

ATTACHMENT 8

**DISCUSSION OF REVISION
TO THE
RADIOLOGICAL EMERGENCY PLAN ANNEX
FOR
OYSTER CREEK NUCLEAR GENERATING STATION
EP-AA-1010**

Enclosures

- Enclosure 8A - EAL Comparison Matrix Document
- Enclosure 8B - EAL Red-Line Basis Document
- Enclosure 8C - EAL Basis Document

NEI 99-01

REVISION 6

DEVELOPMENT OF EMERGENCY ACTION LEVELS FOR NON-PASSIVE REACTORS

ATTACHMENT 8

DISCUSSION OF REVISION TO THE RADIOLOGICAL EMERGENCY PLAN ANNEX FOR OYSTER CREEK NUCLEAR GENERATING STATION



ExelonGeneration®

NEI 99-01 Rev 6	Proposed EAL	Justification										
<p style="text-align: right;">AG1</p> <p>Initiating Condition – GENERAL EMERGENCY</p> <p>Release of gaseous radioactivity resulting in offsite dose greater than 1,000 mrem TEDE or 5,000 mrem thyroid CDE.</p> <p>Operating Mode Applicability: All</p> <p>Example Emergency Action Levels: (1 or 2 or 3)</p> <p>Notes:</p> <ul style="list-style-type: none"> The Emergency Director should declare the General Emergency promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded 15 minutes. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes. The pre-calculated effluent monitor values presented in EAL #1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available. <p>1. Reading on any of the following radiation monitors greater than the reading shown for 15 minutes or longer:</p> <p>(site specific monitor list and threshold values)</p> <p>2. Dose assessment actual meteorology indicates doses greater than 1000 mrem TEDE or 5000 mrem thyroid CDE at or beyond (site specific dose receptor point)</p> <p>3. Field survey results indicate EITHER of the following at or beyond (site specific dose receptor point):</p> <ul style="list-style-type: none"> Closed window dose rates greater than 1000 mR/hr expected to continue for 60 minutes or longer. Analysis of field survey samples indicate thyroid CDE greater than 5000 mrem for one hour of inhalation. 	<p style="text-align: right;">RG1</p> <p>Initiating Condition:</p> <p>Release of gaseous radioactivity resulting in offsite dose greater than 1,000 mRem TEDE or 5,000 mRem thyroid CDE.</p> <p>Operating Mode Applicability:</p> <p>1,2,3,4,D</p> <p>Emergency Action Level (EAL):</p> <p>Notes:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded 15 minutes. Classification based on effluent monitor readings assumes that a release path to the environment is established. 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Analyses of field survey samples indicate > 5000 mRem CDE Thyroid for 60 minutes of inhalation. <table border="1" data-bbox="751 1075 1333 1252" style="width: 100%; border-collapse: collapse;"> <thead> <tr> <th colspan="2" style="text-align: center;">Table R1 Effluent Monitor Thresholds</th> </tr> <tr> <th style="text-align: center;">Effluent Monitor</th> <th style="text-align: center;">General Emergency</th> </tr> </thead> <tbody> <tr> <td style="text-align: center;">Main Stack RAGEMS</td> <td style="text-align: center;">8.69 E+01 uCi/cc HRM OR 3.53 E-08 amps HRM</td> </tr> <tr> <td style="text-align: center;">Turbine Building RAGEMS</td> <td style="text-align: center;">7.17 E-01 uCi/cc HRM</td> </tr> <tr> <td colspan="2" style="text-align: center;">HRM = High Range Monitor</td> </tr> </tbody> </table>	Table R1 Effluent Monitor Thresholds		Effluent Monitor	General Emergency	Main Stack RAGEMS	8.69 E+01 uCi/cc HRM OR 3.53 E-08 amps HRM	Turbine Building RAGEMS	7.17 E-01 uCi/cc HRM	HRM = High Range Monitor		<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>Listed site-specific monitors and Threshold values to ensure timely classification.</p> <p>1) Added the following to bullet #3 " Classification based on effluent monitor readings assumes that a release path to the environment is established." In order to delete the following from the basis "Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes." This allows for more timely classification since all the basis information pertaining to Note bullet 3 will be contained in the IC and therefor readily available on the 11x17 procedure matrix used by the SM.</p>
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The pre-calculated effluent monitor values presented in EAL #1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available. <p>1. Reading on any of the following radiation monitors greater than the reading shown for 15 minutes or longer:</p> <p>(site specific monitor list and threshold values)</p> <p>2. Dose assessment actual meteorology indicates doses greater than 1000 mrem TEDE or 5000 mrem thyroid CDE at or beyond (site specific dose receptor point)</p> <p>3. Field survey results indicate EITHER of the following at or beyond (site specific dose receptor point):</p> <ul style="list-style-type: none"> Closed window dose rates greater than 100 mR/hr expected to continue for 60 minutes or longer. Analysis of field survey samples indicate thyroid CDE greater than 500 mrem for one hour of inhalation. 	<p style="text-align: right;">RS1</p> <p>Initiating Condition:</p> <p>Release of gaseous radioactivity resulting in offsite dose greater than 100 mRem TEDE or 500 mRem thyroid CDE.</p> <p>Operating Mode Applicability:</p> <p>1,2,3,4,D</p> <p>Emergency Action Level (EAL):</p> <p>Notes:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded 15 minutes. Classification based on effluent monitor readings assumes that a release path to the environment is established. 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In order to delete the following from the basis "Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes." This allows for more timely classification since all the basis information pertaining to Note bullet 3 will be contained in the IC and therefore readily available on the 11x17 procedure matrix used by the SM.</p>
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<p style="text-align: right;">AA1</p> <p>Initiating Condition – ALERT</p> <p>Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE or 50 mrem thyroid CDE.</p> <p>Operating Mode Applicability: All</p> <p>Example Emergency Action Levels: (1 or 2 or 3)</p> <p>Note:</p> <ul style="list-style-type: none"> The Emergency Director should declare the Alert promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded 15 minutes. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes. 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Field survey results indicate EITHER of the following at or beyond (site specific dose receptor point):</p> <ul style="list-style-type: none"> Closed window dose rates greater than 10 mR/hr expected to continue for 60 minutes or longer. Analysis of field survey samples indicate thyroid CDE greater than 50 mrem for one hour of inhalation. 	<p style="text-align: right;">RA1</p> <p>Initiating Condition:</p> <p>Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE or 50 mrem thyroid CDE.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>Notes:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded 15 minutes. Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes. The pre-calculated effluent monitor values presented in EAL #1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available. <p>1. Readings on ANY Table R1 Effluent Monitor > Table R1 value for ≥ 15 minutes. OR</p> <p>2. Dose assessment using actual meteorology indicates doses at or beyond the site boundary of EITHER:</p> <p style="padding-left: 20px;">a. > 10 mRem TEDE OR</p> <p style="padding-left: 20px;">b. > 50 mRem CDE Thyroid</p> <p style="text-align: center;">OR</p> <p>3. Analysis of a liquid effluent sample indicates a concentration or release rate that would result in doses greater than EITHER of the following at or beyond the site boundary</p> <p style="padding-left: 20px;">a. 10 mRem TEDE for 60 minutes of exposure OR</p> <p style="padding-left: 20px;">b. 50 mRem CDE Thyroid for 60 minutes of exposure</p> <p style="text-align: center;">OR</p> <p>4. Field survey results at or beyond the site boundary indicate EITHER:</p> <p style="padding-left: 20px;">a. Gamma (closed window) dose rates > 10 mR/hr are expected to continue for ≥ 60 minutes. OR</p> <p style="padding-left: 20px;">b. Analyses of field survey samples indicate > 50 mRem CDE Thyroid for 60 minutes of inhalation.</p> <table border="1" style="width: 100%; border-collapse: collapse; margin-top: 10px;"> <thead> <tr> <th colspan="2" style="text-align: center;">Table R1 Effluent Monitor Thresholds</th> </tr> <tr> <th style="text-align: center;">Effluent Monitor</th> <th style="text-align: center;">Alert</th> </tr> </thead> <tbody> <tr> <td style="text-align: center;">Main Stack RAGEMS</td> <td style="text-align: center;">8.69 E-01 uCi/cc HRM OR 3.53 E-10 amps HRM</td> </tr> <tr> <td style="text-align: center;">Turbine Building RAGEMS</td> <td style="text-align: center;">3.65 E+04 cpm LRM</td> </tr> <tr> <td colspan="2" style="text-align: center;">HRM = High Range Monitor LRM = Low Range Monitor</td> </tr> </tbody> </table>	Table R1 Effluent Monitor Thresholds		Effluent Monitor	Alert	Main Stack RAGEMS	8.69 E-01 uCi/cc HRM OR 3.53 E-10 amps HRM	Turbine Building RAGEMS	3.65 E+04 cpm LRM	HRM = High Range Monitor LRM = Low Range Monitor		<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>Listed site-specific monitors and Threshold values to ensure timely classification.</p> <p>1) Added the following to bullet #3 "Classification based on effluent monitor readings assumes that a release path to the environment is established." 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<p style="text-align: right;">AU1</p> <p>Initiating Condition – UNUSUAL EVENT</p> <p>Release of gaseous or liquid radioactivity greater than 2 times the (site-specific effluent release controlling document) limits for 60 minutes or longer</p> <p>Operating Mode Applicability: All</p> <p>Example Emergency Action Levels: (1 or 2 or 3)</p> <p>Note:</p> <ul style="list-style-type: none"> The Emergency Director should declare the Unusual Event promptly upon determining that 60 minutes has been exceeded, or will likely be exceeded. If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded 60 minutes. If the effluent flow past an effluent monitor is known to have stopped, indicating that the release path is isolated, the effluent monitor reading is no longer valid for classification purposes. <ol style="list-style-type: none"> Reading on ANY effluent radiation monitor greater than 2 times the (site-specific effluent release controlling document) limits for 60 minutes or longer: (site-specific monitor list and threshold values corresponding to 2 times the controlling document limits) Reading on ANY effluent radiation monitor greater than 2 times the alarm setpoint established by a current radioactivity discharge permit for 60 minutes or longer. Sample analysis for a gaseous or liquid release indicates a concentration or release rate greater than 2 times (site-specific effluent release controlling document limits) for 60 minutes or longer. 	<p style="text-align: right;">RU1</p> <p>Initiating Condition:</p> <p>Release of gaseous or liquid radioactivity greater than 2 times the ODCM limits for 60 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>Notes:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded 60 minutes. Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes. <p>1. Readings on ANY Table R1 Effluent Monitor > Table R1 value for ≥ 60 minutes:</p> <table border="1" data-bbox="751 723 1346 888"> <thead> <tr> <th colspan="2">Table R1 Effluent Monitor Thresholds</th> </tr> <tr> <th>Effluent Monitor</th> <th>Unusual Event</th> </tr> </thead> <tbody> <tr> <td>Main Stack RAGEMS</td> <td>4.07 E+03 cps LRM</td> </tr> <tr> <td>Turbine Building RAGEMS</td> <td>4.16 E+02 cpm LRM</td> </tr> <tr> <td colspan="2" style="text-align: center;">LRM = Low Range Monitor</td> </tr> </tbody> </table> <p>OR</p> <p>2. Confirmed sample analyses for gaseous or liquid releases indicate concentrations or release rates > 2 times ODCM Limit with a release duration of ≥ 60 minutes.</p>	Table R1 Effluent Monitor Thresholds		Effluent Monitor	Unusual Event	Main Stack RAGEMS	4.07 E+03 cps LRM	Turbine Building RAGEMS	4.16 E+02 cpm LRM	LRM = Low Range Monitor		<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>Listed site-specific monitors and Threshold values to ensure timely classification.</p> <p>1) Added the following to bullet #3 " Classification based on effluent monitor readings assumes that a release path to the environment is established." In order to delete the following from the basis "Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes." This allows for more timely classification since all the basis information pertaining to Note bullet 3 will be contained in the IC and therefore readily available on the 11x17 procedure matrix used by the SM.</p> <p>2) Calculation was performed to determine the radiation monitor response for a radioactive liquid release with an activity equivalent to provide 2 times the ODCM. The calculation determined the liquid radwaste system was no longer used and was cut and capped off. As such the liquid radwaste effluent monitor was not included in this EAL</p>
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<p style="text-align: right;">AG2</p> <p>Initiating Condition -- GENERAL EMERGENCY</p> <p>Spent fuel pool level cannot be restored to at least (site-specific Level 3 description) for 60 minutes or longer .</p> <p>Operating Mode Applicability: All</p> <p>Example Emergency Action Levels:</p> <p>NOTES: The Emergency Director should declare the General Emergency promptly upon determining that 60 minutes has been exceeded, or will likely be exceeded</p> <p>1. Spent fuel pool level cannot be restored to at least (site-specific Level 3 description) for 60 minutes or longer.</p>	<p style="text-align: right;">RG2</p>	<p> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) EAL not used in accordance with the discussion in Section 1.4, NRC Order EA-12-051, it is recommended that this EAL be implemented when the enhanced spent fuel pool level instrumentation is available for use. The completion of the enhanced SFP level indicators and need for the inclusion of this EAL is being tracked in accordance with Exelon Generation Company, LLC's Initial Status Report to March 12, 2012 Commission Order Modifying Licenses with Regard for Reliable Spent Fuel Pool Instrumentation (Order Number EA-12-051) dated October 25, 2012.</p>

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<p style="text-align: right;">AS2</p> <p>Initiating Condition – SITE AREA EMERGENCY</p> <p>Spent fuel pool level cannot be restored to at least (site-specific Level 3 description)</p> <p>Operating Mode Applicability: All</p> <p>Example Emergency Action Levels:</p> <ol style="list-style-type: none"> Spent fuel pool level cannot be restored to at least (site-specific Level 3 description) 	<p style="text-align: right;">RS2</p>	<p> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) EAL not used in accordance with the discussion in Section 1.4. NRC Order EA-12-051, it is recommended that this EAL be implemented when the enhanced spent fuel pool level instrumentation is available for use. The completion of the enhanced SFP level indicators and need for the inclusion of this EAL is being tracked in accordance with Exelon Generation Company, LLC's Initial Status Report to March 12, 2012 Commission Order Modifying Licenses with Regard for Reliable Spent Fuel Pool Instrumentation (Order Number EA-12-051) dated October 25, 2012.</p>

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<p style="text-align: right;">AA2</p> <p>Initiating Condition – ALERT</p> <p>Significant lowering of water level above, or damage to, irradiated fuel.</p> <p>Operating Mode Applicability: All</p> <p>Example Emergency Action Levels: (1 or 2 or 3)</p> <ol style="list-style-type: none"> Uncovery of irradiated fuel in the REFUELING PATHWAY. Damage to irradiated fuel resulting in a release of radioactivity from the fuel as indicated by ANY of the following radiation monitors: (site-specific listing of radiation monitors, and the associated readings, setpoints and/or alarms) Lowering of spent fuel pool level to (site-specific Level 2 value). 	<p style="text-align: right;">RA2</p> <p>Initiating Condition:</p> <p>Significant lowering of water level above, or damage to, irradiated fuel.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <ol style="list-style-type: none"> Uncovery of irradiated fuel in the REFUELING PATHWAY. <p style="text-align: center;">OR</p> <ol style="list-style-type: none"> Damage to irradiated fuel resulting in a release of radioactivity from the fuel as indicated by ANY Table R2 Radiation Monitor reading ≥ 1000 mRem/hr. <div style="border: 1px solid black; padding: 5px; margin: 10px auto; width: fit-content;"> <p style="text-align: center;">Table R2 Refuel Floor ARM's</p> <ul style="list-style-type: none"> • C-5, Crit Mon • C-10, North Wall • C-9, North Wall • B-9, Open Floor </div>	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site-specific monitors and Threshold values to ensure timely classification.</p> <p>2) EAL #3 not used in accordance with the discussion in Section 1.4. NRC Order EA-12-051, it is recommended that this EAL be implemented when the enhanced spent fuel pool level instrumentation is available for use. The completion of the enhanced SFP level indicators and need for the inclusion of this EAL is being tracked in accordance with Exelon Generation Company, LLC's Initial Status Report to March 12, 2012 Commission Order Modifying Licenses with Regard for Reliable Spent Fuel Pool Instrumentation (Order Number EA-12-051) dated October 25, 2012.</p>

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<p style="text-align: right;">AU2</p> <p>Initiating Condition: UNUSUAL EVENT UNPLANNED loss of water level above irradiated fuel Operating Mode Applicability: All</p> <p>Example Emergency Action Levels:</p> <p>1. a. UNPLANNED water level drop in the REFUELING PATHWAY as indicated by ANY of the following:</p> <p style="padding-left: 40px;">(site-specific level indications).</p> <p>AND</p> <p>b. UNPLANNED rise in area radiation levels as indicated by ANY of the following radiation monitors.</p> <p style="padding-left: 40px;">(site-specific list of area radiation monitors)</p>	<p style="text-align: right;">RU2</p> <p>Initiating Condition: UNPLANNED loss of water level above irradiated fuel Operating Mode Applicability: 1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>1. a. UNPLANNED water level drop in the REFUELING PATHWAY as indicated by ANY of the following:</p> <ul style="list-style-type: none"> • Refueling Cavity water level < 583 inches (GEMAC Wide Range, floodup calibration). <p style="text-align: center;">OR</p> <ul style="list-style-type: none"> • Indication or report of a drop in water level in the REFUELING PATHWAY. <p>AND</p> <p>b. UNPLANNED Area Radiation Monitor reading rise on ANY radiation monitors in Table R2.</p> <div style="border: 1px solid black; padding: 5px; margin: 10px auto; width: fit-content;"> <p style="text-align: center; margin: 0;">Table R2 Refuel Floor ARM's</p> <ul style="list-style-type: none"> • C-5, Crit Mon • C-10, North Wall • C-9, North Wall • B-9, Open Floor </div>	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>Listed site specific level indication and monitors to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification												
<p style="text-align: right;">AA3</p> <p>Initiating Condition – ALERT</p> <p>Radiation levels that impede access to equipment necessary for normal plant operations, cooldown or shutdown.</p> <p>Operating Mode Applicability: All</p> <p>Example Emergency Action Levels: (1 or 2)</p> <p>Note: If the equipment in the listed room or area was already inoperable, or out of service, before the event occurred, then no emergency classification is warranted</p> <ol style="list-style-type: none"> Dose rate greater than 15 mR/hr in ANY of the following areas: <ul style="list-style-type: none"> Control Room Central Alarm Station (other site-specific areas/rooms) An UNPLANNED event results in radiation levels that prevent or significantly impede access to any of the following plant rooms or areas: (site-specific list of plant rooms or areas with entry-related mode applicability identified) 	<p style="text-align: right;">RA3</p> <p>Initiating Condition:</p> <p>Radiation levels that impede access to equipment necessary for normal plant operations, cooldown or shutdown.</p> <p>Operating Mode Applicability: 1, 2, 3, 4,D</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> If the equipment in the room or area listed in Table R3 was already inoperable, or out of service, before the event occurred, then no emergency classification is warranted . <ol style="list-style-type: none"> Dose rate > 15 mR/hr in ANY of the following Table R3 areas: <table border="1" data-bbox="831 665 1278 786" style="margin-left: 40px;"> <thead> <tr> <th colspan="2" style="text-align: center;">Table R3 Areas Requiring Continuous Occupancy</th> </tr> </thead> <tbody> <tr> <td colspan="2"> <ul style="list-style-type: none"> Main Control Room Central Alarm Station – (by survey) </td> </tr> </tbody> </table> <p style="text-align: center;">OR</p> <ol style="list-style-type: none"> UNPLANNED event results in radiation levels that prohibit or significantly impede access to ANY of the following Table R4 plant rooms or areas: <table border="1" data-bbox="787 895 1325 1068" style="margin-left: 40px;"> <thead> <tr> <th colspan="2" style="text-align: center;">Table R4 Areas with Entry Related Mode Applicability</th> </tr> <tr> <th style="text-align: center;">Area</th> <th style="text-align: center;">Entry Related Mode Applicability</th> </tr> </thead> <tbody> <tr> <td style="text-align: center;">Reactor Building*</td> <td style="text-align: center;">Modes 3 and 4</td> </tr> <tr> <td colspan="2" style="text-align: center;">*Areas required to establish shutdown cooling</td> </tr> </tbody> </table> 	Table R3 Areas Requiring Continuous Occupancy		<ul style="list-style-type: none"> Main Control Room Central Alarm Station – (by survey) 		Table R4 Areas with Entry Related Mode Applicability		Area	Entry Related Mode Applicability	Reactor Building*	Modes 3 and 4	*Areas required to establish shutdown cooling		<p style="text-align: center;">Justification</p> <p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>Listed site specific plant rooms and areas with identified mode applicability to ensure timely classification.</p>
Table R3 Areas Requiring Continuous Occupancy														
<ul style="list-style-type: none"> Main Control Room Central Alarm Station – (by survey) 														
Table R4 Areas with Entry Related Mode Applicability														
Area	Entry Related Mode Applicability													
Reactor Building*	Modes 3 and 4													
*Areas required to establish shutdown cooling														

