



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CAROLINA POWER AND LIGHT COMPANY

DOCKET NO. 50-261

H. B. ROBINSON, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50  
License No. DPR-23

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Carolina Power and Light Company (the licensee) dated December 18, 1974, as supplemented September 20, 1979 and July 14, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, paragraph 2 of the Facility Operating License DPR-23 is amended as indicated below:

- B. Pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report as supplemented and amended;
- C. Pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration and as fission detectors in amounts as required;
- D. Pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components.
- E. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

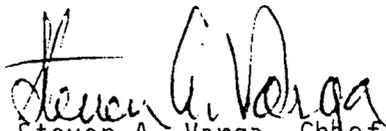
The license is also amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License DPR-23 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 50, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: September 30, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 50

FACILITY OPERATING LICENSE NO. DPR-23

DOCKET NO. 50-261

Replace the following pages of the Appendix A Technical Specification with the enclosed pages. The revised pages are identified by amendment number and contains a vertical line indicating the area of change.

Remove

ii

Insert

ii  
4.16-1

| <u>Section</u> | <u>Title</u>   | <u>Page</u> |
|----------------|--|-------------|
| 3.10.5         | Deleted  | 3.10-8      |
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| 3.10.7         | Power Ramp Rate Limits   | 3.10-9      |
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| 3.12           | Seismic Shutdown   | 3.13-1      |
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| 3.14           | Fire Protection System   | 3.14-1      |
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| 3.14.2         | Fire Suppression Water System  | 3.14-2      |
| 3.14.3         | CO <sub>2</sub> Fire Protection System                                       | 3.14-2a     |
| 3.14.4         | Fire Hose Stations   | 3.14-3      |
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| 4.3            | Primary System Testing Following Opening                                     | 4.4-1       |
| 4.4            | Containment Tests  | 4.4-1       |
| 4.4.1          | Operational Leakage Rate Tests   | 4.4-4       |
| 4.4.2          | Isolation Valve Tests  | 4.4-4       |
| 4.4.3          | Post Accident Recirculation Heat Removal System                              | 4.4-5       |
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| 4.5            | Emergency Core Cooling, Containment Cooling and Iodine Removal Systems Tests | 4.5-1       |
| 4.5.1          | System Tests   | 4.5-1       |
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| 4.7            | Secondary Steam and Power Conversion System                                  | 4.8-1       |
| 4.8            | Auxiliary Feedwater System   | 4.9-1       |
| 4.9            | Reactivity Anomalies   | 4.12-1      |
| 4.10           | Radioactive Effluents  | 4.12-1      |
| 4.11           | Reactor Core   | 4.12-1      |
| 4.12           | Refueling Filter Systems   | 4.13-1      |
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| 4.14           | Fire Protection System   | 4.15-1      |
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| 5.0            | Design Features  | 5.1-1       |
| 5.1            | Site   | 5.2-1       |
| 5.2            | Containment  | 5.2-1       |
| 5.2.1          | Reactor Containment  | 5.2-1       |
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#### 4.16 RADIOACTIVE SOURCE LEAKAGE TESTING

##### Applicability:

Applies to by-product, source and special nuclear radioactive material used at H. B. Robinson Unit 2.

##### Objective:

The objective of this specification is to assure that leakage from by-product, source, and special nuclear radioactive material sources does not exceed allowable limits.

##### Specification:

4.16.1 The leakage test shall be capable of detecting the presence of .005 microcurie of radioactive material on the test sample. If the test reveals the presence of .005 microcurie or more of removable contamination, it shall immediately be withdrawn from use, decontaminated, and repaired, or be disposed of in accordance with Commission regulations. Sealed sources are exempt from such leak tests when the source contains 100 microcuries or less of beta and/or gamma emitting material or 10 microcuries or less of alpha emitting material.

4.16.2 Tests for leakage and/or contamination shall be performed by the licensee or by other persons specifically authorized by the Commission or an agreement State as follows:

- A. Each sealed source, except startup sources subject to core flux, containing radioactive material, other than Hydrogen 3, with a half-life greater than thirty days and in any form other than gas shall be tested for leakage and/or contamination at intervals not to exceed six months.
- B. The periodic leak test required does not apply to sealed sources that are stored and not being used. The sources excepted from this test shall be tested for leakage prior to any use or transfer to another user unless they have been leak tested within six months prior to the date of use or transfer.

In the absence of a certificate from a transferor indicating that a test has been made within six months prior to the transfer, sealed sources shall not be put into use until tested.

- C. Startup sources shall be leak tested prior to and following any repair or maintenance and before being subjected to core flux.