



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power and Light Company (the licensee) dated August 27, 1981, as revised by letter dated August 28, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by the addition of a new paragraph 3.I to Facility Operating License No. DPR-23 to read as follows:
 - 3.I The following operating license condition is effective from the time H. B. Robinson Unit 2 returns to power operation subsequent to the August 1981 steam generator inspection and to remain in effect until the next refueling outage:
 - a. A primary to secondary pressure test at approximately 1900 psi differential shall be performed after operation at power levels such that estimated corrosion is equivalent to or less than

that of 24 effective full power days operation as shown in figure 4.3.3 in Attachment B of CP&L's letter of August 21, 1981. A period of seven additional calendar days is permitted for flexibility for scheduling the test.

- b. At the end of core life of the present cycle, an eddy current examination will be performed. The scope of this inspection will be submitted to the NRC for approval at least 45 days calendar days prior to this end of core life inspection.
 - c. During the remainder of the cycle 8 operations, the following steam generator tube leakage criteria shall be in effect. Specifically, the plant shall be shut down if the verified primary to secondary leakage in one steam generator exceeds any of the following:
 - 1. A sudden increase of 0.1 gallon per minute (gpm) if the total leakage rate in that steam generator exceeds 0.2 gpm.
 - 2. If the leakage rate in that steam generator exceeds 0.2 gpm and an upward trend in leakage rate in excess of 0.02 gpm per day is verified. This trend will be established using at least five valid consecutive daily samples.
 - d. Should the plant be required to shut down to repair a steam generator tube leak as indicated in item (c) above, an inspection shall be performed as mutually agreed upon by the NRC Staff and CP&L.
 - e. The NRC Staff shall be provided with a summary of the results of the eddy current examination performed under item (b) above.
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga
Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to License No. DPR-23

Date of Issuance: August 28, 1981

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 60 TO FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Revise License as follows:

Remove Pages

4a

4b

Insert Pages

4a

4b

- 3.I The following operating license condition is effective from the time the H. B. Robinson Unit 2 returns to power operation subsequent to the August 1981 steam generator inspection and to remain in effect until the next refueling outage:
- a. A primary to secondary pressure test at approximately 1900 psi differential shall be performed after operation at power levels such that estimated corrosion is equivalent to that of 24 effective full power days operation as shown in figure 4.3.3 in Attachment B of CP&L's letter of August 27, 1981. A period of seven additional calendar days is permitted for flexibility for scheduling the necessary test.
 - b. At the end of core life of the present cycle, an eddy current examination shall be performed. The scope of this inspection will be submitted to the NRC for approval at least 45 calendar days prior to this end of core life inspection.
 - c. During the remainder of the cycle 8 operations, the following steam generator tube leakage criteria shall be in effect. Specifically, the plant shall be shut down if the verified primary to secondary leakage in one steam generator exceeds any of the following:
 1. A sudden increase of 0.1 gallon per minute (gpm) if the total leakage rate in that steam generator exceeds 0.2 gpm.
 2. If the leakage rate in that steam generator exceeds 0.2 gpm and an upward trend in leakage rate in excess of 0.02 gpm per day is verified. This trend will be established using at least five valid consecutive daily samples.
 - d. Should the plant be required to shut down to repair a steam generator tube leak as indicated in item (c) above, an inspection shall be performed as mutually agreed upon by the NRC Staff and CP&L.
 - e. The NRC Staff shall be provided with a summary of the results of the eddy current examination performed under item (b) above.
4. CP&L shall observe such standards and requirements for the protection of the environment as are validly imposed pursuant to authority established under Federal and State law and as are determined by the Commission to be applicable to the facility covered by this operating license. This condition does not apply to (a) radiological effects since such effects are dealt with in other provisions of this operating license or (b) matters of water quality covered by section 21(b) of the Federal Water Pollution Control Act.
- Deleted:
Attachment
#2, dtd
11-15-72*

5. This license is effective as of the date of issuance, and shall expire at midnight April 13, 2007.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by
Peter A. Morris

Peter A. Morris, Director
Division of Reactor Licensing

Attachment:
Appendix A - Technical Specifications

Date of Issuance: JUL 31 1970