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">SU3</p> <p>Initiating Condition: UNUSUAL EVENT Reactor coolant activity greater than Technical Specification allowable limits. Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Example Emergency Action Levels:</p> <ol style="list-style-type: none"> 1. (Site-specific radiation monitor) reading greater than (site-specific value). OR 2. Sample analysis indicates that a reactor coolant activity value is greater than an allowable limit specified in Technical Specifications. 	<p style="text-align: right;">RU3</p> <p>Initiating Condition: Reactor coolant activity greater than Technical Specification allowable limits. Operating Mode Applicability: 1, 2 Emergency Action Level (EAL):</p> <ol style="list-style-type: none"> 1. Offgas system radiation monitor Hi-Hi alarm. OR 2. Specific coolant activity > 4.0 uCi/gm Dose equivalent I-131. 	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site-specific monitor and Threshold value to ensure timely classification.</p> <p>2) Listed this system category EAL in the radiological category EAL section to maintain consistency with current and previous revisions of Exelon EALs. This will ensure a timely classification since the threshold values are more aligned with the radiological category vice system category.</p>

Fission Product Barrier Matrix							Hot Matrix
GENERAL EMERGENCY			SITE AREA EMERGENCY		ALERT		
FG1 Loss of any two barriers AND Loss or Potential Loss of third barrier. 1,2			FS1 Loss or Potential Loss of ANY two barriers. 1,2		FA1 ANY Loss or ANY Potential Loss of either Fuel Clad or RCS 1,2		
Sub-Category	FC – Fuel Clad		RC – Reactor Coolant System		CT - Containment		
	Loss	Potential Loss	Loss	Potential Loss	Loss	Potential Loss	
1. RCS Activity / Primary Containment Pressure / Primary Containment Conditions	A. (Site specific indications that reactor coolant activity is greater than 300 uCi/gm dose equivalent I-131)	None	A. Primary containment pressure greater than (site-specific value) due to RCS leakage.	None	A. UNPLANNED rapid drop in primary containment pressure following primary containment pressure rise OR B. Primary containment pressure response not consistent with LOCA conditions.	A. Primary containment pressure greater than (site-specific value) OR B. (site-specific explosive mixture) exists inside primary containment OR C. HCTL exceeded.	
2. RPV Water Level	A. Primary containment flooding required.	A. RPV water level cannot be restored and maintained above (site-specific RPV water level corresponding to top of active fuel) or cannot be determined.	A. RPV water level cannot be restored and maintained above (site-specific RPV level corresponding to the top of active fuel) or cannot be determined.	None	None	A. Primary containment flooding required.	
3. RCS Leak Rate/ Primary Containment Isolation Failure	None	None	A. UNISOLABLE break in any of the following: (site-specific systems with potential for high-energy line breaks) OR B. Emergency RPV Depressurization	A. UNISOLABLE primary system leakage that results in exceeding EITHER of the following: 1. Max Normal Operating Temperature OR 2. Max Normal Operating Area Radiation Level.	A. UNISOLABLE direct downstream pathway to the environment exists after primary containment isolation signal OR B. Intentional primary containment venting per EOPs OR C. UNISOLABLE primary system leakage that results in exceeding EITHER of the following: 1. Max Safe Operating Temperature. OR 2. Max Safe Operating Area Radiation Level.	None	
4. Primary Containment Radiation	A. Primary Containment Radiation Monitor reading greater than (site-specific value).	None	A. Primary Containment Radiation Monitor reading greater than (site-specific value).	None	None	A. Primary Containment Radiation Monitor reading greater than (site-specific value).	
5. Emergency Director Judgment	A. Any Condition in the opinion of the Emergency Director that indicates Loss of the Fuel Clad Barrier.	A. Any Condition in the opinion of the Emergency Director that indicates Potential Loss of the Fuel Clad Barrier.	A. ANY Condition in the opinion of the Emergency Director that indicates Loss of the RCS Barrier.	A. Any Condition in the opinion of the Emergency Director that indicates Potential Loss of the RCS Barrier.	A. Any Condition in the opinion of the Emergency Director that indicates Loss of the Containment Barrier.	A. Any Condition in the opinion of the Emergency Director that indicates Potential Loss of the Containment Barrier.	

Proposed Fission Product Barrier Matrix
Fission Product Barrier Matrix

GENERAL EMERGENCY		SITE AREA EMERGENCY		ALERT		Hot Matrix
FG1 Loss of any two barriers AND Loss or Potential Loss of third barrier. 12		FS1 Loss or Potential Loss of ANY two barriers. 12		FA1 ANY Loss or ANY Potential Loss of either Fuel Clad or RCS 12		
Sub-Category	FC – Fuel Clad		RC – Reactor Coolant System		CT - Containment	
	Loss	Potential Loss	Loss	Potential Loss	Loss	Potential Loss
1. RCS Activity	Coolant activity > 300 uCi/gm Dose Equivalent I-131	None	None	None	None	None
2. RPV Water Level	1. Plant conditions indicate primary containment flooding is required.	2. RPV water level <u>cannot</u> be restored and maintained > 0 inches TAF OR 3. RPV water level <u>cannot</u> be determined.	1. RPV water level <u>cannot</u> be restored and maintained > 0 inches TAF OR 2. RPV water level <u>cannot</u> be determined.	None	None	Plant conditions indicate primary containment flooding is required.
3. Primary Containment Pressure/Conditions	None	None	1. Drywell pressure >3.0 psig. AND 2. Drywell pressure rise is due to RCS leakage	None	1. UNPLANNED rapid drop in Drywell pressure following Drywell pressure rise. OR 2. Drywell pressure response <u>not</u> consistent with LOCA conditions.	3. Drywell pressure > 44 psig and rising. OR 4. a. Drywell or torus hydrogen concentration ≥ 8%. AND b. Drywell or torus oxygen concentration ≥ 5%. OR 5. Heat Capacity Temperature Limit (EMG-3200.02 Fig. F) exceeded.
4. RCS Leak Rate	None	None	1. UNISOLABLE Main Steam Line (MSL), Isolation Condenser, Feedwater, or RWCU line break. OR 2. Emergency RPV Depressurization is required.	3. UNISOLABLE primary system leakage that results in EITHER of the following: a. Secondary Containment area temperature > EMG-3200.11 Max Normal (Table 11) operating level. OR b. Secondary Containment area radiation level > EMG-3200.11 Max Normal (Table 12) operating level.	None	None
5. Primary Containment Radiation	Containment Hi Range Radiation Monitoring System (CHRRMS) reading > 530 R/hr.	None	Containment Hi Range Radiation Monitoring System (CHRRMS) reading > 100R/hr.	None	None	Containment Hi Range Radiation Monitoring System (CHRRMS) reading > 1210 R/hr.
6. Primary Containment Isolation Failure	None	None	None	None	1. UNISOLABLE direct downstream pathway to the environment exists after primary containment isolation signal. OR 2. Intentional Primary Containment venting/purging per EOPs or SAMGs due to accident conditions. OR 3. UNISOLABLE primary system leakage that results in EITHER of the following: a. Secondary Containment area temperature > EMG-3200.11 Max Safe (Table 11) operating level. OR b. Secondary Containment area radiation level > EMG-3200.11 Max Safe (Table 12) operating level.	None
7. Emergency Director Judgment	A. Any Condition in the opinion of the Emergency Director that indicates Loss of the Fuel Clad Barrier.	A. Any Condition in the opinion of the Emergency Director that indicates Potential Loss of the Fuel Clad Barrier.	A. Any Condition in the opinion of the Emergency Director that indicates Loss of the RCS Barrier.	A. Any Condition in the opinion of the Emergency Director that indicates Potential Loss of the RCS Barrier.	A. Any Condition in the opinion of the Emergency Director that indicates Loss of the Containment Barrier.	A. Any Condition in the opinion of the Emergency Director that indicates Potential Loss of the Containment Barrier.

