specification Section 6.9. The inspector examined the licensee's analysis of the event, corrective action taken, and discussed the LER's with licensee representatives. The following LER's were reviewed:

a. LER 79-24 and 79-25, Failure to Perform Fire Patrols

The current implementation and controls to assure completion of required fire patrols was reviewed. The fire patrols are being conducted in accordance prescribed check lists and hourly reports to be shift foreman made to assure completion of the required inspections. This LER is closed.

b. LER 79-26, Snubber Failure During Functional Test

This event was discussed by telephone with the licensee staff on September 20, 1979, in order to ascertain the status of investigation regarding generic concerns identified. Licensee representative stated that a supplemental response would be submitted as soon as the affected snubbers could be rebuilt and tested. During the onsite inspection it was determined that the snubber failure had been caused by the test method. Licensee representatives stated that the details of the

failure mechanism involved in testing would be addressed in the supplemental response. This LER will remain open pending a review and inspection to followup on the supplemental response.

c. LER 79-28, Nitrogen to Accumulators Fill Valve Stuck

The nitrogen to accumalators valve was still in an inoperable status. The inspector verified that required manual valves were locked closed as required by the Technical Specifications. Several plant operators were questioned by the inspector to insure that they were aware of the valve inoperability and associated requirements. The inoperable valve was scheduled for repair when plant conditions permit. This LER is closed.

d. LER 79-30, Service Water Booster Pumps Inoperable

Review of plant logs indicated that a calibration check was performed on the low suction pressure trip switches on both pumps subsequent to loss of both pumps. A review of as-found calibration data indicated that no calibration of the pressure switches was required. In addition, evaluation of possible design or procedure change were still under consideration. This LER is closed.

7. Review of Monthly Operating Reports

The inspector reviewed monthly operating reports from September, 1978 through August, 1979 for consistency with the requirements of Technical Specification 6.9.1.c. The reports were discussed with a licensee representative. The inspector reviewed selected logs and records to verify the accuracy of information provided in the reports and selected computations were checked to verify their accuracy. No problems were identified.

In addition, a review of unplanned reactor trips was performed to determine the cause and to evaluate the licenee's corrective actions to prevent recurrence. During the one year period reviewed, the plant had experienced eight unplanned reactor trips. The inspector catagorized the cause of the eight reactor trip as follows:

a. Four trips due to equipment malfunction.
b. Two trips due to design problems.
c. Two trips due to personnel error.

A review of Unit Outage Reports for the above events was conducted to ascertain corrective actions taken. In addition, the inspector discussed corrective actions described in the Unit Outage Reports to verify that corrective actions had been completed. Based upon the review of reactor trips, Outage Reports and discussions with plant personnel it appear that plant management is responsive in taking corrective action to prevent recurrence of unplanned reactor trips.

8. Review of Special Report

A Special Report dated August 27, 1979 required by Technical Specification 3.14, Fire Protection System, was reviewed and discussed with a licensee representative to ascertain the current status of the fire protection system. The inoperable systems described in paragraphs 1, 2 and 3 of the special report had been returned to operable status as required by the Technical Specifications. However, two items in the special report remained incomplete. In order to insure that all items required would be completed, the following items were identified as open (50-261/79-21-01).

a. Installation of heat detectors in the drumming room in lieu of smoke detectors.

b. Evaluation and upgrading of Halon system to provide an accepable concentration following actuation.

9. Review of IE Bulletins (IEB's)

a. IEB 77-08, Assurance Locking System Safety and Safeguards during an emergency.

The licensee's response to this IEB was reviewed during inspection 50-261/78-17. However, this item remained open pending installation and testing of additional locking systems during the 1979 refueling outage. The inspector determined that actions required by IEB 77-08, items 1.a.(1), 1.a.(2), 1.b, 2, 3, and 4 had been completed. However 1.a(3) - provision for periodic testing of locking system capability to switch to auxiliary power was still under evaluation. The ability to transfer locking systems to auxiliary power had been demonstrated during pre-operational testing. Further the licensee had originally intended to test this feature monthly; however, system design was determined to make monthly testing impractical. In order to simplify closure this item, IEB 77-08 was closed and the establishement of a periodic testing program for the auxiliary power to locking systems was identified as an open item (50-261/79-21-02).

b. IEB 79-05, 79-05A, 79-05B, 79-08 and 79-12

These IEB's were issued to the licensee for information only. The inspector verified that each bulletin had been received and reviewed. Since no actions were required by the licensee, these IEB's were closed.

10. Review of Outstanding Open Items

a. Item 79-08-01, Reactor Vessel 90°F Limitation

The inspector reviewed FT-9.4 (Reactor Vessel Stud Removal and Replacement) and verified that a procedure change had been implemented requiring direct verification of reactor vessel head and closure stud

temperatures prior to full tensioning. In addition, a revision to GP-6 (Plant cooldown) was under consideration to caution operators to not cooldown below 90°F with closure studs fully tensioned. This item is closed.

b. Item 79-11-06, Electrical Distribution Manaul

A review was performed to ascertain the use of the Electrical Distribution Manual for plant activities and to ascertain the type of document controls established. It was determined that the Electrical Distribution Manual was routinely utilized by operating personnel to identify power supplies to plant loads. Further, the inspector determined that no formal administrative controls existed to insure periodic review or revision. In addition, control of the number and holders of copies did not exist. This item was discussed with the Plant Manager and at the exit interview. The licensee committed to perform a timely review of the use and distribution of the Electrical Distribution Manual and to implement appropriate document control measures. This item remains open pending a review of licensee actions with regard to this item.

c. Petroleum Jelly In Certain Relays

The item was reviewed due to NRC concern of a potential problem associated with the use of petroleum jelly to lubricate certain General Electric induction relays. The inspector discussed this item with the plant engineer responsible for electrical equipment. It was determined that no relays of the type described were utilized. Further, petroleum jelly was not used as a lubricant for any plant electrical equipment. This item is closed.

11. Observation of Plant Fire Drill

The inspector observed an unannounced fire drill that was conducted on September 20, 1979. The drill consisted of a actual fire in a 55 gallon drum in an open area inside the Unit 2 protected area. The inspector observed the plant response and actions with regard to the fire and verified that actions were in accordance with the requirements of the Plant Fire Protection Manual. Two hose stations were utilized and the fire was extinguished. No problems were identified.