



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

July 2, 2014

Mr. C. R. Pierce  
Regulatory Affairs Director  
Southern Nuclear Operating Co., Inc.  
P.O. Box 1295, Bin 038  
Birmingham, AL 35201-1295

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2, REQUEST FOR  
ADDITIONAL INFORMATION (TAC NOS. MF3687 AND MF3688)

Dear Mr. Pierce:

By letter dated March 24, 2014, the Southern Nuclear Operating Company, Inc. (SNC, the licensee) submitted a proposed alternative to the inservice inspection requirement of ASME Code Case N-770-1 for the reactor pressure vessel cold leg nozzle dissimilar metal welds for Joseph M. Farley Nuclear Plant, Units 1 and 2, during the fourth inservice inspection interval.

The U.S. Nuclear Regulatory Commission staff has determined that additional information is needed as discussed in the Enclosure. We request that SNC respond within 30 days of the date of this letter. Please note that the NRC staff's review is continuing and further requests for information may be developed.

Sincerely,

A handwritten signature in black ink, reading "Shawn Williams", is positioned above the typed name.

Shawn Williams, Project Manager  
Plant Licensing Branch, II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosure:  
Request for Additional Information

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION

JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2

PROPOSED ALTERNATIVE FNP-ISI-ALT-15, VERSION 1.0

REGARDING DEFERRAL OF INSERVICE INSPECTION OF REACTOR

PRESSURE VESSEL COLD LEG NOZZLE DISSIMILAR METAL WELD

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

DOCKET NOS. 50-348 AND 50-364

By letter dated March 24, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML14084A203), Southern Nuclear Operating Company (the licensee) submitted for the U.S. Nuclear Regulatory Commission (NRC) approval request for alternative (RFA) FNP-ISI-ALT-15, Version 1. The licensee proposed an alternative to a certain requirement of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI. The request relates to the inservice inspection (ISI) requirement of ASME Code Case N-770-1 for the reactor pressure vessel (RPV) cold leg nozzle dissimilar metal (DM) welds. The licensee submitted the request for the Joseph M. Farley Nuclear Plant (Farley), Units 1 and 2.

The NRC staff requests the following additional information.

**RAI NO. 1**

Provide detail description of the geometric and surface conditions of the ASME Code required inside diameter (ID) surface area of the subject DM welds examined by the eddy current testing (ET).

**RAI NO. 2**

Discuss whether the full examination coverage of the ASME Code required surface area of the DM welds was obtained when performing the ET from the ID surface.

**RAI NO. 3**

Describe the qualification process used for the ET procedure.

**RAI NO. 4**

What is the longest and deepest ID surface-connected flaw that the ET procedure would not be qualified to detect?

Enclosure

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/RA/

Shawn Williams, Project Manager  
Plant Licensing Branch, II-1  
Division of Operating Reactor Licensing  
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**ADAMS Accession No.: ML14174B129**

\*concurrence by e-mail

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