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">FC1</p> <p>Category: Fuel Clad Barrier RCS Activity Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Loss A. (Site specific indications that reactor coolant activity is greater than 300 uCi/gm dose equivalent I-131)</p>	<p style="text-align: right;">FC1</p> <p>Category: Fuel Clad Barrier RCS Activity Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: <u>Loss</u> Coolant activity > 300 uCi/gm Dose Equivalent I-131.</p>	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site-specific threshold value to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">FC2</p> <p>Category: Fuel Clad Barrier RPV Water Level Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Loss A. Primary containment flooding required. Potential Loss A. RPV water level cannot be restored and maintained above (site-specific RPV water level corresponding to top of active fuel) or cannot be determined.</p>	<p style="text-align: right;">FC2</p> <p>Category: Fuel Clad Barrier RCS Activity Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: <u>Loss</u> 1. Plant conditions indicate primary containment flooding is required. <u>Potential Loss</u> 2. RPV water level <u>cannot</u> be restored and maintained > 0 inches TAF OR 3. RPV water level <u>cannot</u> be determined.</p>	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site-specific threshold value to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">FC4</p> <p>Category: Fuel Clad Barrier Primary Containment Radiation Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Loss A. Primary Containment Radiation Monitor reading greater than (site-specific value).</p>	<p style="text-align: right;">FC5</p> <p>Category: Fuel Clad Barrier Primary Containment Radiation Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: <u>Loss</u> Containment Hi Range Radiation Monitoring System (CHRRMS) reading > 530 R/hr.</p>	<p> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site-specific monitor and threshold value to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">FC6</p> <p>Category: Fuel Clad Barrier Emergency Director Judgment</p> <p>Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown</p> <p>Fission Product Barrier Threshold: Loss</p> <p>A. Any Condition in the opinion of the Emergency Director that indicates Loss of the Fuel Clad Barrier.</p> <p>Potential Loss</p> <p>A. Any Condition in the opinion of the Emergency Director that indicates Potential Loss of the Fuel Clad Barrier.</p>	<p style="text-align: right;">FC7</p> <p>Category: Fuel Clad Barrier Emergency Director Judgment</p> <p>Operating Mode Applicability: 1, 2, 3</p> <p>Fission Product Barrier (FPB) Threshold: <u>Loss</u></p> <p>1. Any Condition in the opinion of the Emergency Director that indicates Loss of the Fuel Clad Barrier.</p> <p><u>Potential Loss</u></p> <p>2. Any Condition in the opinion of the Emergency Director that indicates Potential Loss of the Fuel Clad Barrier.</p>	<p><input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">RC1</p> <p>Category: Reactor Coolant System Barrier Primary Containment Pressure Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Loss A. Primary containment pressure greater than (site-specific value) due to RCS leakage.</p>	<p style="text-align: right;">RC3</p> <p>Category: Reactor Coolant System Barrier Primary Containment Pressure/Conditions Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: <u>Loss</u> 1. Drywell pressure > 3.0 psig. AND 2. Drywell pressure rise is due to RCS leakage</p>	<p> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site-specific threshold value to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">RC2</p> <p>Category: Reactor Coolant System Barrier RPV Water Level Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Loss A. RPV water level cannot be restored and maintained above (site-specific RPV level corresponding to the top of active fuel) or cannot be determined.</p>	<p style="text-align: right;">RC2</p> <p>Category: Reactor Coolant System Barrier RPV Water Level Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: <u>Loss</u> 1. RPV water level <u>cannot</u> be restored and maintained > 0 inches TAF OR 2. RPV water level <u>cannot</u> be determined.</p>	<p> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site-specific threshold value to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">RC3</p> <p>Category: Reactor Coolant System Barrier RCS Leak Rate Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Loss A. UNISOLABLE break in any of the following: (site-specific systems with potential for high-energy line breas) OR B. Emergency RPV Depressurization Potential Loss A. UNISOLABLE primary system leakage that results in exceeding EITHER of the following: 1. Max Normal Operating Temperature OR 2. Max Normal Operating Area Radiation Level.</p>	<p style="text-align: right;">RC4</p> <p>Category: Reactor Coolant System Barrier RCS Leak Rate Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: Loss 1. UNISOLABLE Main Steam Line (MSL), Isolation Condenser, Feedwater, or RWCU line break. OR 2. Emergency RPV Depressurization is required. <u>Potential Loss</u> 3. UNISOLABLE primary system leakage that results in EITHER of the following: a. Secondary Containment area temperature > EMG-3200.11 Max Normal (Table 11) operating level. OR b. Secondary Containment area radiation level > EMG-3200.11 Max Normal (Table 12) operating level.</p>	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site-specific systems and threshold values to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">RC4</p> <p>Category: Reactor Coolant System Barrier Primary Containment Radiation Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Loss A. Primary Containment Radiation Monitor reading greater than (site-specific value).</p>	<p style="text-align: right;">RC5</p> <p>Category: Reactor Coolant System Barrier Primary Containment Radiation Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: <u>Loss</u> Containment Hi Range Radiation Monitoring System (CHRRMS) reading > 1000 R/hr.</p>	<p> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site-specific monitor and threshold value to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">RC6</p> <p>Category: Reactor Coolant System Barrier Emergency Director Judgment Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Loss A. Any Condition in the opinion of the Emergency Director that indicates Loss of the RCS Barrier. Potential Loss A. Any Condition in the opinion of the Emergency Director that indicates Potential Loss of the RCS Barrier.</p>	<p style="text-align: right;">RC7</p> <p>Category: Reactor Coolant System Barrier Emergency director Judgment Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: Loss 1. ANY Condition in the opinion of the Emergency Director that indicates Loss of the RCS Barrier. <u>Potential Loss</u> 2. Any Condition in the opinion of the Emergency Director that indicates Potential Loss of the RCS Barrier.</p>	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">CT1</p> <p>Category: Containment Barrier Primary Containment Conditions Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Loss C. UNPLANNED rapid drop in primary containment pressure following primary containment pressure rise OR B. Primary containment pressure response not consistent with LOCA conditions. Potential Loss D. Primary containment pressure greater than (site-specific value) OR E. (site-specific explosive mixture) exists inside primary containment OR 3. HCTL exceeded.</p>	<p style="text-align: right;">CT3</p> <p>Category: Containment Barrier Primary Containment Pressure/Conditions Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: <u>Loss</u> 1. UNPLANNED rapid drop in Drywell pressure following Drywell pressure rise. OR 2. Drywell pressure response <u>not</u> consistent with LOCA conditions. <u>Potential Loss</u> 3. Drywell pressure > 44 psig and rising. OR 4. a. Drywell or torus hydrogen concentration ≥ 6%. AND b. Drywell or torus oxygen concentration ≥ 5%. OR 5. Heat Capacity Temperature Limit (EMG-3200.02 Fig. F) exceeded.</p>	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site-specific threshold values to ensure timely classification.</p> <p>2) The words "and rising" were added to account for the momentary spike in pressure where pressure is now lowering, the risk of a potential loss of containment is no longer present, this wording is also consistent with present EAL wording.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">CT2</p> <p>Category: Containment Barrier RPV Water Level Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Potential Loss A. Primary containment flooding required.</p>	<p style="text-align: right;">CT2</p> <p>Category: Containment Barrier RPV Water Level Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: <u>Potential Loss</u> Plant conditions indicate primary containment flooding is required.</p>	<p> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">CT3</p> <p>Category: Containment Barrier Primary Containment Isolation Failure Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Loss A. UNISOLABLE direct downstream pathway to the environment exists after primary containment isolation signal OR B. Intentional primary containment venting per EOPs OR C. UNISOLABLE primary system leakage that results in exceeding EITHER of the following: 1. Max Safe Operating Temperature. OR 2. Max Safe Operating Area Radiation Level.</p>	<p style="text-align: right;">CT6</p> <p>Category: Containment Barrier Primary Containment Isolation Failure Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: Loss 1. UNISOLABLE direct downstream pathway to the environment exists after primary containment isolation signal. OR 2. Intentional Primary Containment venting/purging per EOPs or SAMGs due to accident conditions. OR 3. UNISOLABLE primary system leakage that results in EITHER of the following: a. Secondary Containment area temperature > EMG-3200.11 Max Safe (Table 11) operating level. OR b. Secondary Containment area radiation level > EMG-3200.11 Max Safe (Table 12) operating level.</p>	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site-specific threshold values to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">CT4</p> <p>Category: Containment Barrier Primary Containment Radiation Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Potential Loss A. Primary Containment Radiation Monitor reading greater than (site-specific value).</p>	<p style="text-align: right;">CT5</p> <p>Category: Containment Barrier Primary Containment Radiation Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: <u>Potential Loss</u> Containment Hi Range Radiation Monitoring System (CHRRMS) reading > 1210 R/hr.</p>	<p> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site-specific monitor and threshold value to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">CT6</p> <p>Category: Containment Barrier Emergency director Judgment Operating Mode Applicability: Power Operation, Startup, Hot Standby, Hot Shutdown Fission Product Barrier Threshold: Loss A. Any Condition in the opinion of the Emergency Director that indicates Loss of the Containment Barrier. Potential Loss A. Any Condition in the opinion of the Emergency Director that indicates Potential Loss of the Containment Barrier.</p>	<p style="text-align: right;">CT7</p> <p>Category: Containment Barrier Emergency Director Judgment Operating Mode Applicability: 1, 2, 3 Fission Product Barrier (FPB) Threshold: <u>Loss</u> 1. Any Condition in the opinion of the Emergency Director that indicates Loss of the Containment Barrier. <u>Potential Loss</u> 2. Any Condition in the opinion of the Emergency Director that indicates Potential Loss of the Containment Barrier.</p>	<p> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">SG1</p> <p>Initiating Condition: GENERAL EMERGENCY</p> <p>Prolonged loss of all offsite and all onsite AC power to emergency buses.</p> <p>Operating Mode Applicability:</p> <p>Power Operation, Startup, Hot Standby, Hot Shutdown</p> <p>Example Emergency Action Levels:</p> <p>Note: The Emergency Director should declare the General Emergency promptly upon determining that (site-specific hours) has been exceeded, or will likely be exceeded.</p> <p>1. a. Loss of ALL offsite and ALL onsite AC power to (site-specific emergency buses).</p> <p style="padding-left: 40px;">AND</p> <p>b. EITHER of the following:</p> <ul style="list-style-type: none"> • Restoration of at least one emergency bus in less than (site-specific hours) is not likely. • (Site-specific indication of an inability to adequately remove heat from the core) 	<p style="text-align: right;">MG1</p> <p>Initiating Condition:</p> <p>Prolonged loss of all offsite and all onsite AC power to emergency buses.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> • The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <p>1 Loss of ALL offsite AC power to 4160V Buses 1C and 1D.</p> <p style="padding-left: 40px;">AND</p> <p>2. Failure of EDG-1 and EDG-2 Emergency Diesel Generators to supply power to 4160V Buses 1C and 1D.</p> <p style="padding-left: 40px;">AND</p> <p>3. EITHER of the following:</p> <ul style="list-style-type: none"> a. Restoration of at least one 4160V Bus (1C or 1D) in < 1 hour is not likely. <li style="padding-left: 40px;">OR b. RPV water level cannot be restored and maintained > -20 inches TAF. 	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>Listed site specific equipment, site specific time based on station blackout coping analysis, and site specific indication to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">SS1</p> <p>Initiating Condition: SITE AREA EMERGENCY</p> <p>Loss of all offsite and all onsite AC power to emergency buses for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>Power Operation, Startup, Hot Standby, Hot Shutdown</p> <p>Example Emergency Action Levels:</p> <p>Note: The Emergency Director should declare the Site Area Emergency promptly upon determining that 15 minutes time has been exceeded, or will likely be exceeded.</p> <p>Loss of ALL offsite and ALL onsite AC Power to (site-specific emergency buses) for 15 minutes or longer.</p>	<p style="text-align: right;">MS1</p> <p>Initiating Condition:</p> <p>Loss of all offsite and onsite AC power to emergency buses for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <p>1. Loss of ALL offsite AC Power to 4160V Buses 1C and 1D.</p> <p>AND</p> <p>2. Failure of EDG-1 and EDG-2 Emergency Diesel Generators to supply power to 4160V Buses 1C and 1D.</p> <p>AND</p> <p>3. Failure to restore power to at least one 4160V Bus (1C or 1D) in < 15 minutes from the time of loss of both offsite and onsite AC power.</p>	<p><input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation</p> <p>1) Listed site specific equipment to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">SA1</p> <p>Initiating Condition: ALERT</p> <p>Loss of all but one AC power source to emergency buses for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>Power Operation, Startup, Hot Standby, Hot Shutdown</p> <p>Example Emergency Action Levels:</p> <p>Note: The Emergency Director should declare the Alert promptly upon determining that 15 minutes time has been exceeded, or will likely be exceeded.</p> <p>1. a. AC power capability to (site-specific emergency buses) is reduced to a single power source for 15 minutes or longer.</p> <p>AND</p> <p>b. Any additional single power source failure will result in loss of all AC power to SAFETY SYSTEMS.</p>	<p style="text-align: right;">MA1</p> <p>Initiating Condition:</p> <p>Loss of all but one AC power source to emergency buses for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> • The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <p>1. AC power capability to 4160V Buses 1C and 1D reduced to only one of the following power sources for ≥ 15 minutes.</p> <ul style="list-style-type: none"> • Startup Transformer SA • Startup Transformer SB • EDG-1 Emergency Diesel Generator • EDG-2 Emergency Diesel Generator <p>AND</p> <p>2. ANY additional single power source failure will result in a loss of all AC power to SAFETY SYSTEMS.</p>	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific equipment to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">SU1</p> <p>Initiating Condition: UNUSUAL EVENT</p> <p>Loss of all offsite AC power capability to emergency buses for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>Power Operation, Startup, Hot Standby, Hot Shutdown</p> <p>Example Emergency Action Levels:</p> <p>Note: The Emergency Director should declare the Unusual Event promptly upon determining that 15 minutes has been exceeded, or will likely be exceeded.</p> <p>Loss of ALL offsite AC power capability to (site-specific emergency buses) for 15 minutes or longer</p>	<p style="text-align: right;">MU1</p> <p>Initiating Condition:</p> <p>Loss of all offsite AC power capability to emergency buses for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <p>Loss of ALL offsite AC power capability to 4160V Buses 1C and 1D for ≥15 minutes.</p>	<p> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific equipment to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">SG8</p> <p>Initiating Condition: GENERAL EMERGENCY</p> <p>Loss of all AC and Vital DC power sources for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>Power Operation, Startup, Hot Standby, Hot Shutdown</p> <p>Example Emergency Action Levels:</p> <p>Note: The Emergency Director should declare the General Emergency promptly upon determining that 15 minutes has been exceeded, or will likely be exceeded.</p> <p>1. Loss of ALL offsite and ALL onsite AC power to (site-specific emergency buses) for 15 minutes or longer.</p> <p>AND</p> <p>Indicated voltage is less than (site-specific bus voltage value) on ALL (site-specific vital DC buses) for 15 minutes or longer.</p>	<p style="text-align: right;">MG2</p> <p>Initiating Condition:</p> <p>Loss of all AC and Vital DC power sources for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> • The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <p>1. Loss of ALL offsite AC power to 4160V Buses 1C and 1D. AND</p> <p>2. Failure of EDG-1 and EDG-2 Emergency Diesel Generators to supply power to 4160V Buses 1C and 1D.. AND</p> <p>3. Voltage is < 115 VDC on 125 VDC battery busses B and C. AND</p> <p>4. ALL AC and Vital DC power sources have been lost for ≥ 15 minutes.</p>	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific equipment to ensure timely classification.</p> <p>2) Removed the word "indicated" this will allow for an indication problem to not cause confusion on the need to declare.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">SS8</p> <p>Initiating Condition: SITE AREA EMERGENCY</p> <p>Loss of all Vital DC power for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>Power Operation, Startup, Hot Standby, Hot Shutdown</p> <p>Example Emergency Action Levels:</p> <p>Note: The Emergency Director should declare the Site Area Emergency promptly upon determining that 15 minutes time has been exceeded, or will likely be exceeded.</p> <p>Indicated voltage is less than (site-specific bus voltage value) on ALL Vital DC buses for 15 minutes or longer.</p>	<p style="text-align: right;">MS2</p> <p>Initiating Condition:</p> <p>Loss of all Vital DC power for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <p>Voltage is < 115 VDC on 125 VDC battery busses B and C for ≥ 15 minutes.</p>	<p> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific equipment and site specific value to ensure timely classification.</p> <p>2) Removed the word "indicated" this will allow for an indication problem to not cause confusion on the need to declare.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">SS5</p> <p>Initiating Condition: SITE AREA EMERGENCY</p> <p>Inability to shutdown the reactor causing a challenge to (core cooling [PWR] / RPV water level [BWR]) or RCS heat removal.</p> <p>Operating Mode Applicability:</p> <p>Power Operation</p> <p>Example Emergency Action Levels:</p> <p>1. a. An automatic (trip [PWR] / scram [BWR]) did not shutdown the reactor.</p> <p style="padding-left: 40px;">AND</p> <p>b. All manual actions to shutdown the reactor have been unsuccessful.</p> <p style="padding-left: 40px;">AND</p> <p>c. EITHER of the following conditions exist:</p> <p style="padding-left: 80px;">1. (Site-specific indication of an inability to adequately remove heat from the core)</p> <p style="padding-left: 80px;">OR</p> <p style="padding-left: 80px;">2. (Site-specific indication of an inability to adequately remove heat from the RCS)</p>	<p style="text-align: right;">MS3</p> <p>Initiating Condition:</p> <p>Inability to shutdown the reactor causing a challenge to RPV water level or RCS heat removal.</p> <p>Operating Mode Applicability:</p> <p>1,2</p> <p>Emergency Action Level (EAL):</p> <p>1. Automatic scram did <u>not</u> shutdown the reactor as indicated by Reactor Power > 2%.</p> <p style="padding-left: 40px;">AND</p> <p>2. ALL manual / ARI actions to shutdown the reactor have been unsuccessful as indicated by Reactor Power > 2%.</p> <p style="padding-left: 40px;">AND</p> <p>3. EITHER of the following conditions exist:</p> <ul style="list-style-type: none"> • RPV water level <u>cannot</u> be restored and maintained > -20 inches TAF. <li style="padding-left: 120px;">OR • Heat Capacity Temperature Limit (EMG-3200.02 Fig. F) exceeded. 	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific indications to ensure timely classification.</p> <p>2) Mode 2 included in operating mode applicability as per developer notes.</p> <p>3) Added ARI as an equivalent scram</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p>Initiating Condition: ALERT SA5</p> <p>Automatic or manual (trip [PWR] / scram [BWR]) fails to shutdown the reactor, and subsequent manual actions taken at the reactor control consoles are not successful in shutting down the reactor.</p> <p>Operating Mode Applicability:</p> <p>Power Operation</p> <p>Example Emergency Action Levels:</p> <p>Note: A manual action is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does not include manually driving in control rods or implementation of boron injection strategies.</p> <p>1. a. An automatic (trip [PWR] / scram [BWR]) did not shutdown the reactor.</p> <p style="text-align: center;">AND</p> <p>b. Manual action taken at the reactor control consoles are not successful in shutting down the reactor.</p>	<p>Initiating Condition: MA3</p> <p>Automatic or manual scram fails to shutdown the reactor, and subsequent manual actions taken at the reactor control consoles are not successful in shutting down the reactor.</p> <p>Operating Mode Applicability:</p> <p>1,2</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> A manual action is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does not include manually driving in control rods or implementation of boron injection strategies. <p>1. Automatic or manual scram did <u>not</u> shutdown the reactor as indicated by Reactor Power > 2%.</p> <p style="text-align: center;">AND</p> <p>2. Manual / ARI actions taken at the Reactor Console are <u>not</u> successful in shutting down the reactor as indicated by Reactor Power > 2%.</p>	<p><input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation</p> <p>1) Listed site specific indications to ensure timely classification.</p> <p>2) Mode 2 included in operating mode applicability as per developer notes.</p> <p>3) Added ARI as an equivalent scram</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">SU5</p> <p>Initiating Condition: UNUSUAL EVENT</p> <p>Automatic or manual (trip [PWR] / scram [BWR]) fails to shutdown the reactor.</p> <p>Operating Mode Applicability:</p> <p>Power Operation</p> <p>Example Emergency Action Levels: (1 or 2)</p> <p>Note: A manual action is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does not include manually driving in control rods or implementation of boron injection strategies.</p> <p>1. a. An automatic (trip [PWR] / scram [BWR]) did not shutdown the reactor.</p> <p style="text-align: center;">AND</p> <p>b. A subsequent manual action taken at the reactor control consoles is successful in shutting down the reactor.</p> <p>2. a. A manual scram ([PWR] / scram [BWR]) did not shutdown the reactor.</p> <p style="text-align: center;">AND</p> <p>b. EITHER of the following:</p> <p>1. A subsequent manual action taken at the reactor control consoles is successful in shutting down the reactor.</p> <p style="text-align: center;">OR</p> <p>2. A subsequent automatic (trip [PWR] / scram [BWR]) is successful in shutting down the reactor.</p>	<p style="text-align: right;">MU3</p> <p>Initiating Condition:</p> <p>Automatic or manual scram fails to shutdown the reactor.</p> <p>Operating Mode Applicability:</p> <p>1,2</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> • A manual action is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does not include manually driving in control rods or implementation of boron injection strategies. <p>1. a. Automatic scram did not shutdown the reactor as indicated by Reactor Power > 2%.</p> <p style="text-align: center;">AND</p> <p>b. Subsequent manual / ARI action taken at the Reactor Console is successful in shutting down the reactor.</p> <p style="text-align: center;">OR</p> <p>2. a. Manual scram did not shutdown the reactor as indicated by Reactor Power > 2%.</p> <p style="text-align: center;">AND</p> <p>b. EITHER of the following:</p> <p>1. Subsequent manual / ARI action taken at the Reactor Console is successful in shutting down the reactor.</p> <p style="text-align: center;">OR</p> <p>2. Subsequent automatic scram / ARI is successful in shutting down the reactor.</p>	<p><input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation</p> <p>1) Listed site specific indications to ensure timely classification.</p> <p>2) Mode 2 included in operating mode applicability as per developer notes.</p> <p>3) Added ARI as an equivalent scram</p>

