

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of)
CAROLINA POWER AND LIGHT COMPANY) Docket No. 50-261
(H. B. Robinson Steam Electric)
Plant Unit No. 2)

ORDER MODIFYING LICENSE CONFIRMING ADDITIONAL LICENSEE COMMITMENTS
ON EMERGENCY RESPONSE CAPABILITY

I.

Carolina Power Light Company (CP&L), (the licensee) is the holder of Facility Operating License No. DPR-23 which authorizes the operation of the H. B. Robinson Steam Electric Plant Unit No. 2 (the facility) at steady-state power levels not in excess of 2300 megawatts thermal. The facility is a pressurized water reactor (PWR) located in Darlington County, South Carolina.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to provide substantial additional protection in the operation of nuclear facilities and significant upgrading of emergency response capability based on the experience from the accident at TMI-2 and the official studies and investigations of the accident. The requirements are set forth in NUREG-0737, "Clarification of TMI

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Action Plan Requirements," and in Supplement 1 to NUREG-0737, "Requirements for Emergency Response Capability." Among these requirements are a number of items consisting of emergency response facility operability, emergency procedure implementation, addition of instrumentation, possible control room design modification, and specific information to be submitted.

On December 17, 1982, a letter (Generic Letter 82-33) was sent to all licensees of operating reactors, applicants for operating licenses, and holders of construction permits enclosing Supplement 1 to NUREG-0737. In this letter operating reactor licensees and holders of construction permits were requested to furnish the following information, pursuant to 10 CFR 50.54(f), no later than April 15, 1983:

- (1) A proposed schedule for completing each of the basic requirements for the items identified in Supplement 1 to NUREG-0737, and
- (2) A description of plans for phased implementation and integration of emergency response activities including training.

III.

CP&L responded to Generic Letter 82-33 by letter dated April 15, 1983. By letter dated August 24, 1983, CP&L modified several dates as a result of negotiations with the NRC staff. CP&L made commitments to supply schedules and plans (items (1) and (2) of Section II above) by December 1984. On February 21, 1984, the NRC issued "Order Confirming Licensee's Commitments on Emergency Response Capability" which confirmed CP&L's Commitments. By letter dated December 31, 1984, CP&L made commitments to complete the basic requirements. CP&L commitments include (1) dates for providing required submittals to the NRC, (2) dates for implementing certain requirements, and (3) a schedule for providing implementation dates for other requirements.

IV.

The February 21, 1984, Order stated that for those requirements for which CP&L committed to a schedule for providing implementation dates, those dates would be reviewed, negotiated and confirmed by a subsequent order. In conformance with the milestones in the February 21, 1984 Order, as implemented by CP&L letter dated December 31, 1984, CP&L provided completion schedules for the following requirements:

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| 1. Safety Parameter Display System (SPDS) | 1b. SPDS fully operational and operators trained. |
| 2. Detailed Control Room Design Review (DCRDR) | 2b. Submit a summary report to the NRC including a proposed schedule for implementation. |
| 3. Regulatory Guide 1.97 Application to Emergency Response Facility | 3b. Implement (installation or upgrade) requirements. |
| 5. Emergency Response Facilities | 5a. Technical Support Center fully functional. |
| | 5b. Operational Support Center fully functional. |
| | 5c. Emergency Operations Facility fully functional. |

The attached Table summarizing CP&L's schedular commitments for the above items was developed by the NRC staff from the information provided by CP&L.

The staff reviewed CP&L's August 24, 1984 and December 31, 1984 letters and discussed the dates with the licensee.

The NRC staff finds that these dates are reasonable and achievable dates for meeting the Commission requirements. The NRC staff concludes that the schedule proposed by the licensee will provide timely upgrading of the licensee's emergency response capability.

In view of the foregoing, I have determined that the implementation of CP&L's commitments are required in the interest of the public health and safety and should, therefore, be confirmed by an immediately effective Order.

V.

Accordingly, pursuant to Sections 103, 161i, 161o and 182 of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR 2.204 and 10 CFR Part 50, IT IS HEREBY ORDERED, EFFECTIVE IMMEDIATELY, THAT license DPR-20 is modified to provide that the licensee shall:

Implement the specific items described in the Attachment to this ORDER in the manner described in CP&L's submittals noted in Section IV herein no later than the dates in the Attachment.

Extension of time for completing these items may be granted by the Director, Division of Licensing, for good cause shown.

The licensee or any other person with an adversely affected interest may request a hearing on this Order within 20 days of the date of publication of this Order in the Federal Register. Any request for a hearing should be addressed to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. A copy should be sent to the

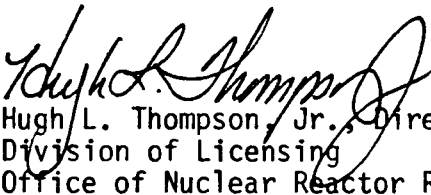
Executive Legal Director at the same address. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is to be held, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether the licensee should comply with the requirements set forth in Section V of this Order.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Hugh L. Thompson, Jr., Director
Division of Licensing
Office of Nuclear Reactor Regulation

Dated in Bethesda, Maryland
this 9th day of July, 1985.

Attachment:
Licensee's Additional Commitments
on Requirements Specified in
Supplement 1 to NUREG-0737

LICENSEE'S ADDITIONAL COMMITMENTS ON SUPPLEMENT 1 TO NUREG-0737

TITLE	REQUIREMENTS	LICENSEE'S COMPLETION SCHEDULE (OR STATUS)
1. Safety Parameter Display System (SPDS)	1b. SPDS fully operational and operators trained.	Scheduled to be fully operational 12 months after RO #10
2. Detailed Control Room Design Review (DCRDR)	2b. Submit a summary report to the NRC including a proposed schedule for implementation.	Summary report and proposed implementation schedule - scheduled for 6 months after RO #10
3. Regulatory Guide 1.97 - Application to Emergency Response Facilities	3b. Implement (installation or upgrade) requirements.	Full compliance to RG 1.97 scheduled from 3 months after RO #11
4. Upgrade Emergency Operating Procedures (EOPs)	4a. Submit a Procedures Generation Package to the NRC:	Complete
	4b. Implement the upgraded EOPs.	Complete
5. Emergency Response Facilities	5a. Technical Support Center fully functional.	Fully functional scheduled for 12 months after RO #10
	5b. Operational Support Center fully functional.	Complete
	5c. Emergency Operations Facility fully functional.	Fully functional schedule for 12 months after RO #10