



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 65  
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Carolina Power and Light Company (the licensee) dated January 26, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

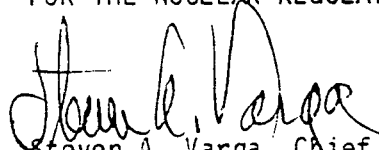
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-23 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 65, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 11, 1982

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 65 TO FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise Appendix A as follows:

Remove Pages

4.1-4  
4.1-6  
4.1-6a

Insert Pages

4.1-4  
4.1-6  
4.1-6a

TABLE 4.1-1  
MINIMUM FREQUENCIES FOR CHECKS, CALIBRATIONS AND TEST OF INSTRUMENT CHANNELS

<u>Channel</u>	<u>Check</u>	<u>Calibration</u>	<u>Test</u>	<u>Remarks</u>
1. Nuclear Power Range	S	D (1) M* (3) R* (4)	B/W (2)	(1) Thermal Power calculations during power operations (2) Signal to $\Delta T$ ; bistable action (permissive, rod stop, trips) (3) Upper and lower chambers for symmetric offset: monthly during power operations. When periods of reactor shutdown extend this interval beyond one month, the calibration shall be performed immediately following return to power. (4) Calibration of excore detectors.
2. Nuclear Intermediate Range	S (1)	N.A.	S/U (2)	(1) Once/shift when in service (2) Log level; bistable action (permissive, rod stop, trip)
3. Nuclear Source Range	S (1)	N.A.	S/U (2)	(1) Once/shift when in service (2) Bistable action (alarm, trip)
4. Reactor Coolant Temperature	S	R	B/W (1) (2)	(1) Overtemperature- $\Delta T$ (2) Overpower- $\Delta T$
5. Reactor Coolant Flow	S	R	M	
6. Pressurizer Water Level	S	R	M	
7. Pressurizer Pressure	S	R	M	
8. 4 kv Voltage	N.A.	R	M	Reactor Protection circuits only

\*By means of the movable in-core detector system

TABLE 4.1-1 (Continued)

<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
21. Containment Sump Level	N.A.	R	N.A.	
22. Turbine Trip Set Point **	N.A.	R	R	
23. Accumulator Level and Pressure	S	R	N.A.	
24. Steam Generator Pressure	S	R	M	
25. Turbine First Stage Pressure	S	R	M	
26. Emergency Plant Portable Survey Instruments	M	R	M	
27. Logic Channel Testing	N.A.	N.A.	M(1)	(1). During hot shutdown and power operations. When periods of reactor cold shutdown and re-fueling extend this interval beyond one month, the test shall be performed prior to startup.
28. Turbine Overspeed Protection Trip Channel (Electrical)	N.A.	R	M	
29. 4 Kv Frequency	N.A.	R	R	
30. Control Rod Drive Trip Breakers	N.A.	N.A.	M	
31. Overpressure Protection System	N.A.	R	M	

\*\*Stop valve closure or low EH fluid pressure

TABLE 4.1-1 (Continued)

MINIMUM FREQUENCIES FOR CHECKS, CALIBRATIONS AND TEST OF INSTRUMENT CHANNELS

<u>Channel Description</u>	<u>Check</u>	<u>Calibration</u>	<u>Test</u>	<u>Remarks</u>
32. Loss of Power				
a. 480 Emerg. Bus Undervoltage (Loss of Voltage)	N.A.	R	R	
b. 480 Emerg. Bus Undervoltage (Degraded Voltage)	N.A.	R	R	
33. Auxiliary Feedwater Flow**** Indication	M	N.A.	R	
34. Reactor Coolant System** Subcooling Monitor	N	R	N.A.	
35. PORV Position Indicator***	N.A.	N.A.	R	
36. PORV Blocking Valve*** Position Indicator	N.A.	N.A.	R	
37. Safety Relief Valve Position*** Indicator	N.A.	N.A.	R	

\*\* Instrumentation for Detection of Inadequate Core Cooling - NUREG 0578 Item 2.1.3.b.

\*\*\* Direct Indication of Power Operated Relief Valve and Safety Valve Position - NUREG 0578 Item 2.1.3.a.

\*\*\*\* Auxiliary Feedwater Flow Indication to Steam Generator NUREG 0578 Item 2.1.7.b.

S - At least once per 12 hours  
D - At least once per 24 hours  
W - At least once per 7 days  
B/W - At least once per 14 days  
M - At least once per 31 days  
Q - At least once per 92 days  
S/U - Prior to each reactor startup  
R - At least once per 18 months  
N.A. - Not applicable