NEI 99-01 Rev 6	Proposed EAL	Justification																		
<p style="text-align: right;">SA2</p> <p>Initiating Condition: ALERT</p> <p>UNPLANNED loss of Control Room indications for 15 minutes or longer with a significant transient in progress.</p> <p>Operating Mode Applicability:</p> <p>Power Operation, Startup, Hot Standby, Hot Shutdown</p> <p>Example Emergency Action Levels:</p> <p>Note: The Emergency Director should declare the Alert promptly upon determining that 15 minutes has been exceeded, or will likely be exceeded.</p> <p>1. a. An UNPLANNED event results in the inability to monitor one or more of the following parameters from within the Control Room for 15 minutes or longer. [see table below]</p> <table border="1" data-bbox="126 702 703 1037"> <thead> <tr> <th>[BWR parameter list]</th> <th>[PWR parameter list]</th> </tr> </thead> <tbody> <tr> <td>Reactor Power</td> <td>Reactor Power</td> </tr> <tr> <td>RPV Level</td> <td>RCS Level</td> </tr> <tr> <td>RPV Pressure</td> <td>RCS Pressure</td> </tr> <tr> <td>Primary Containment Pressure</td> <td>In Core/Core Exit Temperature</td> </tr> <tr> <td>Suppression Pool Level</td> <td>Levels in at least (site specific number) steam generators</td> </tr> <tr> <td>Suppression Pool Temperature</td> <td>Steam Generator Auxiliary or Emergency Feed Water Flow</td> </tr> </tbody> </table> <p>AND</p> <p>b. Any of the following transient events in progress.</p> <ul style="list-style-type: none"> • Automatic or Manual runback greater than 25% thermal reactor power • Electrical load rejection greater than 25% full electrical load • Reactor Scram [BWR] / trip [PWR] • ECCS (SI) actuation • Thermal power oscillations greater than 10% [BWR] 	[BWR parameter list]	[PWR parameter list]	Reactor Power	Reactor Power	RPV Level	RCS Level	RPV Pressure	RCS Pressure	Primary Containment Pressure	In Core/Core Exit Temperature	Suppression Pool Level	Levels in at least (site specific number) steam generators	Suppression Pool Temperature	Steam Generator Auxiliary or Emergency Feed Water Flow	<p style="text-align: right;">MA4</p> <p>Initiating Condition:</p> <p>UNPLANNED loss of Control Room indications for 15 minutes or longer with a significant transient in progress.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> • The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <p>1. UNPLANNED event results in the inability to monitor ANY Table M1 parameters from within the Control Room for ≥ 15 minutes.</p> <table border="1" data-bbox="850 718 1260 925"> <thead> <tr> <th>Table M1 Control Room Parameters</th> </tr> </thead> <tbody> <tr> <td> <ul style="list-style-type: none"> • Reactor Power • RPV Water Level • RPV Pressure • Drywell Pressure • Torus Water Level • Torus Water Temperature </td> </tr> </tbody> </table> <p>AND</p> <p>2. ANY Table M2 transient in progress.</p> <table border="1" data-bbox="745 999 1360 1230"> <thead> <tr> <th>Table M2 Significant Transients</th> </tr> </thead> <tbody> <tr> <td> <ul style="list-style-type: none"> • Turbine Trip • Reactor Scram • ECCS Actuation • Thermal power change > 25% • Thermal Power oscillations > 10% </td> </tr> </tbody> </table>	Table M1 Control Room Parameters	<ul style="list-style-type: none"> • Reactor Power • RPV Water Level • RPV Pressure • Drywell Pressure • Torus Water Level • Torus Water Temperature 	Table M2 Significant Transients	<ul style="list-style-type: none"> • Turbine Trip • Reactor Scram • ECCS Actuation • Thermal power change > 25% • Thermal Power oscillations > 10% 	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p>
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<p style="text-align: right;">SU2</p> <p>Initiating Condition: UNUSUAL EVENT</p> <p>UNPLANNED loss of Control Room indications for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>Power Operation, Startup, Hot Standby, Hot Shutdown</p> <p>Example Emergency Action Levels:</p> <p>Note: The Emergency Director should declare the Unusual Event promptly upon determining that 15 minutes has been exceeded, or will likely be exceeded.</p> <p>An UNPLANNED event results in the inability to monitor one or more of the following parameters from within the Control Room for 15 minutes or longer. [see table below]</p> <table border="1" data-bbox="121 662 701 996"> <thead> <tr> <th>[BWR parameter list]</th> <th>[PWR parameter list]</th> </tr> </thead> <tbody> <tr> <td>Reactor Power</td> <td>Reactor Power</td> </tr> <tr> <td>RPV Level</td> <td>RCS Level</td> </tr> <tr> <td>RPV Pressure</td> <td>RCS Pressure</td> </tr> <tr> <td>Primary Containment Pressure</td> <td>In Core/Core Exit Temperature</td> </tr> <tr> <td>Suppression Pool Level</td> <td>Levels in at least (site specific number) steam generators</td> </tr> <tr> <td>Suppression Pool Temperature</td> <td>Steam Generator Auxiliary or Emergency Feed Water Flow</td> </tr> </tbody> </table>	[BWR parameter list]	[PWR parameter list]	Reactor Power	Reactor Power	RPV Level	RCS Level	RPV Pressure	RCS Pressure	Primary Containment Pressure	In Core/Core Exit Temperature	Suppression Pool Level	Levels in at least (site specific number) steam generators	Suppression Pool Temperature	Steam Generator Auxiliary or Emergency Feed Water Flow	<p style="text-align: right;">MU4</p> <p>Initiating Condition:</p> <p>UNPLANNED loss of Control Room indications for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <p>UNPLANNED event results in the inability to monitor ANY Table M1 parameters from within the Control Room for <u>≥ 15 minutes</u>.</p> <table border="1" data-bbox="800 645 1312 850"> <thead> <tr> <th>Table M1 Control Room Parameters</th> </tr> </thead> <tbody> <tr> <td> <ul style="list-style-type: none"> Reactor Power RPV Water Level RPV Pressure Drywell Pressure Torus Water Level Torus Water Temperature </td> </tr> </tbody> </table>	Table M1 Control Room Parameters	<ul style="list-style-type: none"> Reactor Power RPV Water Level RPV Pressure Drywell Pressure Torus Water Level Torus Water Temperature 	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p>
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NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">SA9</p> <p>Initiating Condition: ALERT</p> <p>Hazardous event affecting a SAFETY SYSTEM needed for the current operating mode.</p> <p>Operating Mode Applicability:</p> <p>Power Operation, Startup, Hot Standby, Hot Shutdown</p> <p>Example Emergency Action Levels:</p> <p>1. a. The occurrence of ANY of the following hazardous events:</p> <ul style="list-style-type: none"> • Seismic event (earthquake) • Internal or external flooding event • High winds or tornado strike • FIRE • EXPLOSION • (site-specific hazards) • Other events with similar hazard characteristics as determined by the Shift Manager <p style="text-align: center;">AND</p> <p>b. EITHER of the following:</p> <p>1. Event damage has caused indications of degraded performance in at least one train of a SAFETY SYSTEM needed for the current operating mode.</p> <p style="text-align: center;">OR</p> <p>2. The event has caused VISIBLE DAMAGE to a SAFETY SYSTEM component or structure needed for the current operating mode.</p>	<p style="text-align: right;">MA5</p> <p>Initiating Condition: ALERT</p> <p>Hazardous event affecting a SAFETY SYSTEM required for the current operating mode.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3</p> <p>Emergency Action Level (EAL):</p> <p>1. The occurrence of ANY of the following hazardous events:</p> <ul style="list-style-type: none"> • Seismic event (earthquake) • Internal or external flooding event • High winds or tornado strike • FIRE • EXPLOSION • Other events with similar hazard characteristics as determined by the Shift Manager <p style="text-align: center;">AND</p> <p>2. EITHER of the following:</p> <p>a. Event damage has caused indications of degraded performance in at least one train of a SAFETY SYSTEM required by Technical Specifications for the current operating mode.</p> <p style="text-align: center;">OR</p> <p>b. The event has caused VISIBLE DAMAGE to a SAFETY SYSTEM component or structure required by Technical Specifications for the current operating mode.</p> <p style="text-align: center;">OR</p> <p>c. A seismic event required a manual reactor scram per ABN-38, Station Seismic Event.</p>	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) No additional site specific hazards noted</p> <p>2) Changed the word "needed" to "required" in the IC and "required by Technical Specification" in the EAL to be consistent with terminology used by operators and minimize confusion.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">SU4</p> <p>Initiating Condition: UNUSUAL EVENT</p> <p>RCS leakage for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>Power Operation, Startup, Hot Standby, Hot Shutdown</p> <p>Example Emergency Action Levels: (1 or 2 or 3)</p> <p>Note: The Emergency Director should declare the Unusual Event promptly upon determining that 15 minutes has been exceeded, or will likely be exceeded.</p> <ol style="list-style-type: none"> 1. RCS unidentified or pressure boundary leakage greater than (site-specific value) for 15 minutes or longer. 2. RCS identified leakage greater than (site-specific value) for 15 minutes or longer 3. Leakage from the RCS to a location outside containment greater than 25 gpm for 15 minutes or longer 	<p style="text-align: right;">MU6</p> <p>Initiating Condition:</p> <p>RCS leakage for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> • The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <ol style="list-style-type: none"> 1. RCS unidentified or pressure boundary leakage in the Drywell > 10 gpm for ≥ 15 minutes. OR 2. RCS identified leakage in the Drywell >25 gpm for ≥ 15 minutes. OR 3. Leakage from the RCS to a location outside the Drywell >25 gpm for ≥ 15 minutes. 	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific values to ensure timely classification.</p> <p>2) in EAL #3 Changed wording from containment to Drywell for clarity to better define the primary containment structure.</p> <p>3) In EAL #1 and 2 added into the Drywell to differentiate between EAL #1/2 and #3. Without this wording would have been in EAL #1 or #2 concurrent with #3. With the added wording each EAL can be called separately.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification																																				
<p style="text-align: right;">SU6</p> <p>Initiating Condition: UNUSUAL EVENT</p> <p>Loss of all onsite or offsite communications capabilities</p> <p>Operating Mode Applicability:</p> <p>Power Operation, Startup, Hot Standby, Hot Shutdown</p> <p>Example Emergency Action Levels: (1 or 2 or 3)</p> <ol style="list-style-type: none"> Loss of ALL of the following onsite communication methods: (site-specific list of communications methods) Loss of ALL of the following ORO communication methods: (site-specific list of communications methods) Loss of ALL of the following NRC communication methods: (site-specific list of communications methods) 	<p style="text-align: right;">MU7</p> <p>Initiating Condition:</p> <p>Loss of all onsite or offsite communication capabilities.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3</p> <p>Emergency Action Level (EAL):</p> <ol style="list-style-type: none"> Loss of all Table M3 Onsite communications capability affecting the ability to perform routine operations. <p>OR</p> <ol style="list-style-type: none"> Loss of all Table M3 Offsite communication capability affecting the ability to perform offsite notifications. <p>OR</p> <ol style="list-style-type: none"> Loss of all Table M3 NRC communication capability affecting the ability to perform NRC notifications. <table border="1" data-bbox="762 761 1348 992" style="margin-left: auto; margin-right: auto;"> <thead> <tr> <th colspan="4" style="text-align: center;">Table M3 Communications Capability</th> </tr> <tr> <th style="text-align: center;">System</th> <th style="text-align: center;">Onsite</th> <th style="text-align: center;">Offsite</th> <th style="text-align: center;">NRC</th> </tr> </thead> <tbody> <tr> <td>Plant Paging System</td> <td style="text-align: center;">X</td> <td></td> <td></td> </tr> <tr> <td>Station Radio</td> <td style="text-align: center;">X</td> <td></td> <td></td> </tr> <tr> <td>Conventional Telephone lines</td> <td style="text-align: center;">X</td> <td style="text-align: center;">X</td> <td style="text-align: center;">X</td> </tr> <tr> <td>Satellite Phone system</td> <td style="text-align: center;">X</td> <td style="text-align: center;">X</td> <td style="text-align: center;">X</td> </tr> <tr> <td>NARS</td> <td></td> <td style="text-align: center;">X</td> <td></td> </tr> <tr> <td>HPN</td> <td></td> <td style="text-align: center;">X</td> <td style="text-align: center;">X</td> </tr> <tr> <td>ENS</td> <td></td> <td style="text-align: center;">X</td> <td style="text-align: center;">X</td> </tr> </tbody> </table>	Table M3 Communications Capability				System	Onsite	Offsite	NRC	Plant Paging System	X			Station Radio	X			Conventional Telephone lines	X	X	X	Satellite Phone system	X	X	X	NARS		X		HPN		X	X	ENS		X	X	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific communication methods to ensure timely classification.</p>
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NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">CA2</p> <p>Initiating Condition: ALERT</p> <p>Loss of all offsite and all onsite AC power to emergency buses for 15 minutes or longer.</p> <p>Operating Mode Applicability: Cold Shutdown, Refueling, Defueled</p> <p>Example Emergency Action Levels:</p> <p>Note: The Emergency Director should declare the Alert promptly upon determining that 15 minutes time has been exceeded, or will likely be exceeded.</p> <p>Loss of ALL offsite and ALL onsite AC Power to (site-specific emergency buses) for 15 minutes or longer.</p>	<p style="text-align: right;">CA1</p> <p>Initiating Condition:</p> <p>Loss of all offsite and onsite AC power to emergency buses for 15 minutes or longer.</p> <p>Operating Mode Applicability: 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <ol style="list-style-type: none"> Loss of all offsite AC power to 4160V Buses 1C and 1D. AND Failure of EDG-1 and EDG-2 Emergency Diesel Generators to supply power to 4160V Buses 1C and 1D.. AND Failure to restore power to at least one 4160V bus (1C or 1D) in < 15 minutes from the time of loss of both offsite and onsite AC power. 	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific equipment to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">CU2</p> <p>Initiating Condition: UNUSUAL EVENT</p> <p>Loss of all but one AC power source to emergency buses for 15 minutes or longer.</p> <p>Operating Mode Applicability: Cold Shutdown, Refueling, Defueled</p> <p>Example Emergency Action Levels:</p> <p>Note: The Emergency Director should declare the Unusual Event promptly upon determining that 15 minutes time has been exceeded, or will likely be exceeded.</p> <p>1. a. AC power capability to (site-specific emergency buses) is reduced to a single power source for 15 minutes or longer.</p> <p>AND</p> <p>b. Any additional single power source failure will result in loss of all AC power to SAFETY SYSTEMS.</p>	<p style="text-align: right;">CU1</p> <p>Initiating Condition:</p> <p>Loss of all but one AC power source to emergency buses for 15 minutes or longer.</p> <p>Operating Mode Applicability: 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> • The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <ul style="list-style-type: none"> 1. AC power capability to 4160V Buses 1C and 1D reduced to only one of the following power sources for ≥ 15 minutes. <ul style="list-style-type: none"> • Startup Transformer SA • Startup Transformer SB • EDG-1 Emergency Diesel Generator • EDG-2 Emergency Diesel Generator <p>AND</p> <p>2. ANY additional single power source failure will result in a loss of all AC power to SAFETY SYSTEMS.</p>	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific equipment to ensure timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">CA6</p> <p>Initiating Condition – ALERT Hazardous event affecting SAFETY SYSTEM needed for the current operating mode.</p> <p>Operating Mode Applicability: Cold Shutdown, Refueling</p> <p>Example Emergency Action Levels:</p> <p>1. a. The occurrence of ANY of the following hazardous events:</p> <ul style="list-style-type: none"> ● Seismic event (earthquake) ● Internal or external flooding event ● High winds or tornado strike ● FIRE ● EXPLOSION ● (site-specific hazards) ● Other events with similar hazard characteristics as determined by the Shift Manager <p>AND</p> <p>b. EITHER of the following:</p> <p>1. Event damage has caused indications of degraded performance in at least one train of a SAFETY SYSTEM needed for the current operating mode.</p> <p>OR</p> <p>2. The event has caused VISIBLE DAMAGE to a SAFETY SYSTEM component or structure needed for the current operating mode.</p>	<p style="text-align: right;">CA2</p> <p>Initiating Condition: Hazardous event affecting SAFETY SYSTEM required for the current operating mode.</p> <p>Operating Mode Applicability: 3, 4</p> <p>Emergency Action Level (EAL):</p> <p>1. The occurrence of ANY of the following hazardous events:</p> <ul style="list-style-type: none"> ● Seismic event (earthquake) ● Internal or external flooding event ● High winds or tornado strike ● FIRE ● EXPLOSION ● Other events with similar hazard characteristics as determined by the Shift Manager <p>AND</p> <p>2. EITHER of the following:</p> <p>a. Event damage has caused indications of degraded performance in at least one train of a SAFETY SYSTEM required by Technical Specifications for the current operating mode.</p> <p>OR</p> <p>b. The event has caused VISIBLE DAMAGE to a SAFETY SYSTEM component or structure required by Technical Specifications for the current operating mode.</p> <p>OR</p> <p>c. A seismic event required a manual reactor scram per ABN-38, Station Seismic Event.</p>	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) No additional site specific hazards noted</p> <p>2) Changed the word "needed" to "required" in the IC and "required by Technical Specification" in the EAL to be consistent with terminology used by operators and minimize confusion.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">CU4</p> <p>Initiating Condition: UNUSUAL EVENT</p> <p>Loss of Vital DC power for 15 minutes or longer.</p> <p>Operating Mode Applicability: Cold Shutdown, Refueling</p> <p>Example Emergency Action Levels:</p> <p>Note: The Emergency Director should declare the Unusual Event promptly upon determining that 15 minutes time has been exceeded, or will likely be exceeded.</p> <p>Indicated voltage is less than (site-specific bus voltage value) on required Vital DC buses for 15 minutes or longer.</p>	<p style="text-align: right;">CU3</p> <p>Initiating Condition:</p> <p>Loss of Vital DC power for 15 minutes or longer.</p> <p>Operating Mode Applicability: 3, 4</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <p>Voltage is < 115 VDC on required 125 VDC battery busses B and C for ≥ 15 minutes.</p>	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific voltage and equipment to ensure timely classification.</p> <p>2) Removed the word "indicated" this will allow for an indication problem to not cause confusion on the need to declare.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification																																				
<p style="text-align: right;">CU5</p> <p>Initiating Condition: UNUSUAL EVENT Loss of all onsite or offsite communications capabilities Operating Mode Applicability: Cold Shutdown, Refueling, Defuled Example Emergency Action Levels: (1 or 2 or 3)</p> <ol style="list-style-type: none"> Loss of ALL of the following onsite communication methods: (site-specific list of communications method) Loss of ALL of the following ORO communications s) methods: (site-specific list of communications methods) Loss of ALL of the following NRC communications methods: (site-specific list of communications methods) 	<p style="text-align: right;">CU4</p> <p>Initiating Condition: Loss of all onsite or offsite communication capabilities. Operating Mode Applicability: 3, 4, D Emergency Action Level (EAL):</p> <ol style="list-style-type: none"> Loss of all Table C1 Onsite communications capability affecting the ability to perform routine operations. OR Loss of all Table C1 Offsite communication capability affecting the ability to perform offsite notifications. OR Loss of all Table C1 NRC communication capability affecting the ability to perform NRC notifications. <table border="1" data-bbox="772 640 1346 872"> <thead> <tr> <th colspan="4">Table C1 Communications Capability</th> </tr> <tr> <th>System</th> <th>Onsite</th> <th>Offsite</th> <th>NRC</th> </tr> </thead> <tbody> <tr> <td>Plant Paging System</td> <td>X</td> <td></td> <td></td> </tr> <tr> <td>Station Radio</td> <td>X</td> <td></td> <td></td> </tr> <tr> <td>Conventional Telephone lines</td> <td>X</td> <td>X</td> <td>X</td> </tr> <tr> <td>Satellite Phone system</td> <td>X</td> <td>X</td> <td>X</td> </tr> <tr> <td>NARS</td> <td></td> <td>X</td> <td></td> </tr> <tr> <td>HPN</td> <td></td> <td>X</td> <td>X</td> </tr> <tr> <td>ENS</td> <td></td> <td>X</td> <td>X</td> </tr> </tbody> </table>	Table C1 Communications Capability				System	Onsite	Offsite	NRC	Plant Paging System	X			Station Radio	X			Conventional Telephone lines	X	X	X	Satellite Phone system	X	X	X	NARS		X		HPN		X	X	ENS		X	X	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific communications methods to ensure timely classification</p>
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NEI 99-01 Rev 6	Proposed EAL	Justification																						
<p style="text-align: right;">CA3</p> <p>Initiating Condition: ALERT Inability to maintain the plant in cold shutdown.</p> <p>Operating Mode Applicability: Cold Shutdown, Refueling</p> <p>Example Emergency Action Levels: (1 or 2)</p> <p>Note: The Emergency Director should declare the Alert promptly upon determining that the applicable has been exceeded, or will likely be exceeded.</p> <ol style="list-style-type: none"> UNPLANNED increase in RCS temperature to greater than (site-specific Technical Specification cold shutdown temperature limit) for greater than the duration specified in the following table. UNPLANNED RCS pressure increase greater than (site-specific pressure reading). (This EAL does not apply during water-solid plant conditions. [PWR]) <table border="1" data-bbox="132 612 703 860"> <caption>Table: RCS Heat-up Duration Thresholds</caption> <thead> <tr> <th>RCS Status</th> <th>Containment Closure Status</th> <th>Heat-up Duration</th> </tr> </thead> <tbody> <tr> <td>Intact (but not RCS Reduced Inventory [PWR])</td> <td>Not Applicable</td> <td>60 minutes*</td> </tr> <tr> <td rowspan="2">Not Intact (or at reduced inventory [PWR])</td> <td>Established</td> <td>20 minutes*</td> </tr> <tr> <td>Not Established</td> <td>0 minutes</td> </tr> </tbody> </table> <p>* If an RCS heat removal system is in operation within this time frame and RCS temperature is being reduced, the EAL is not applicable.</p>	RCS Status	Containment Closure Status	Heat-up Duration	Intact (but not RCS Reduced Inventory [PWR])	Not Applicable	60 minutes*	Not Intact (or at reduced inventory [PWR])	Established	20 minutes*	Not Established	0 minutes	<p style="text-align: right;">CA5</p> <p>Initiating Condition: Inability to maintain plant in cold shutdown.</p> <p>Operating Mode Applicability: 3, 4</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <ol style="list-style-type: none"> UNPLANNED rise in RCS temperature > 212°F due to loss of decay heat removal for > Table C2 duration. <table border="1" data-bbox="781 583 1306 847"> <caption>Table C2 RCS Heat-up Duration Thresholds</caption> <thead> <tr> <th>RCS Status</th> <th>Containment Closure Status</th> <th>Heat-up Duration</th> </tr> </thead> <tbody> <tr> <td>Intact</td> <td>Not Applicable</td> <td>60 minutes*</td> </tr> <tr> <td rowspan="2">Not Intact</td> <td>Established</td> <td>20 minutes*</td> </tr> <tr> <td>Not Established</td> <td>0 minutes</td> </tr> </tbody> </table> <p>* If an RCS heat removal system is in operation within this time frame and RCS temperature is being reduced, then EAL #1 is not applicable.</p> <p>OR</p> <ol style="list-style-type: none"> UNPLANNED RPV pressure rise > 10 psig as a result of temperature rise due to loss of decay heat removal. 	RCS Status	Containment Closure Status	Heat-up Duration	Intact	Not Applicable	60 minutes*	Not Intact	Established	20 minutes*	Not Established	0 minutes	<p><input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation</p> <p>1) Listed site specific Technical Specification cold shutdown temperature limit and site-specific pressure reading to ensure timely classification.</p>
RCS Status	Containment Closure Status	Heat-up Duration																						
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NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">CU3</p> <p>Initiating Condition: UNUSUAL EVENT UNPLANNED increase in RCS temperature.</p> <p>Operating Mode Applicability: Cold Shutdown, Refueling</p> <p>Example Emergency Action Levels: (1 or 2)</p> <p>Note: The Emergency Director should declare the Unusual Event promptly upon determining that 15 minutes time has been exceeded, or will likely be exceeded.</p> <ol style="list-style-type: none"> UNPLANNED increase in RCS temperature to greater than (site-specific Technical Specification cold shutdown temperature limit). Loss of ALL RCS temperature and (reactor vessel/RCS [PWR] or RPV [BWR]) level indication for 15 minutes or longer. 	<p style="text-align: right;">CU5</p> <p>Initiating Condition: UNPLANNED rise in RCS temperature.</p> <p>Operating Mode Applicability: 3, 4</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <ol style="list-style-type: none"> UNPLANNED rise in RCS temperature > 212°F due to loss of decay heat removal. <p>OR</p> <ol style="list-style-type: none"> Loss of the following for <u>≥ 15 minutes</u>. <ul style="list-style-type: none"> ALL RCS temperature indications AND ALLRPV level indications 	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific Technical Specification cold shutdown temperature limit to ensure timely classification.</p> <p>2) Changed the word increase to rise in the initiating condition to be consistent with operations language and training.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification									
<p style="text-align: right;">CG1</p> <p>Initiating Condition: GENERAL EMERGENCY</p> <p>Loss of (reactor vessel/RCS [PWR] or RPV [BWR]) inventory affecting fuel clad integrity with containment challenged.</p> <p>Operating Mode Applicability: Cold Shutdown, Refueling</p> <p>Example Emergency Action Levels: (1 or 2)</p> <p>Note: The Emergency Director should declare the General Emergency promptly upon determining that 30 minutes time has been exceeded, or will likely be exceeded.</p> <p>1. a. (Reactor vessel/RCS [PWR] or RPV [BWR]) vessel level less than (site-specific level) for 30 minutes or longer.</p> <p style="padding-left: 20px;">AND</p> <p>b. ANY indication from the Containment Challenge Table</p> <p>2. a. (Reactor vessel/RCS [PWR] or RPV [BWR]) vessel level cannot be monitored for 30 minutes or longer.</p> <p style="padding-left: 20px;">AND</p> <p>b. Core uncover is indicated by ANY of the following:</p> <ul style="list-style-type: none"> • (Site-specific radiation monitor) reading greater than (site-specific value) • Erratic source range monitor indication [PWR] • UNPLANNED increase in (site-specific sump and/or tank levels) of sufficient magnitude to indicate core uncover • (Other site-specific indications) <p style="padding-left: 20px;">AND</p> <p>c. ANY indication from the Containment Challenge Table).</p> <table border="1" style="width: 100%; border-collapse: collapse;"> <tr> <td style="padding: 2px;">Table: Containment Challenge Table</td> </tr> <tr> <td style="padding: 2px;"> <ul style="list-style-type: none"> • CONTAINMENT CLOSURE not established* • (Explosive mixture) exists inside containment • UNPLANNED increase in containment pressure • Secondary containment radiation monitor reading above (site-specific value) [BWR] </td> </tr> <tr> <td style="padding: 2px;">* if CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute core uncover time limit, then escalation to a General Emergency is not required.</td> </tr> </table>	Table: Containment Challenge Table	<ul style="list-style-type: none"> • CONTAINMENT CLOSURE not established* • (Explosive mixture) exists inside containment • UNPLANNED increase in containment pressure • Secondary containment radiation monitor reading above (site-specific value) [BWR] 	* if CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute core uncover time limit, then escalation to a General Emergency is not required.	<p style="text-align: right;">CG6</p> <p>Initiating Condition:</p> <p>Loss of reactor vessel / RCS inventory affecting fuel clad integrity with containment challenged.</p> <p>Operating Mode Applicability: 3, 4</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> • The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <p>1. a. RPV water level < 0 inches TAF for ≥ 30 minutes.</p> <p style="padding-left: 20px;">AND</p> <p>b. Any Containment Challenge Indication (Table C4)</p> <p style="padding-left: 20px;">OR</p> <p>2. a. RPV water level unknown for ≥ 30 minutes.</p> <p style="padding-left: 20px;">AND</p> <p>b. Core uncover is indicated by ANY of the following:</p> <ol style="list-style-type: none"> 1. Table C3 indications of a sufficient magnitude to indicate core uncover. <li style="padding-left: 20px;">OR 2. Refuel Floor Area Radiation Monitor C-10, North Wall, reading >3 R/hr. <p style="padding-left: 20px;">AND</p> <p>c. ANY Containment Challenge Indication (Table C4)</p> <table border="1" style="width: 100%; border-collapse: collapse; margin-top: 10px;"> <tr> <td style="text-align: center; padding: 2px;">Table C3 Indications of RCS Leakage</td> </tr> <tr> <td style="padding: 2px;"> <ul style="list-style-type: none"> • UNPLANNED floor or equipment sump level rise* • UNPLANNED Torus level rise* • UNPLANNED vessel make up rate rise* • Observation of leakage or inventory loss </td> </tr> <tr> <td style="padding: 2px;">*Rise in level is attributed to a loss of RPV inventory.</td> </tr> </table> <table border="1" style="width: 100%; border-collapse: collapse; margin-top: 10px;"> <tr> <td style="text-align: center; padding: 2px;">Table C4 Containment Challenge Indications</td> </tr> <tr> <td style="padding: 2px;"> <ul style="list-style-type: none"> • Primary Containment Hydrogen Concentration ≥ 6% and Oxygen Concentration ≥ 5% • UNPLANNED rise in containment pressure • CONTAINMENT CLOSURE <u>not</u> established* • ANY Secondary Containment radiation monitor > EMG-3200.11 Maximum Safe (Table 12). </td> </tr> <tr> <td style="padding: 2px;">* if CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute core uncover time limit, then escalation to a General Emergency is not required.</td> </tr> </table>	Table C3 Indications of RCS Leakage	<ul style="list-style-type: none"> • UNPLANNED floor or equipment sump level rise* • UNPLANNED Torus level rise* • UNPLANNED vessel make up rate rise* • Observation of leakage or inventory loss 	*Rise in level is attributed to a loss of RPV inventory.	Table C4 Containment Challenge Indications	<ul style="list-style-type: none"> • Primary Containment Hydrogen Concentration ≥ 6% and Oxygen Concentration ≥ 5% • UNPLANNED rise in containment pressure • CONTAINMENT CLOSURE <u>not</u> established* • ANY Secondary Containment radiation monitor > EMG-3200.11 Maximum Safe (Table 12). 	* if CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute core uncover time limit, then escalation to a General Emergency is not required.	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific levels, radiation monitors, and sumps and tanks to ensure timely classification.</p> <p>2) Listed Explosive mixture in the Containment Challenge Table to ensure timely classification.</p> <p>3) Worded "cannot be monitored" as unknown to ensure clarity for instances when the indicator is working but is over/under ranged. This is also in keeping with current EAL wording.</p>
Table: Containment Challenge Table											
<ul style="list-style-type: none"> • CONTAINMENT CLOSURE not established* • (Explosive mixture) exists inside containment • UNPLANNED increase in containment pressure • Secondary containment radiation monitor reading above (site-specific value) [BWR] 											
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* if CONTAINMENT CLOSURE is re-established prior to exceeding the 30-minute core uncover time limit, then escalation to a General Emergency is not required.											

