



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.77
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power and Light Company (the licensee) dated July 1, 1982, as supplemented, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the Facility Operating License No. DPR-23 is hereby amended by deleting paragraph 3.I and adding a new paragraph 3.J as follows:

3.J Steam Generator Repair Program

- (1) The H. B. Robinson Steam Electric Plant Unit No. 2 Steam Generator Repair Program is approved.
- (2) The following temporary license conditions will be imposed before and during the repair program.

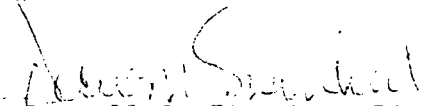
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- (a) All fuel shall be removed from the reactor pressure vessel and stored in the spent fuel pool before the steam generator repair task is initiated.
- (b) Before the steam generator repair task is initiated, the following planning and preparation (i.e., specific applications identified and details finalized) shall be completed in accordance with Regulatory Guide 8.8.
 - (i) general area decontamination,
 - (ii) primary surface decontamination,
 - (iii) use of temporary shielding,
 - (iv) use of specialized tools,
 - (v) removal of selected valves and piping,
 - (vi) establishment of low-background wait areas,
 - (vii) establishment of laydown areas,
 - (viii) training and training facilities for plant and contractor personnel,
 - (ix) access control,
 - (x) equipment decontamination and decontamination facilities,
 - (xi) engineering controls which limit the need for respiratory protection consistent with ALARA,
 - (xii) dose tracking.
- (c) Interim reports which summarize each 90-day period of the repair effort are to be provided to the NRC within 60 days of the completion of each such report period.
- (d) The collective occupational dose estimates will be updated each 90-day period. If the updated estimates exceed the person-rem estimate by more than 10%, the licensee will provide a revised estimate, including the reasons for such changes, to the NRC with the 90-Day Progress Reports.
- (e) Sixty days prior to fuel loading, the program for preoperational testing and startup shall be submitted for NRC review (3.3.16) (4.2).
- (f) A final report shall be provided to the NRC within 60 days after repair completion. This report shall include the following:
 - (i) a summary by major task of the occupational dose received,
 - (ii) a comparison of estimated doses with the doses actually received,
 - (iii) a discussion of ALARA measures employed, and

(iv) a summary of decontamination efforts and radwaste generation.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Darrell G. Eisenhut, Director
Division of Licensing

Date of Issuance: February 28, 1984