



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 86
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power and Light Company (the licensee) dated January 9, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-23 is hereby amended to read as follows:

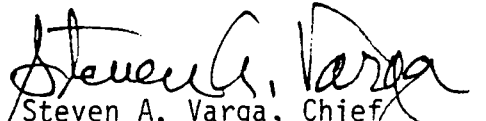
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(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 86, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 12, 1984

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 86 FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise Appendix A as follows:

Remove Page

3.4-5

Insert Page

3.4-5

TABLE 3.4-1

AUXILIARY FEEDWATER FLOW AUTOMATIC INITIATION*

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1</u> MINIMUM CHANNELS OPERABLE	<u>2</u> MINIMUM DEGREE OF REDUNDANCY	<u>3</u> OPERATOR ACTION IF CONDITIONS OF COLUMN 1 OR 2 CANNOT BE MET
1.	Steam Gen. Water Level-low-low a. Start Motor-Driven Pumps b. Start Turbine-Driven Pump	2/Steam Generator 2/Steam Generator	1/Steam Generator 1/Steam Generator	Maintain Hot Shutdown Maintain Hot Shutdown
2.	Undervoltage-4KV Busses 1 & 4 Start Turbine-Driven Pump (15 Second Time Delay Pickup)	2 Per Bus	0	Note 1
3.	S.I. Start Motor-Driven Pumps	See Table 3.5-3, Item No.1		
4.	Station Blackout Start Motor-Driven Pumps (40 Second Time Delay Prior to Starting MD AFW Pumps on Black- out Sequence)	2 Per Bus	0	Note 2
5.	Trip of Main Feedwater Pumps Start Motor-Driven Pumps	1/Pump	0	Note 2

* This table is applicable whenever the RCS is $>350^{\circ}\text{F}$ except Item 5. Item 5 is applicable only when the RCS is at normal operating temperature and the reactor is critical.

Note 1: 4KV Busses 1, 2, and 4 each have two undervoltage relays. One relay on each of the three busses provides an input to the reactor trip logic. Both relays on Busses 1 and 4 provide inputs to the SD AFW pump start logic. If the undervoltage relay on Busses 1 or 4 that provides the input to the reactor trip logic fails, follow the requirements of Table 3.5-2 Item 14 in addition to the following. If either 4KV undervoltage relay on Busses 1 or 4 fails, within 4 hours insert the equivalent of an undervoltage signal from the affected relay in the SD AFW pump start circuit and repair the affected relay within 7 days. If the affected relay is not repaired in the 7 days, then commence a normal plant shutdown to hot standby.

Note 2: Restore the inoperable channel to operable status within 48 hours. If the inoperable channel is not restored to an operable status within 48 hours, then commence a normal plant shutdown and cooldown to $\leq 350^{\circ}\text{F}$.