NEI 99-01 Rev 6	Proposed EAL	Justification			
<p style="text-align: right;">CS1</p> <p>Initiating Condition: SITE AREA EMERGENCY</p> <p>Loss of (reactor vessel/RCS [PWR] or RPV [BWR]) inventory affecting core decay heat removal capability.</p> <p>Operating Mode Applicability: Cold Shutdown, Refueling</p> <p>Example Emergency Action Levels: (1 or 2 or 3)</p> <p>Note: The Emergency Director should declare the Site Area Emergency promptly upon determining that 30 minutes time has been exceeded, or will likely be exceeded.</p> <ol style="list-style-type: none"> 1. a. CONTAINMENT CLOSURE not established. AND b. (Reactor vessel/RCS [PWR] or RPV [BWR]) level less than (site-specific level). 2. a. CONTAINMENT CLOSURE established. AND b. (Reactor vessel/RCS [PWR] or RPV [BWR]) level less than (site-specific level). 3. a. (Reactor vessel/RCS [PWR] or RPV [BWR]) level cannot be monitored for 30 minutes or longer. AND b. Core uncover is indicated by ANY of the following: <ul style="list-style-type: none"> • (Site-specific radiation monitor) reading greater than (site-specific value) • Erratic source range monitor indication [PWR] • UNPLANNED increase in (site-specific sump and/or tank levels) of sufficient magnitude to indicate core uncover • (Other site-specific indications) 	<p style="text-align: right;">CS6</p> <p>Initiating Condition:</p> <p>Loss of reactor vessel / RCS inventory affecting core decay heat removal capabilities.</p> <p>Operating Mode Applicability: 3, 4</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> • The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <ol style="list-style-type: none"> 1. With CONTAINMENT CLOSURE <u>not</u> established, RPV water level < 56 inches TAF. OR 2. With CONTAINMENT CLOSURE established, RPV water level < 0 inches TAF. OR 3. a. RPV water level unknown for ≥ 30 minutes AND b. Core uncover is indicated by ANY of the following: <ul style="list-style-type: none"> • Table C3 indications of a sufficient magnitude to indicate core uncover. OR <ul style="list-style-type: none"> • Refuel Floor Area Radiation Monitor C-10, North Wall, reading >3 R/hr. <table border="1" style="width: 100%; margin-top: 10px;"> <thead> <tr> <th style="text-align: center;">Table C3 Indications of RCS Leakage</th> </tr> </thead> <tbody> <tr> <td> <ul style="list-style-type: none"> • UNPLANNED floor or equipment sump level rise* • UNPLANNED Torus level rise* • UNPLANNED vessel make up rate rise* • Observation of leakage or inventory loss </td> </tr> <tr> <td>*Rise in level is attributed to a loss of RPV inventory.</td> </tr> </tbody> </table>	Table C3 Indications of RCS Leakage	<ul style="list-style-type: none"> • UNPLANNED floor or equipment sump level rise* • UNPLANNED Torus level rise* • UNPLANNED vessel make up rate rise* • Observation of leakage or inventory loss 	*Rise in level is attributed to a loss of RPV inventory.	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific values for level, radiation monitors, and sumps and tanks to ensure timely classification.</p> <p>2) Worded "cannot be monitored" as unknown to ensure clarity for instances when the indicator is working but is over/under ranged. This is also in keeping with current EAL wording.</p>
Table C3 Indications of RCS Leakage					
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*Rise in level is attributed to a loss of RPV inventory.					

NEI 99-01 Rev 6	Proposed EAL	Justification			
<p style="text-align: right;">CA1</p> <p>Initiating Condition: ALERT Loss of (reactor vessel/RCS [PWR] or RPV [BWR]) inventory Operating Mode Applicability: Cold Shutdown, Refueling Example Emergency Action Levels: (1 or 2) Note: The Emergency Director should declare the Alert promptly upon determining that 15 minutes time has been exceeded, or will likely be exceeded.</p> <ol style="list-style-type: none"> 1. Loss of (reactor vessel/RCS [PWR] or RPV [BWR]) inventory as indicated by level less than (site-specific level). 2. <ol style="list-style-type: none"> a. (Reactor vessel/RCS [PWR] or RPV [BWR]) level cannot be monitored for 15 minutes or longer <p>AND</p> <ol style="list-style-type: none"> b. UNPLANNED increase in (site-specific sump and/or tank) levels due to a loss of (reactor vessel/RCS [PWR] or RPV [BWR]) inventory. 	<p style="text-align: right;">CA6</p> <p>Initiating Condition: Loss of reactor vessel / RCS inventory Operating Mode Applicability: 4, 5 Emergency Action Level (EAL): Note:</p> <ul style="list-style-type: none"> • The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <ol style="list-style-type: none"> 1. Loss of RPV inventory as indicated by level < 86 inches TAF. <p style="text-align: center;">OR</p> <ol style="list-style-type: none"> 2. <ol style="list-style-type: none"> a. RPV level unknown for <u>≥ 15 minutes.</u> <p style="text-align: center;">AND</p> <ol style="list-style-type: none"> b. Loss of RPV inventory per Table C3 indications. <table border="1" style="width: 100%; margin-top: 10px;"> <thead> <tr> <th style="text-align: center;">Table C3 Indications of RCS Leakage</th> </tr> </thead> <tbody> <tr> <td style="text-align: center;"> <ul style="list-style-type: none"> • UNPLANNED floor or equipment sump level rise* • UNPLANNED Torus level rise* • UNPLANNED vessel make up rate rise* • Observation of leakage or inventory loss </td> </tr> <tr> <td style="text-align: center; padding-top: 5px;"> *Rise in level is attributed to a loss of RPV inventory. </td> </tr> </tbody> </table>	Table C3 Indications of RCS Leakage	<ul style="list-style-type: none"> • UNPLANNED floor or equipment sump level rise* • UNPLANNED Torus level rise* • UNPLANNED vessel make up rate rise* • Observation of leakage or inventory loss 	*Rise in level is attributed to a loss of RPV inventory.	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific levels, and sumps and tanks to ensure timely classification.</p> <p>2) Worded "cannot be monitored" as unknown to ensure clarity for instances when the indicator is working but is over/under ranged. This is also in keeping with current EAL wording.</p>
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*Rise in level is attributed to a loss of RPV inventory.					

NEI 99-01 Rev 6	Proposed EAL	Justification			
<p style="text-align: right;">CU1</p> <p>Initiating Condition: UNUSUAL EVENT</p> <p>UNPLANNED loss of (reactor vessel/RCS [PWR] or RPV [BWR]) inventory for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>Cold Shutdown, Refueling</p> <p>Example Emergency Action Levels: (1 or 2)</p> <p>Note: The Emergency Director should declare the Unusual Event promptly upon determining that 15 minutes has been exceeded, or will likely be exceeded.</p> <ol style="list-style-type: none"> 1. UNPLANNED loss of reactor coolant results in (reactor vessel/RCS [PWR] or RPV [BWR]) level less than a required lower limit for 15 minutes or longer. 2. a. (Reactor vessel/RCS [PWR] or RPV [BWR]) level cannot be monitored. <p style="text-align: center;">AND</p> <ol style="list-style-type: none"> b. UNPLANNED increase in (site-specific sump and/or tank) levels. 	<p style="text-align: right;">CU6</p> <p>Initiating Condition:</p> <p>UNPLANNED loss of reactor vessel / RCS inventory for 15 minutes or longer.</p> <p>Operating Mode Applicability:</p> <p>4, 5</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> • The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <ol style="list-style-type: none"> 1. UNPLANNED loss of reactor coolant results in the inability to restore and maintain RPV level above the procedurally established lower limit for ≥ 15 minutes. <p style="text-align: center;">OR</p> <ol style="list-style-type: none"> 2. a. RPV level unknown <p style="text-align: center;">AND</p> <ol style="list-style-type: none"> b. Loss of RPV inventory per Table C3 indications. <table border="1" data-bbox="764 728 1354 987" style="width: 100%; border-collapse: collapse;"> <thead> <tr> <th style="text-align: center;">Table C3 Indications of RCS Leakage</th> </tr> </thead> <tbody> <tr> <td style="padding: 5px;"> <ul style="list-style-type: none"> • UNPLANNED floor or equipment sump level rise* • UNPLANNED Torus level rise* • UNPLANNED vessel make up rate rise* • Observation of leakage or inventory loss </td> </tr> <tr> <td style="padding: 5px;"> <p>*Rise in level is attributed to a loss of RPV inventory.</p> </td> </tr> </tbody> </table>	Table C3 Indications of RCS Leakage	<ul style="list-style-type: none"> • UNPLANNED floor or equipment sump level rise* • UNPLANNED Torus level rise* • UNPLANNED vessel make up rate rise* • Observation of leakage or inventory loss 	<p>*Rise in level is attributed to a loss of RPV inventory.</p>	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Described "a required lower limit" as a procedurally established lower limit, and listed site specific sumps and tanks to ensure timely classification.</p> <p>2) Worded "cannot be monitored" as unknown to ensure clarity for instances when the indicator is working but is over/under ranged. This is also in keeping with current EAL wording.</p>
Table C3 Indications of RCS Leakage					
<ul style="list-style-type: none"> • UNPLANNED floor or equipment sump level rise* • UNPLANNED Torus level rise* • UNPLANNED vessel make up rate rise* • Observation of leakage or inventory loss 					
<p>*Rise in level is attributed to a loss of RPV inventory.</p>					

