



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Carolina Power and Light Company (the licensee) dated October 22, 1982 and January 20, 1983, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

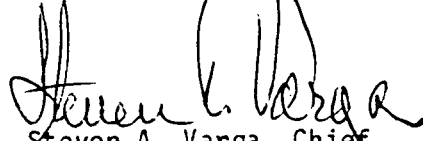
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-23 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 75, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 25, 1983

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise Appendix A as follows:

Remove Pages

3.1-2

3.4-2

3.4-2a

3.4-3

Insert Pages

3.1-2

3.4-2

3.4-2a

3.4-3

3.1.1.2 Steam Generator

At least two steam generators shall be operable whenever the average primary coolant temperature is above 350°F.

3.1.1.3 Pressurizer (Pzr)

- a. At least one Pzr code safety valve shall be operable whenever the Reactor Head is on the vessel and the RCS is not open for maintenance.
- b. The Pzr, including necessary spray and heater control systems, shall be operable before the reactor is made critical.
- c. Whenever the RCS temperature is above 350°F or the reactor is critical:
 1. All three pressurizer code safety valves shall be operable. Their lift settings shall be maintained between 2485 psig and 2560 psig.
 2. At least 125 kw of pressurizer heaters capable of being powered from an emergency power source shall be operable.
- d. If the requirements of 3.1.1.3.c.2 are not met and at least 125 kw of Pzr heaters capable of being powered from an emergency source cannot be provided within 72 hrs., commence a normal plant shutdown and cooldown to an RCS average temperature of less than or equal to 350°F.

3.4.2 The specific activity of the secondary coolant system shall be $< 0.10 \mu\text{Ci}/\text{gram}$ DOSE EQUIVALENT I-131 under all modes of operation from cold shutdown through power operation. When the specific activity of the secondary coolant system is $> 0.10 \mu\text{Ci}/\text{gram}$ DOSE EQUIVALENT I-131, be in at least HOT SHUTDOWN within 6 hours and COLD SHUTDOWN within the following 30 hours.

The specific activity of the secondary coolant system shall be determined to be within the limit by performance of the sampling and analysis program of Table 4.1-2.

3.4.3 If, during power operations, any of the specifications in 3.4.1, with the exception of 3.4.1.b and 3.4.1.d as it applies to 3.4.1.b above, cannot be met within 24 hours, the operator shall initiate procedures to put the plant in the hot shutdown condition. If any of these specifications cannot be met within an additional 48 hours, the operator shall cool the reactor below 350°F using normal procedures.

3.4.4 With one auxiliary feedwater pump and/or essential features INOPERABLE, restore that auxiliary feedwater pump and/or essential features to OPERABLE status within 72 hours, or;

a. Submit a special report to the Commission in accordance with Specification 6.9.3.f within 30 days outlining the cause of the inoperability and the action taken to return the pump and/or essential features to OPERABLE status, and;

b. Restore all three auxiliary feedwater pumps and their essential features to operable status within 7 days or be in at least HOT SHUTDOWN within 6 hours.

3.4.5 With two auxiliary feedwater pumps INOPERABLE, restore at least one inoperable auxiliary feedwater pump to OPERABLE status within 24 hours or be in at least HOT SHUTDOWN within 6 hours.

3.4.6 In the event that the number of channels of the Auxiliary Feedwater Initiation circuits falls below the limits given in the column entitled Minimum Operable Channels, or Minimum Degree of Redundancy cannot be achieved, operation shall be limited according to the requirements shown in Column 3 of Table 3.4-1. The Auxiliary Feedwater System Automatic Initiation Setting Limits are shown in Table 3.4-2. If the setpoint is less conservative than the value shown in the Allowable Values column to Table 3.4-2, declare the channel inoperable and operation shall be limited according to the requirement shown in Column 3 of Table 3.4-1.

Basis

A reactor shutdown from power requires removal of core decay heat. Immediate decay heat removal requirements are normally satisfied by the steam bypass to the condenser. Therefore, core decay heat can be continuously dissipated via the steam bypass to the condenser as feedwater in the steam generator is converted to steam by heat absorption. Normally, the capability to return feedwater flow to the steam generators is provided by operation of the turbine cycle feedwater system.

The twelve main steam safety valves have a total combined rated capability of 10,068,845 lbs/hr. The total full power steam flow is 10,068,845 lbs/hr.; therefore, twelve (12) main steam safety valves will be able to relieve the total steam flow if necessary.⁽¹⁾ Following a loss of load, which represents the worst transient, steam flows are below the total capacity of the 12 safety valves. Therefore, over-pressurization of the secondary system is not possible.

In the unlikely event of complete loss of turbine-generator and offsite electrical power to the plant, decay heat removal would continue to be assured by the availability of either the steam-driven auxiliary feedwater pump or one of the two motor-driven auxiliary steam generator feedwater pumps operated from the diesel generators and steam discharge to the atmosphere via the main steam safety valves and atmospheric relief valves. One motor-driven auxiliary feedwater pump can supply sufficient feedwater for removal of decay heat from the plant.⁽²⁾ The auxiliary feedwater system essential features are those features that provide auxiliary feedwater flow to two out of three steam generators consistent with auxiliary feedwater pump operability. In order to provide a high degree of reliability all three auxiliary feedwater pumps will be operable prior to exceeding 350°F. The minimum amount of water in the condensate storage tank is the amount needed for at least two-hours operation at hot standby conditions. If the outage is more than two hours, deep well or Lake Robinson water may be used.

An unlimited supply is available from the lake via either leg of the plant Service Water System for an indefinite time period.