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">HG1</p> <p>Initiating Condition: GENERAL EMERGENCY</p> <p>HOSTILE ACTION resulting in loss of physical control of the facility.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels:</p> <p>1. a. A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by the (site-specific security shift supervision).</p> <p style="padding-left: 40px;">AND</p> <p>b. EITHER of the following:</p> <p style="padding-left: 80px;">1. ANY of the following safety functions cannot be controlled or maintained.</p> <ul style="list-style-type: none"> ▪ Reactivity control ▪ Core cooling [PWR] / RPV water level [BWR] ▪ RCS heat removal <p style="padding-left: 80px;">OR</p> <p style="padding-left: 80px;">2. Damage to spent fuel has occurred or is IMMINENT</p>	<p style="text-align: right;">HG1</p> <p>Initiating Condition:</p> <p>HOSTILE ACTION resulting in loss of physical control of the facility.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>1. A notification from the Security Force that a HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA.</p> <p style="padding-left: 40px;">AND</p> <p>2. a. ANY Table H1 safety function <u>cannot</u> be controlled or maintained.</p> <p style="padding-left: 80px;">OR</p> <p style="padding-left: 80px;">b. Damage to spent fuel has occurred or is IMMINENT</p> <div style="border: 1px solid black; padding: 5px; margin: 10px auto; width: fit-content;"> <p style="text-align: center; margin: 0;">Table H1 Safety Functions</p> <ul style="list-style-type: none"> • Reactivity Control (ability to shutdown the reactor and keep it shutdown) • RPV Water Level (ability to cool the core) • RCS Heat Removal (ability to maintain a heat sink) </div>	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) List site security shift supervision as Security Force.</p> <p>2) Added descriptors to better explain each safety function and allow for a timely classification.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">HS1</p> <p>Initiating Condition: SITE AREA EMERGENCY</p> <p>HOSTILE ACTION within the Protected Area.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels:</p> <p>A HOSITLE ACTION is occurring or has occurred within the PROTECTED AREA as reported by the (site-security shift supervision).</p>	<p style="text-align: right;">HS1</p> <p>Initiating Condition:</p> <p>HOSTILE ACTION within the Protected Area.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>A notification from the Security Force that a HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA.</p>	<p> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) List site security shift supervision as Security Force.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">HA1</p> <p>Initiating Condition: ALERT</p> <p>HOSTILE ACTION within the OWNER CONTROLLED AREA or airborne attack threat within 30 minutes.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels: (1 or 2)</p> <ol style="list-style-type: none"> 1. A HOSTILE ACTION is occurring or has occurred within the OWNER CONTROLLED AREA as reported by the (site-specific security shift supervision). 2. A validated notification from NRC of an aircraft attack threat within 30 minutes of the site. 	<p style="text-align: right;">HA1</p> <p>Initiating Condition:</p> <p>HOSTILE ACTION within the OWNER CONTROLLED AREA or airborne attack threat within 30 minutes.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <ol style="list-style-type: none"> 1. A validated notification from NRC of an aircraft attack threat < 30 minutes from the site. <p style="text-align: center;">OR</p> <ol style="list-style-type: none"> 2. Notification by the Security Force that a HOSTILE ACTION is occurring or has occurred within the OWNER CONTROLLED AREA. 	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) List site security shift supervision as Security Force.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p>Initiating Condition: UNUSUAL EVENT HU1</p> <p>Confirmed SECURITY CONDITION or threat.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels: (1 or 2 or 3)</p> <ol style="list-style-type: none"> 1. A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by the (site-specific security shift supervision). 2. Notification of a credible security threat directed at the site. 3. A validated notification from the NRC providing information of an aircraft threat. 	<p>Initiating Condition: HU1</p> <p>Confirmed SECURITY CONDITION or threat.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>:</p> <ol style="list-style-type: none"> 1. Notification of a credible security threat directed at the site as determined per SY-AA-101-132, Security Assessment and Response to Unusual Activities. <p>OR</p> <ol style="list-style-type: none"> 2. A validated notification from the NRC providing information of an aircraft threat. <p>OR</p> <ol style="list-style-type: none"> 3. Notification by the Security Force of a SECURITY CONDITION that does <u>not</u> involve a HOSTILE ACTION. 	<p><input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation</p> <p>1) List site security shift supervision as Security Force.</p> <p>2) Further described credible security threat through listing a site specific procedure.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification		
<p style="text-align: right;">HS6</p> <p>Initiating Condition: SITE AREA EMERGENCY</p> <p>Inability to control a key safety function from outside the Control Room.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels: (1 and 2)</p> <p>Note: The Emergency Director should declare the Site Area Emergency promptly upon determining that (site-specific number of minutes) has been exceeded, or will likely be exceeded.</p> <ol style="list-style-type: none"> An event has resulted in plant control being transferred from the Control Room to (site-specific remote shutdown panels and local control stations). Control of ANY of the following key safety functions is not reestablished within (site-specific number of minutes). <ul style="list-style-type: none"> Reactivity control Core cooling [PWR] / RPV water level [BWR] RCS heat removal 	<p style="text-align: right;">HS2</p> <p>Initiating Condition:</p> <p>Inability to control a key safety function from outside the Control Room.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <ol style="list-style-type: none"> A Control Room evacuation has resulted in plant control being transferred from the Control Room to alternate locations per ABN-30 Control Room Evacuation. <p>AND</p> <ol style="list-style-type: none"> Control of ANY Table H1 key safety function is not reestablished in < 15 minutes. <table border="1" data-bbox="821 745 1306 935" style="margin-left: auto; margin-right: auto;"> <thead> <tr> <th>Table H1 Safety Functions</th> </tr> </thead> <tbody> <tr> <td> <ul style="list-style-type: none"> Reactivity Control (ability to shutdown the reactor and keep it shutdown) RPV Water Level (ability to cool the core) RCS Heat Removal (ability to maintain a heat sink) </td> </tr> </tbody> </table>	Table H1 Safety Functions	<ul style="list-style-type: none"> Reactivity Control (ability to shutdown the reactor and keep it shutdown) RPV Water Level (ability to cool the core) RCS Heat Removal (ability to maintain a heat sink) 	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) EAL uses the site specific Control Room evacuation procedures to effectively list all of the alternate locations, panels, and stations requested by the developer notes. This would be the procedures the Control Room would enter should such an event occur, this allows for greater clarity as to when this EAL would apply than if each panel and station used in alternate shutdown were to be listed.</p> <p>2) Added descriptors to better explain each safety function and allow for a timely classification.</p> <p>3) Changed "An event" to "A Control Room evacuation" to remove confusion if partial plant control was transferred to outside the control room with the control room still manned, due to testing or equipment failure.</p>
Table H1 Safety Functions				
<ul style="list-style-type: none"> Reactivity Control (ability to shutdown the reactor and keep it shutdown) RPV Water Level (ability to cool the core) RCS Heat Removal (ability to maintain a heat sink) 				

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">HA6</p> <p>Initiating Condition: ALERT</p> <p>Control Room evacuation resulting in transfer of plant control to alternate locations.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels:</p> <p>An event has resulted in plant control being transferred from the Control Room to (site-specific remote shutdown panels and local control stations).</p>	<p style="text-align: right;">HA2</p> <p>Initiating Condition:</p> <p>Control Room evacuation resulting in transfer of plant control to alternate locations.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>A Control Room evacuation has resulted in plant control being transferred from the Control Room to alternate locations per ABN-30 Control Room Evacuation.</p>	<p> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) EAL uses the site specific Control Room evacuation procedures to effectively list all of the alternate locations, panels, and stations requested by the developer notes. This would be the procedures the Control Room would enter should such an event occur, this allows for greater clarity as to when this EAL would apply than if each panel and station used in alternate shutdown were to be listed,</p> <p>2) Changed "An event" to "A Control Room evacuation" to remove confusion if partial plant control was transferred to outside the control room with the control room still manned, due to testing or equipment failure.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification		
<p style="text-align: right;">HU4</p> <p>Initiating Condition: UNUSUAL EVENT</p> <p>FIRE potentially degrading the level of safety of the plant.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels: (1 or 2 or 3 or 4)</p> <p>Note: The Emergency Director should declare the Unusual Event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded.</p> <ol style="list-style-type: none"> 1. a. A FIRE is NOT extinguished within 15-minutes of ANY of the following FIRE detection indications: <ul style="list-style-type: none"> • Report from the field (i.e., visual observation) • Receipt of multiple (more than 1) fire alarms or indications • Field verification of a single fire alarm <p style="text-align: center;">AND</p> b. The FIRE is located within ANY of the following plant rooms or areas: (site-specific list of plant rooms or areas) 2. a. Receipt of a single fire alarm (i.e., no other indications of a FIRE). AND b. The FIRE is located within ANY of the following plant rooms or areas: (site-specific list of plant rooms or areas) AND c. The existence of a FIRE is not verified within 30-minutes of alarm receipt. 3. A FIRE within the plant or ISFSI [for plants with an ISFSI outside the plant Protected Area] PROTECTED AREA not extinguished within 60-minutes of the initial report, alarm or indication. 4. A FIRE within the plant or ISFSI [for plants with an ISFSI outside the plant Protected Area] PROTECTED AREA that requires firefighting support by an offsite fire response agency to extinguish. 	<p style="text-align: right;">HU3</p> <p>Initiating Condition:</p> <p>FIRE potentially degrading the level of safety of the plant.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> • The Emergency Director should declare the event promptly upon determining that the applicable time has been exceeded, or will likely be exceeded. <ol style="list-style-type: none"> 1. A FIRE in ANY Table H2 area is <u>not</u> extinguished in < 15-minutes of ANY of the following FIRE detection indications: <ul style="list-style-type: none"> • Report from the field (i.e., visual observation) • Receipt of multiple (more than 1) fire alarms or indications • Field verification of a single fire alarm <table border="1" style="margin-left: 40px;"> <thead> <tr> <th style="text-align: center;">Table H2 Vital Areas</th> </tr> </thead> <tbody> <tr> <td> <ul style="list-style-type: none"> • Reactor Building (when inerted the Drywell is exempt) • 4160V Switchgear Rooms (1C and 1D) • Control Room Complex (MOB, Upper and Lower Cable Spreading Rooms) • Main Transformer/Condensate Transfer Pad • Intake Structure • #1 EDG Vault • #2 EDG Vault • EDG Fuel Oil Storage Tank </td> </tr> </tbody> </table> <p style="text-align: center;">OR</p> <ol style="list-style-type: none"> 2. a. Receipt of a single fire alarm in ANY Table H2 area (i.e., no other indications of a FIRE). AND b. The existence of a FIRE is <u>not</u> verified in < 30 minutes of alarm receipt. OR 3. A FIRE within the plant or ISFSI PROTECTED AREA <u>not</u> extinguished in < 60 minutes of the initial report, alarm or indication. OR 4. A FIRE within the plant or ISFSI PROTECTED AREA that requires firefighting support by an offsite fire response agency to extinguish. 	Table H2 Vital Areas	<ul style="list-style-type: none"> • Reactor Building (when inerted the Drywell is exempt) • 4160V Switchgear Rooms (1C and 1D) • Control Room Complex (MOB, Upper and Lower Cable Spreading Rooms) • Main Transformer/Condensate Transfer Pad • Intake Structure • #1 EDG Vault • #2 EDG Vault • EDG Fuel Oil Storage Tank 	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed site specific list of plant rooms or areas that contain SAFETY SYSTEM equipment to ensure timely classification.</p>
Table H2 Vital Areas				
<ul style="list-style-type: none"> • Reactor Building (when inerted the Drywell is exempt) • 4160V Switchgear Rooms (1C and 1D) • Control Room Complex (MOB, Upper and Lower Cable Spreading Rooms) • Main Transformer/Condensate Transfer Pad • Intake Structure • #1 EDG Vault • #2 EDG Vault • EDG Fuel Oil Storage Tank 				

NEI 99-01 Rev 6	Proposed EAL	Justification
<p>Initiating Condition: UNUSUAL EVENT HU2</p> <p>Seismic event greater than OBE levels.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels:</p> <p>Seismic event greater than Operating Basis Earthquake (OBE) as indicated by: a. (site-specific indication that a seismic event met or exceeded OBE limits)</p>	<p>Initiating Condition: HU4</p> <p>Seismic event greater than OBE levels.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>Seismic event > Operating Basis Earthquake (OBE) as indicated by:</p> <ol style="list-style-type: none"> 1. Control Room personnel feel an actual or potential seismic event. <p>AND</p> <ol style="list-style-type: none"> 2. The occurrence of a seismic event has resulted in a spurious Reactor Scram or required a Plant shutdown in accordance with ABN-38, Station Seismic Event. 	<p><input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation</p> <p>1) Used Alternate developer notes allowed wording since specific Control Room indication of a seismic event > OBE is not available.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification								
<p style="text-align: right;">HA5</p> <p>Initiating Condition: ALERT</p> <p>Gaseous release impeding access to equipment necessary for normal plant operations, cooldown or shutdown.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels:</p> <p>Note: If the equipment in the listed room or area was already inoperable, or out of service, before the event occurred, then no emergency classification is warranted.</p> <p>1. a. Release of a toxic, corrosive, asphyxiant or flammable gas into any of the following plant rooms or areas: (site-specific list of plant rooms or areas with entry-related mode applicability identified)</p> <p>AND</p> <p>b. Entry into the room or area is prohibited or impeded.</p>	<p style="text-align: right;">HA5</p> <p>Initiating Condition:</p> <p>Gaseous release impeding access to equipment necessary for normal plant operations, cooldown or shutdown.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> If the equipment in the listed room or area was already inoperable, or out of service, before the event occurred, then no emergency classification is warranted. <p>1. Release of a toxic, corrosive, asphyxiant or flammable gas in a Table H3 area.</p> <table border="1" data-bbox="789 707 1325 880"> <thead> <tr> <th colspan="2" style="text-align: center;">Table H3 Areas with Entry Related Mode Applicability</th> </tr> <tr> <th style="text-align: center;">Area</th> <th style="text-align: center;">Entry Related Mode Applicability</th> </tr> </thead> <tbody> <tr> <td style="text-align: center;">Reactor Building*</td> <td style="text-align: center;">Modes 3 and 4</td> </tr> <tr> <td colspan="2" style="text-align: center;">*Areas required to establish shutdown cooling</td> </tr> </tbody> </table> <p>AND</p> <p>2. Entry into the room or area is prohibited or impeded</p>	Table H3 Areas with Entry Related Mode Applicability		Area	Entry Related Mode Applicability	Reactor Building*	Modes 3 and 4	*Areas required to establish shutdown cooling		<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed plant specific rooms and areas with entry related mode applicability to ensure timely classification.</p>
Table H3 Areas with Entry Related Mode Applicability										
Area	Entry Related Mode Applicability									
Reactor Building*	Modes 3 and 4									
*Areas required to establish shutdown cooling										

NEI 99-01 Rev 6	Proposed EAL	Justification
<p>Initiating Condition: UNUSUAL EVENT HU3</p> <p>Hazardous Event</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels: (1 or 2 or 3 or 4)</p> <p>Note: EAL #4 does not apply to routine traffic impediments such as fog, snow, ice, or vehicle breakdowns or accidents.</p> <ol style="list-style-type: none"> 1. A tomado strike within the PROTECTED AREA. 2. Internal room or area flooding of a magnitude sufficient to require manual or automatic electrical isolation of a SAFETY SYSTEM component needed for the current operating mode. 3. Movement of personnel within the PROTECTED AREA is impeded due to an offsite event involving hazardous materials (e.g., an offsite chemical spill or toxic gas release). 4. A hazardous event that results in on-site conditions sufficient to prohibit the plant staff from accessing the site via personal vehicles. 5. (Site-specific list of natural or technological hazard events) 	<p>Initiating Condition: HU6</p> <p>Hazardous Event</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>Note:</p> <ul style="list-style-type: none"> • EAL #4 does not apply to routine traffic impediments such as fog, snow, ice, or vehicle breakdowns or accidents. <ol style="list-style-type: none"> 1. Tornado strike within the PROTECTED AREA. <p>OR</p> <ol style="list-style-type: none"> 2. Internal room or area flooding of a magnitude sufficient to require manual or automatic electrical isolation of a SAFETY SYSTEM component required by Technical Specifications for the current operating mode. <p>OR</p> <ol style="list-style-type: none"> 3. Movement of personnel within the PROTECTED AREA is impeded due to an offsite event involving hazardous materials (e.g., an offsite chemical spill or toxic gas release). <p>OR</p> <ol style="list-style-type: none"> 4. A hazardous event that results in on-site conditions sufficient to prohibit the plant staff from accessing the site via personal vehicles. <p>OR</p> <ol style="list-style-type: none"> 5. Abnormal Intake Structure level, as indicated by EITHER: <ul style="list-style-type: none"> • > 4.5 ft. MSL (> 4.25 psig on PI-533-1172 and PI-533-1173 or > 4.5 ft MSL on CR-423-11 pt 24 and pt 23). <p>OR</p> <ul style="list-style-type: none"> • b. ≤ -3.0 ft. MSL (≤ 0.95 psig on PI-533-1172 and PI-533-1173 or ≤ -3.0 ft MSL on CR-423-11 pt 24 and pt 23). <p style="text-align: center;">MSL = Mean Sea Level</p> 	<p><input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation</p> <p>1) Included River water level as part of the site specific list of natural or technological hazard events. The EAL values selected are the current Approved UE EAL values.</p> <p>2) Changed the word "needed" to "required by Technical Specifications" in the EAL to be consistent with terminology used by operators and minimize confusion.</p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">HG8</p> <p>Initiating Condition: GENERAL EMERGENCY</p> <p>Other conditions exist which in the judgment of the Emergency Director warrant declaration of a General Emergency.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels: Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity or HOSTILE ACTION that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA Protective Action Guideline exposure levels off-site for more than the immediate site area.</p>	<p style="text-align: right;">HG7</p> <p>Initiating Condition:</p> <p>Other conditions exist which in the judgment of the Emergency Director warrant declaration of a General Emergency.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL): Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity or HOSTILE ACTION that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA Protective Action Guideline exposure levels off-site for more than the immediate site area..</p>	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">HS8</p> <p>Initiating Condition: SITE AREA EMERGENCY</p> <p>Other conditions exist which in the judgment of the Emergency Director warrant declaration of a Site Area Emergency.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels:</p> <p>Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public or HOSTILE ACTION that results in intentional damage or malicious acts; (1) toward site personnel or equipment that could lead to the likely failure of or; (2) that prevent effective access to equipment needed for the protection of the public. Any releases are not expected to result in exposure levels which exceed EPA Protective Action Guideline exposure levels beyond the site boundary.</p>	<p style="text-align: right;">HS7</p> <p>Initiating Condition:</p> <p>Other conditions exist which in the judgment of the Emergency Director warrant declaration of a Site Area Emergency.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public or HOSTILE ACTION that results in intentional damage or malicious acts; (1) toward site personnel or equipment that could lead to the likely failure of or; (2) that prevent effective access to equipment needed for the protection of the public. Any releases are not expected to result in exposure levels which exceed EPA Protective Action Guideline exposure levels beyond the site boundary.</p>	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">HA6</p> <p>Initiating Condition: ALERT</p> <p>Other conditions exist which in the judgment of the Emergency Director warrant declaration of an Alert.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels: Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION. Any releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.</p>	<p style="text-align: right;">HA7</p> <p>Initiating Condition:</p> <p>Other conditions exist which in the judgment of the Emergency Director warrant declaration of an Alert.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL): Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION. Any releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels.</p>	<p style="text-align: center;"> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">HU7</p> <p>Initiating Condition: UNUSUAL EVENT</p> <p>Other conditions existing which in the judgment of the Emergency director warrant declaration of an UNUSUAL EVENT.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels:</p> <p>Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which indicate a potential degradation of the level of safety of the plant or indicate a security threat to facility protection has been initiated. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs.</p>	<p style="text-align: right;">HU7</p> <p>Initiating Condition:</p> <p>Other conditions existing which in the judgment of the Emergency director warrant declaration of an UNUSUAL EVENT.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which indicate a potential degradation of the level of safety of the plant or indicate a security threat to facility protection has been initiated. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs.</p>	<p> <input checked="" type="checkbox"/> No Change <input type="checkbox"/> Difference <input type="checkbox"/> Deviation </p>

NEI 99-01 Rev 6	Proposed EAL	Justification
<p style="text-align: right;">E-HU1</p> <p>Initiating Condition: UNUSUAL EVENT</p> <p>Damage to a loaded cask CONFINEMENT BOUNDARY.</p> <p>Operating Mode Applicability:</p> <p>All</p> <p>Example Emergency Action Levels:</p> <p>Damage to a loaded cask CONFINEMENT BOUNDARY as indicated by an on-contact radiation reading greater than (2 times the site-specific cask specific technical specification allowable radiation level) on the surface of the spent fuel cask.</p>	<p style="text-align: right;">E-HU1</p> <p>Initiating Condition:</p> <p>Damage to a loaded cask CONFINEMENT BOUNDARY.</p> <p>Operating Mode Applicability:</p> <p>1, 2, 3, 4, D</p> <p>Emergency Action Level (EAL):</p> <p>Damage to a loaded cask CONFINEMENT BOUNDARY as indicated by a radiation reading:</p> <ul style="list-style-type: none"> • > 800mr/hr (gamma + neutron) 3 feet from the HSM surface OR • > 200mr/hr (gamma + neutron) outside the HSM door on centerline of DSC OR • > 40mr/hr (gamma + neutron) end of shield wall exterior 	<p style="text-align: center;"> <input type="checkbox"/> No Change <input checked="" type="checkbox"/> Difference <input type="checkbox"/> Deviation </p> <p>1) Listed 2x the site specific cask specific allowable radiation level as per Oyster Creek Generating Station ISFSI 10CFR72.212 Evaluation Rev. 5</p> <p>2) Not all technical specification radiation readings were on contact, one is a radiation reading at 3ft required by technical specification. Modified the EAL by removing the "on the surface" requirement, and required "a radiation reading" allowing for the technical specification 3ft reading to be added to the EAL.</p>

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS**

ARG1

Initiating Condition:

Release of gaseous radioactivity resulting in offsite dose greater than 1000 mR_{fem} TEDE or 5000 mR_{fem} thyroid CDE.

Operating Mode Applicability:

1, 2, 3, 4, D

Emergency Acton Level (EAL):

Notes:

- The Emergency Director should declare the ~~General Emergency event~~ promptly upon determining that the applicable time has been exceeded, or will likely be exceeded.
- If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded ~~the applicable time~~ 15 minutes.
- ~~Classification based on effluent monitor readings assumes that a release path to the environment is established.~~ If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.
- The pre-calculated effluent monitor values presented in EAL #1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

~~(1) Reading on ANY of the following radiation monitors greater than the reading shown for 15 minutes or longer:~~

~~(site specific monitor list and threshold values)~~

1. Readings on **ANY** Table R1 Effluent Monitor > **Table R1 value** for **≥ 15 minutes:**

Table R1 Effluent Monitor Thresholds	
Effluent Monitor	General Emergency
Main Stack RAGEMS	8.69 E+01 uCi/cc HRM OR 3.53 E-08 amps HRM
Turbine Building RAGEMS	7.17 E-01 uCi/cc HRM
HRM = High Range Monitor	

OR

Table OCGS 3-2 OCGS EAL Technical Basis

RECOGNITION CATEGORY

ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS

2. Dose assessment using actual meteorology indicates doses at or beyond (~~site-specific dose-receptor point~~) the site boundary of **EITHER**:

a. **> 1000 mRem TEDE**

OR

b. **> 5000 mRem CDE Thyroid**

OR

~~Field survey results indicate **EITHER** of the following at or beyond (site-specific dose-receptor point):~~

~~Closed window dose rates greater than 1,000 mR/hr expected to continue for 60 minutes or longer.~~

~~Analyses of field survey samples indicate thyroid CDE greater than 5,000 mrem for one hour of inhalation.~~

3. Field survey results at or beyond the site boundary indicate **EITHER**:

a. Gamma (closed window) dose rates **>1000 mR/hr** are expected to continue for **≥ 60 minutes**.

OR

b. Analyses of field survey samples indicate **> 5000 mRem CDE Thyroid** for **60 minutes** of inhalation.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude will require implementation of protective actions for the public.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at the EPA PAG of 1000 mRrem while the 5000 mRrem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.

Table OCGS 3-2 OCGS EAL Technical Basis

RECOGNITION CATEGORY

ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

Basis Reference(s):

1. NEI 99-01 Rev 6, AG1
2. EP-EAL-0610, Criteria for Choosing Radiological Gaseous Effluent EAL Threshold Oyster Creek Generating Station
3. BNE Correspondence dated February 1, 2007
4. CY-OC-170-301, Offsite Dose Calculation Manual for Oyster Creek
5. EP-AA-110-200, Dose Assessment
6. EP-AA-110-201, On Shift Dose Assessment

Table OCGS 3-2 OCGS EAL Technical Basis

RECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS

ARS1**Initiating Condition:**

Release of gaseous radioactivity resulting in offsite dose greater than 100 mRrem TEDE or 500 mRrem thyroid CDE.

Operating Mode Applicability:

1, 2, 3, 4, D

Emergency Acton Level (EAL):**Notes:**

- The Emergency Director should declare the **Site Area Emergency event** promptly upon determining that the applicable time has been exceeded, or will likely be exceeded.
- If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded **the applicable time 15 minutes**.
- **Classification based on effluent monitor readings assumes that a release path to the environment is established.** If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.
- The pre-calculated effluent monitor values presented in EAL #1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.

~~(1) — Reading on ANY of the following radiation monitors greater than the reading shown for 15 minutes or longer:~~

~~(site specific monitor list and threshold values)~~

~~(2) — Dose assessment using actual meteorology indicates doses greater than 100 mrem TEDE or 500 mrem thyroid CDE at or beyond (site specific dose receptor point):~~

~~(3) — Field survey results indicate **EITHER** of the following at or beyond (site specific dose receptor point):~~

- ~~• — Closed window dose rates greater than 100 mR/hr expected to continue for 60 minutes or longer.~~
- ~~• — Analyses of field survey samples indicate thyroid CDE greater than 500 mrem for one hour of inhalation.~~

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS**

1. Readings on **ANY** Table R1 Effluent Monitor > **Table R1 value** for **≥ 15 minutes**:

Table R1 Effluent Monitor Thresholds	
Effluent Monitor	Site Area Emergency
Main Stack RAGEMS	8.69 E+00 uCi/cc HRM OR 3.53 E-09 amps HRM
Turbine Building RAGEMS	3.65 E+05 cpm LRM
HRM = High Range Monitor LRM = Low Range Monitor	

OR

2. Dose assessment using actual meteorology indicates doses at or beyond the site boundary of **EITHER**:

a. > 100 mRem TEDE

OR

b. > 500 mRem CDE Thyroid

OR

3. Field survey results at or beyond the site boundary indicate **EITHER**:

a. Gamma (closed window) dose rates >100 mR/hr are expected to continue for **≥ 60 minutes**.

OR

b. Analyses of field survey samples indicate > 500 mRem CDE Thyroid for **60 minutes** of inhalation.

Basis:

This IC addresses a release of gaseous radioactivity that results in projected or actual offsite doses greater than or equal to 10% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude are associated with the failure of plant systems needed for the protection of the public.

Table OCGS 3-2 OCGS EAL Technical Basis

RECOGNITION CATEGORY

ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 10% of the EPA PAG of 1000 mRrem while the 500 mRrem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

Escalation of the emergency classification level would be via IC RAG1.

Basis Reference(s):

1. NEI 99-01 Rev 6, AS1
2. EP-EAL-0610, Criteria for Choosing Radiological Gaseous Effluent EAL Threshold Oyster Creek Generating Station
3. BNE Correspondence dated February 1, 2007
4. CY-OC-170-301, Offsite Dose Calculation Manual for Oyster Creek
5. EP-AA-110-200, Dose Assessment
6. EP-AA-110-201, On Shift Dose Assessment

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS**ARA1****Initiating Condition:**

Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mRrem TEDE or 50 mRrem thyroid CDE.

Operating Mode Applicability:

1, 2, 3, 4, D

Emergency Acton Level (EAL):**Notes:**

- The Emergency Director should declare the **Alert event** promptly upon determining that the applicable time has been exceeded, or will likely be exceeded.
 - If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded **the applicable time 15 minutes**.
 - **Classification based on effluent monitor readings assumes that a release path to the environment is established**. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.
 - The pre-calculated effluent monitor values presented in EAL #1 should be used for emergency classification assessments until the results from a dose assessment using actual meteorology are available.
- ~~(1) — Reading on ANY of the following radiation monitors greater than the reading shown for 15 minutes or longer:~~
- ~~(site specific monitor list and threshold values)~~
- ~~(2) — Dose assessment using actual meteorology indicates doses greater than 10 mrem TEDE or 50 mrem thyroid CDE at or beyond (site specific dose receptor point).~~
- ~~(3) — Analysis of a liquid effluent sample indicates a concentration or release rate that would result in doses greater than 10 mrem TEDE or 50 mrem thyroid CDE at or beyond (site specific dose receptor point) for one hour of exposure.~~
- ~~(4) — Field survey results indicate EITHER of the following at or beyond (site specific dose receptor point):~~
- ~~• Closed window dose rates greater than 10 mR/hr expected to continue for 60 minutes or longer.~~
 - ~~• Analyses of field survey samples indicate thyroid CDE greater than 50 mrem for one hour of inhalation.~~

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS**

1. Readings on **ANY** Table R1 Effluent Monitor > **Table R1 value** for **≥ 15 minutes**.

Table R1 Effluent Monitor Thresholds	
Effluent Monitor	Alert
Main Stack RAGEMS	8.69 E-01 uCi/cc HRM OR 3.53 E-10 amps HRM
Turbine Building RAGEMS	3.65 E+04 cpm LRM
HRM = High Range Monitor LRM = Low Range Monitor	

OR

2. Dose assessment using actual meteorology indicates doses at or beyond the site boundary of **EITHER**:

- a. > 10 mRem TEDE

OR

- b. > 50 mRem CDE Thyroid

OR

3. Analysis of a liquid effluent sample indicates a concentration or release rate that would result in doses greater than **EITHER** of the following at or beyond the site boundary:

- a. 10 mRem TEDE for 60 minutes of exposure

OR

- b. 50 mRem CDE Thyroid for 60 minutes of exposure

OR

4. Field survey results at or beyond the site boundary indicate **EITHER**:

Table OCGS 3-2 OCGS EAL Technical Basis

RECOGNITION CATEGORY

ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS

- a. Gamma (closed window) dose rates > 10 mR/hr are expected to continue for \geq 60 minutes.

OR

- b. Analyses of field survey samples indicate > 50 mRem CDE Thyroid for 60 minutes of inhalation.

Basis:

This IC addresses a release of gaseous or liquid radioactivity that results in projected or actual offsite doses greater than or equal to 1% of the EPA Protective Action Guides (PAGs). It includes both monitored and un-monitored releases. Releases of this magnitude represent an actual or potential substantial degradation of the level of safety of the plant as indicated by a radiological release that significantly exceeds regulatory limits (e.g., a significant uncontrolled release).

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

The TEDE dose is set at 1% of the EPA PAG of 1000 mRem while the 50 mRem thyroid CDE was established in consideration of the 1:5 ratio of the EPA PAG for TEDE and thyroid CDE.

The radwaste liquid discharge system is currently closed off with a plant modification installed blank flange. To perform a discharge would require a plant modification to remove the flange. Since the liquid radwaste system is not operable, no EAL threshold has been developed for this release point.

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

Escalation of the emergency classification level would be via IC RAS1.

Basis Reference(s):

1. NEI 99-01 Rev 6, AA1

Table OCGS 3-2 OCGS EAL Technical Basis

RECOGNITION CATEGORY

ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS

2. EP-EAL-0610, Criteria for Choosing Radiological Gaseous Effluent EAL Threshold Oyster Creek Generating Station
3. BNE Correspondence dated February 1, 2007
4. CY-OC-170-301, Offsite Dose Calculation Manual for Oyster Creek
5. ABN-27, Inadvertent Overboard Radioactive Release or Cross Contamination
6. EP-EAL-0617, Oyster Creek Criteria for Choosing Liquid Effluent EAL Threshold Values

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS**

ARU1

Initiating Condition:

Release of gaseous or liquid radioactivity greater than 2 times the ~~ODCM (site-specific effluent release controlling document)~~ limits for 60 minutes or longer.

Operating Mode Applicability:

1, 2, 3, 4, D

Emergency Acton Level (EAL):

Notes:

- The Emergency Director should declare the ~~Unusual Event event~~ promptly upon determining that ~~60 minutes~~ the applicable time has been exceeded, or will likely be exceeded.
- If an ongoing release is detected and the release start time is unknown, assume that the release duration has exceeded ~~the applicable time~~ 60 minutes.
- ~~Classification based on effluent monitor readings assumes that a release path to the environment is established.~~ If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.

~~(1) Reading on ANY effluent radiation monitor greater than 2 times the (site-specific effluent release controlling document) limits for 60 minutes or longer:~~

~~(site-specific monitor list and threshold values corresponding to 2 times the controlling document limits)~~

~~(2) Reading on ANY effluent radiation monitor greater than 2 times the alarm setpoint established by a current radioactivity discharge permit for 60 minutes or longer.~~

~~(3) Sample analysis for a gaseous or liquid release indicates a concentration or release rate greater than 2 times the (site-specific effluent release controlling document) limits for 60 minutes or longer.~~

1. Readings on **ANY** Table R1 Effluent Monitor > **Table R1 value** for **≥ 60 minutes**:

Table R1 Effluent Monitor Thresholds	
Effluent Monitor	Unusual Event
Main Stack RAGEMS	4.07 E+03 cps LRM
Turbine Building RAGEMS	4.16 E+02 cpm LRM
LRM = Low Range Monitor	

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS

OR

2. Confirmed sample analyses for gaseous or liquid releases indicate concentrations or release rates > 2 times ODCM Limit with a release duration of \geq 60 minutes.

Basis:

This IC addresses a potential decrease in the level of safety of the plant as indicated by a low-level radiological release that exceeds regulatory commitments for an extended period of time (e.g., an uncontrolled release). It includes any gaseous or liquid radiological release, monitored or un-monitored, including those for which a radioactivity discharge permit is normally prepared.

Nuclear power plants incorporate design features intended to control the release of radioactive effluents to the environment. Further, there are administrative controls established to prevent unintentional releases, and to control and monitor intentional releases. The occurrence of an extended, uncontrolled radioactive release to the environment is indicative of degradation in these features and/or controls.

Radiological effluent EALs are also included to provide a basis for classifying events and conditions that cannot be readily or appropriately classified on the basis of plant conditions alone. The inclusion of both plant condition and radiological effluent EALs more fully addresses the spectrum of possible accident events and conditions.

~~Classification based on effluent monitor readings assumes that a release path to the environment is established. If the effluent flow past an effluent monitor is known to have stopped due to actions to isolate the release path, then the effluent monitor reading is no longer valid for classification purposes.~~

Releases should not be prorated or averaged. For example, a release exceeding 4 times release limits for 30 minutes does not meet the EAL.

The radwaste liquid discharge system is currently closed off with a plant modification installed blank flange. To perform a discharge would require a plant modification to remove the flange. Since the liquid radwaste system is not operable, no EAL threshold has been developed for this release point.

~~EAL #2—This EAL addresses radioactivity releases that cause effluent radiation monitor readings to exceed 2 times the limit established by a radioactivity discharge permit. This EAL will typically be associated with planned batch releases from non-continuous release pathways (e.g., radwaste, waste gas).~~

EAL #1 Basis:

~~EAL #1—This EAL addresses normally occurring continuous radioactivity releases from monitored gaseous or liquid effluent pathways.~~

Table OCGS 3-2 OCGS EAL Technical Basis

RECOGNITION CATEGORY

ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS

EAL #2 Basis:

~~EAL #3~~—This EAL addresses uncontrolled gaseous or liquid releases that are detected by sample analyses or environmental surveys, particularly on unmonitored pathways (e.g., spills of radioactive liquids into storm drains, heat exchanger leakage in river water systems, etc.).

Escalation of the emergency classification level would be via IC **RAA1**.

Basis Reference(s):

1. NEI 99-01 Rev 6, AU1
2. EP-EAL-0610, Criteria for Choosing Radiological Gaseous Effluent EAL Threshold Oyster Creek Generating Station
3. BNE Correspondence dated February 1, 2007
4. CY-OC-170-301, Offsite Dose Calculation Manual for Oyster Creek
5. ABN-27, Inadvertent Overboard Radioactive Release or Cross Contamination

Table OCGS 3-2 OCGS EAL Technical Basis

RECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS

ARA2

Initiating Condition:

Significant lowering of water level above, or damage to, irradiated fuel.

Operating Mode Applicability:

1, 2, 3, 4, D

Emergency Acton Level (EAL):

- ~~(1) — Uncovery of irradiated fuel in the REFUELING PATHWAY.~~
- ~~(2) — Damage to irradiated fuel resulting in a release of radioactivity from the fuel as indicated by **ANY** of the following radiation monitors:

—— (site specific listing of radiation monitors, and the associated readings, setpoints and/or alarms)~~
- ~~(3) — Lowering of spent fuel pool level to (site specific Level 2 value). [See Developer Notes]~~

1. Uncovery of irradiated fuel in the REFUELING PATHWAY.

OR

2. Damage to irradiated fuel resulting in a release of radioactivity from the fuel as indicated by **ANY** Table R2 Radiation Monitor reading ≥ 1000 mRem/hr

Table R2 Refuel Floor ARM's
<ul style="list-style-type: none"> • C-5, Crit Mon • C-10, North Wall • C-9, North Wall • B-9, Open Floor

Table OCGS 3-2 OCGS EAL Technical Basis

RECOGNITION CATEGORY

ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS

Basis:

REFUELING PATHWAY: all the cavities, tubes, canals and pools through which irradiated fuel may be moved or stored, but not including the reactor vessel below the flange.

IMMINENT: The trajectory of events or conditions is such that an EAL will be met within a relatively short period of time regardless of mitigation or corrective actions.

CONFINEMENT BOUNDARY: The irradiated fuel dry storage cask barrier(s) between areas containing radioactive substances and the environment.

This IC addresses events that have caused IMMEDIATE or actual damage to an irradiated fuel assembly, ~~or a significant lowering of water level within the spent fuel pool (see Developer Notes).~~ These events present radiological safety challenges to plant personnel and are precursors to a release of radioactivity to the environment. As such, they represent an actual or potential substantial degradation of the level of safety of the plant.

This IC applies to irradiated fuel that is licensed for dry storage up to the point that the loaded storage cask is sealed. Once sealed, damage to a loaded cask causing loss of the CONFINEMENT BOUNDARY is classified in accordance with IC E-HU1.

EAL #1 Basis:**EAL #1**

This EAL escalates from RAU2 in that the loss of level, in the affected portion of the REFUELING PATHWAY, is of sufficient magnitude to have resulted in uncover of irradiated fuel. Indications of irradiated fuel uncover may include direct or indirect visual observation (e.g., reports from personnel or camera images), as well as significant changes in water and radiation levels, or other plant parameters. Computational aids may also be used (e.g., a boil-off curve). Classification of an event using this EAL should be based on the totality of available indications, reports and observations.

While an area radiation monitor could detect an **increase** in a dose rate due to a lowering of water level in some portion of the REFUELING PATHWAY, the reading may not be a reliable indication of whether or not the fuel is actually uncovered. To the degree possible, readings should be considered in combination with other available indications of inventory loss.

A drop in water level above irradiated fuel within the reactor vessel may be classified in accordance Recognition Category C during the Cold Shutdown and Refueling modes.

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS**EAL #2 Basis:****EAL #2**

This EAL addresses a release of radioactive material caused by mechanical damage to irradiated fuel. Damaging events may include the dropping, bumping or binding of an assembly, or dropping a heavy load onto an assembly. A rise in readings on radiation monitors should be considered in conjunction with in-plant reports or observations of a potential fuel damaging event (e.g., a fuel handling accident).

Escalation of the emergency would be based on either Recognition Category RA or C ICs.

EAL #3

~~Spent fuel pool water level at this value is within the lower end of the level range necessary to prevent significant dose consequences from direct gamma radiation to personnel performing operations in the vicinity of the spent fuel pool. This condition reflects a significant loss of spent fuel pool water inventory and thus it is also a precursor to a loss of the ability to adequately cool the irradiated fuel assemblies stored in the pool.~~

~~Escalation of the emergency classification level would be via ICs AS1 or AS2 (see AS2 Developer Notes).~~

Basis Reference(s):

1. NEI 99-01 Rev 6, AA2
2. RAP G-7-a, SKM SRG TNK LVL LO-LO
3. RAP-10F-1-m, Crit Mon C5 Hi
4. RAP-10F-3-m, North Wall C9 Hi Vent Trip
5. RAP-10F-2-m, North Wall C10 Hi
6. RAP-10F-4-m, North Wall B9 Hi Vent Trip

Table OCGS 3-2 OCGS EAL Technical Basis

RECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS

ARU2

Initiating Condition:

UNPLANNED loss of water level above irradiated fuel.

Operating Mode Applicability:

1, 2, 3, 4, D

Emergency Acton Level (EAL):

~~(1) a. UNPLANNED water level drop in the REFUELING PATHWAY as indicated by ANY of the following:~~

~~_____ (site-specific level indications)._____~~

~~_____ **AND**~~

~~b. UNPLANNED rise in area radiation levels as indicated by ANY of the following radiation monitors.~~

~~_____ (site-specific list of area radiation monitors)~~

1. a. UNPLANNED water level drop in the REFUELING PATHWAY as indicated by **ANY** of the following:

- Refueling Cavity water level < **583 inches** (GEMAC Wide Range, floodup calibration).

OR

- Indication or report of a drop in water level in the REFUELING PATHWAY.

AND

b. UNPLANNED Area Radiation Monitor reading rise on **ANY** radiation monitors in Table R2.

Table R2 Refuel Floor ARM's
<ul style="list-style-type: none"> • C-5, Crit Mon • C-10, North Wall • C-9, North Wall • B-9, Open Floor

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS****Basis:**

UNPLANNED: A parameter change or an event that is not 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

REFUELING PATHWAY: all the cavities, tubes, canals and pools through which irradiated fuel may be moved or stored, but not including the reactor vessel below the flange.

This IC addresses a **decrease-loss** in water level above irradiated fuel sufficient to cause elevated radiation levels. This condition could be a precursor to a more serious event and is also indicative of a minor loss in the ability to control radiation levels within the plant. It is therefore a potential degradation in the level of safety of the plant.

A water level **decrease-loss** will be primarily determined by indications from available level instrumentation. Other sources of level indications may include reports from plant personnel (e.g., from a refueling crew) or video camera observations (if available) **or from any other temporarily installed monitoring instrumentation**. A significant drop in the water level may also cause an **increaserise** in the radiation levels of adjacent areas that can be detected by monitors in those locations.

The effects of planned evolutions should be considered. For example, a refueling bridge area radiation monitor reading may **increaserise** due to planned evolutions such as lifting of the reactor vessel head or movement of a fuel assembly. Note that this EAL is applicable only in cases where the elevated reading is due to an UNPLANNED loss of water level.

A drop in water level above irradiated fuel within the reactor vessel may be classified in accordance Recognition Category C during the Cold Shutdown and Refueling modes.

Escalation of the emergency classification level would be via IC **RAA2**.

Basis Reference(s):

1. NEI 99-01 Rev 6, AU2
2. RP-AA-203 Exposure Control and Authorization
3. RAP-G-7-a, SKM SRG TNK LVL LO-LO
4. 205.94.0 RPV Floodup Using Core Spray
5. 205.95.0 Reactor Flood-up / Drain-down
6. FSAR Figure 7.6-3

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS**

ARA3

Initiating Condition:

Radiation levels that impede access to equipment necessary for normal plant operations, cooldown or shutdown.

Operating Mode Applicability:

1, 2, 3, 4, D

Emergency Acton Level (EAL):

Note: If the equipment in the ~~listed~~ room or area ~~listed in Table R4~~ was already inoperable, or out of service, before the event occurred, then no emergency classification is warranted

~~(1) Dose rate greater than 15 mR/hr in ANY of the following areas:~~

- ~~• Control Room~~
- ~~• Central Alarm Station~~
- ~~• (other site specific areas/rooms)~~

~~(2) An UNPLANNED event results in radiation levels that prohibit or impede access to any of the following plant rooms or areas:~~

~~(site specific list of plant rooms or areas with entry related mode applicability identified)~~

1. Dose rate greater than > 15 mR/hr in ANY of the following Table R3 areas:

Table R3 Areas Requiring Continuous Occupancy
<ul style="list-style-type: none"> • Main Control Room • Central Alarm Station – (by survey)

OR

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS**

2. UNPLANNED event results in radiation levels that prohibit or significantly impede access to any of the following Table R4 plant rooms or areas:

Table R4 Areas with Entry Related Mode Applicability	
Area	Entry Related Mode Applicability
Reactor Building*	Modes 3 and 4
*Areas required to establish shutdown cooling	

Basis:

UNPLANNED: A parameter change or an event that is not 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

This IC addresses elevated radiation levels in certain plant rooms/areas sufficient to preclude or impede personnel from performing actions necessary to **transition the plant from normal plant operation to cooldown and shutdown as specified in normal plant procedures**~~maintain normal plant operation, or to perform a normal plant cooldown and shutdown~~. As such, it represents an actual or potential substantial degradation of the level of safety of the plant. The Emergency Director should consider the cause of the increased radiation levels and determine if another IC may be applicable.

Table R4 is a list of plant rooms or areas with entry-related mode applicability that contain equipment which require a manual/local action necessary to transition the plant from normal plant operation to cooldown and shutdown as specified in normal operating procedures (establish shutdown cooling), where if this action is not completed the plant would not be able to attain and maintain cold shutdown.

This Table does not include rooms or areas for which entry is required solely to perform actions of an administrative or record keeping nature (e.g., normal rounds or routine inspections).

Rooms and areas listed in EAL #1 do not need to be included in EAL #2, including the Control Room.

For EAL #2, an Alert declaration is warranted if entry into the affected room/area is, or may be, procedurally required during the plant operating mode in effect **and the elevated radiation levels preclude the ability to place shutdown cooling in service at the time of the elevated radiation levels**. The emergency classification is not contingent upon whether entry is actually necessary at the time of the increased radiation levels. Access should be

Table OCGS 3-2 OCGS EAL Technical Basis

RECOGNITION CATEGORY

ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS

considered as impeded if extraordinary measures are necessary to facilitate entry of personnel into the affected room/area (e.g., installing temporary shielding, requiring use of non-routine protective equipment, requesting an extension in dose limits beyond normal administrative limits).

An emergency declaration is not warranted if any of the following conditions apply.

- The plant is in an operating mode different than the mode specified for the affected room/area (i.e., entry is not required during the operating mode in effect at the time of the elevated radiation levels). For example, the plant is in Mode 1 when the radiation **increaserise** occurs, and the procedures used for normal operation, cooldown and shutdown do not require entry into the affected room until Mode 4.
- The increased radiation levels are a result of a planned activity that includes compensatory measures which address the temporary inaccessibility of a room or area (e.g., radiography, spent filter or resin transfer, etc.).
- The action for which room/area entry is required is of an administrative or record keeping nature (e.g., normal rounds or routine inspections).
- The access control measures are of a conservative or precautionary nature, and would not actually prevent or impede a required action.

Escalation of the emergency classification level would be via Recognition Category **RA**, C or F ICs.

Basis Reference(s):

1. NEI 99-01 Rev 6, AA3
2. ABN-29, Plant Fires
3. EMG-3200.11, Secondary Containment Control Safe Shutdown Area

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
ABNORMAL RAD LEVELS / RADIOLOGICAL EFFLUENTS**SRU3****Initiating Condition:**

Reactor coolant activity greater than Technical Specification allowable limits.

Operating Mode Applicability:

1, 2

Emergency Acton Level (EAL):

- ~~(1) — (Site-specific radiation monitor) reading greater than (site-specific value).~~
- ~~(2) — Sample analysis indicates that a reactor coolant activity value is greater than an allowable limit specified in Technical Specifications.~~

1. Offgas system radiation monitor **Hi-Hi** alarm.

OR

2. Specific coolant activity > **4.0 uCi/gm** Dose equivalent I-131.

Basis:

This IC addresses a reactor coolant activity value that exceeds an allowable limit specified in Technical Specifications. This condition is a precursor to a more significant event and represents a potential degradation of the level of safety of the plant.

Conditions that cause the specified monitor to alarm that are not related to fuel clad degradation should not result in the declaration of an Unusual Event.

This EAL addresses site-specific radiation monitor readings that provide indication of a degradation of fuel clad integrity.

An Unusual Event is only warranted when actual fuel clad damage is the cause of the elevated coolant sample activity (as determined by laboratory confirmation). Fuel clad damage should be assumed to be the cause of elevated Reactor Coolant activity unless another cause is known.

Escalation of the emergency classification level would be via ICs FA1 or the Recognition Category **RA** ICs.

Basis Reference(s):

1. NEI 99-01 Rev 6, SU3
2. Technical Specifications 3.6.A
3. ABN-26, High Main Steam Line or Off Gas Activity
4. RAP10F-1-c, Offgas HI-HI

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

FG1

Initiating Condition:

Loss of ANY Two Barriers AND Loss or Potential Loss of the third barrier.

Operating Mode Applicability:

1, 2

Emergency Acton Level (EAL):

Refer to Fission Product Barrier Loss and Potential Loss threshold values to determine barrier status.

Basis:

Fuel Cladding, RCS and Containment comprise the fission product barriers.

At the General Emergency classification level each barrier is weighted equally.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

FS1

Initiating Condition:

Loss or Potential Loss of ANY two barriers.

Operating Mode Applicability:

1, 2

Emergency Acton Level (EAL):

Refer to Fission Product Barrier Loss and Potential Loss threshold values to determine barrier status.

Basis:

Fuel Cladding, RCS and Containment comprise the fission product barriers.

At the Site Area Emergency classification level, each barrier is weighted equally.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION****FA1****Initiating Condition:**

ANY Loss or ANY Potential Loss of either Fuel Clad or RCS.

Operating Mode Applicability:

1, 2

Emergency Acton Level (EAL):

Refer to Fission Product Barrier Loss and Potential Loss threshold values to determine barrier status.

Basis:

Fuel Cladding, RCS and Containment comprise the fission product barriers.

At the Alert classification level, Fuel Cladding and RCS barriers are weighted more heavily than the Containment barrier. Unlike the Containment barrier, loss or potential loss of either the Fuel Cladding or RCS barrier may result in the relocation of radioactive materials or degradation of core cooling capability. Note that the loss or potential loss of Containment barrier in combination with loss or potential loss of either Fuel Cladding or RCS barrier results in declaration of a Site Area Emergency under EAL FS1.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION

FC1

Initiating Condition:

RCS Activity

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:LOSS

~~A. (Site-Specific indications that reactor coolant activity is greater than 300uCi/gm dose equivalent I-131)Coolant activity > 300 uCi/gm Dose Equivalent I-131.~~

Basis:

This threshold indicates that RCS radioactivity concentration is greater than 300 $\mu\text{Ci/gm}$ dose equivalent I-131. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to an approximate range of 2% to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier.

It is recognized that sample collection and analysis of reactor coolant with highly elevated activity levels could require several hours to complete. Nonetheless, a sample-related threshold is included as a backup to other indications.

There is no Potential Loss threshold associated with RCS Activity.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

FC2**Initiating Condition:**

RPV Water Level

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:LOSS

~~A-1. Plant conditions indicate Primary-primary~~ containment flooding is required.

POTENTIAL LOSS

~~A-2. RPV water level **cannot** be restored and maintained ~~above (site-specific RPV water level corresponding to the top of active fuel)~~ > 0 inches TAF.~~

~~or-OR~~

3. RPV water level **cannot** be determined.

Basis:Loss ~~2-A~~Threshold #1 Basis

The Loss threshold represents the EOP requirement for primary containment flooding. This is identified in the BWROG EPGs/SAGs when the phrase, "Primary Containment Flooding Is Required," appears. Since a site-specific RPV water level is not specified here, the Loss threshold phrase, "Primary containment flooding required," also accommodates the EOP need to flood the primary containment when RPV water level cannot be determined and core damage due to inadequate core cooling is believed to be occurring.

Potential Loss ~~2-A~~Threshold #2 and #3 Basis:

This water level corresponds to the top of the active fuel and is used in the EOPs to indicate a challenge to core cooling.

The RPV water level threshold is the same as RCS ~~barrier-Barrier RC2~~ Loss threshold ~~2-A~~. Thus, this threshold indicates a Potential Loss of the Fuel Clad barrier and a Loss of the RCS barrier that appropriately escalates the emergency classification level to a Site Area Emergency.

This threshold is considered to be exceeded when, as specified in the site-specific EOPs, RPV water level cannot be restored and maintained above the specified level following depressurization of the RPV (either manually, automatically or by failure of the RCS barrier) or when procedural guidance or a lack of low pressure RPV injection sources preclude Emergency RPV depressurization. EOPs allow the operator a wide choice of RPV injection sources to consider when restoring RPV water level to within prescribed limits. EOPs also specify depressurization of the RPV in order to facilitate RPV water level control with low-pressure injection sources. In some events, elevated

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

RPV pressure may prevent restoration of RPV water level until pressure drops below the shutoff heads of available injection sources. Therefore, this Fuel Clad barrier Potential Loss is met only after either: 1) the RPV has been depressurized, or required emergency RPV depressurization has been attempted, giving the operator an opportunity to assess the capability of low-pressure injection sources to restore RPV water level or 2) no low pressure RPV injection systems are available, precluding RPV depressurization in an attempt to minimize loss of RPV inventory.

The term "cannot be restored and maintained above" means the value of RPV water level is not able to be brought above the specified limit (top of active fuel). The determination requires an evaluation of system performance and availability in relation to the RPV water level value and trend. A threshold prescribing declaration when a threshold value *cannot* be restored and maintained above a specified limit does not require immediate action simply because the current value is below the top of active fuel, but does not permit extended operation below the limit; the threshold must be considered reached as soon as it is apparent that the top of active fuel cannot be attained.

In high-power ATWS/failure to scram events, EOPs may direct the operator to deliberately lower RPV water level ~~to the top of active fuel~~ in order to reduce reactor power. ~~RPV water level is then controlled between the top of active fuel and the Minimum Steam Cooling RPV Water Level (MSCRWL).~~ Although such action is a challenge to core cooling and the Fuel Clad barrier, the immediate need to reduce reactor power is the higher priority. For such events, ICs ~~SA5-MA3~~ or ~~SS5-MS3~~ will dictate the need for emergency classification.

Since the loss of ability to determine if adequate core cooling is being provided presents a significant challenge to the fuel clad barrier, a potential loss of the fuel clad barrier is specified.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2
2. EMG-3200.01A, RPV Control – No ATWS
3. EMG-3200.01B, RPV Control – With ATWS
4. EMG-3200.08A, RPV Flooding – No ATWS
5. EMG-3200.08B, RPV Flooding – With ATWS
6. EMG-3200.02, Primary Containment Control

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION

FC5

Initiating Condition:

Primary Containment Radiation

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:LOSS

~~A. Primary containment radiation monitor reading greater than (site specific value)~~
Containment Hi Range Radiation Monitoring System (CHRRMS) reading > 530 R/hr.

Basis:~~Loss 4.A~~

The radiation monitor reading corresponds to an instantaneous release of all reactor coolant mass into the primary containment, assuming that reactor coolant activity equals 300 $\mu\text{Ci/gm}$ dose equivalent I-131. Reactor coolant activity above this level is greater than that expected for iodine spikes and corresponds to an approximate range of 2% to 5% fuel clad damage. Since this condition indicates that a significant amount of fuel clad damage has occurred, it represents a loss of the Fuel Clad Barrier.

The radiation monitor reading in this threshold is higher than that specified for RCS Barrier ~~RC5 Loss T~~ threshold ~~4.A~~ since it indicates a loss of both the Fuel Clad Barrier and the RCS Barrier. Note that a combination of the two monitor readings appropriately escalates the emergency classification level to a Site Area Emergency.

There is no ~~Fuel Clad Barrier~~ Potential Loss threshold associated with Primary Containment Radiation.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2
2. Core Damage Assessment Methodology

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION

FC7

Initiating Condition:

Emergency Director Judgment.

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:LOSS

1A. Any condition in the opinion of the Emergency Director that indicates Loss of the Fuel Clad Barrier.

POTENTIAL LOSS

2A. Any condition in the opinion of the Emergency Director that indicates Potential Loss of the Fuel Clad Barrier.

Basis:**Loss Threshold #1 Basis**Loss 6.A

This threshold addresses any other factors that are to be used by the Emergency Director in determining whether the Fuel Clad Barrier is lost.

Potential Loss Threshold #2 BasisPotential Loss 6.A

This threshold addresses any other factors that may be used by the Emergency Director in determining whether the Fuel Clad Barrier is potentially lost. The Emergency Director should also consider whether or not to declare the barrier potentially lost in the event that barrier status cannot be monitored.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

RC2**Initiating Condition:**

RPV Water Level

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:LOSS

1. RPV water level **cannot** be restored and maintained ~~above (site-specific RPV water level corresponding to the top of active fuel)~~ **> 0 inches TAF**

~~or~~ **OR**

2. RPV water level **cannot** be determined.

Basis:Loss 2.A

This water level corresponds to the ~~top-Top~~ of ~~active-Active fuel-Fuel~~ (TAF) and is used in the EOPs to indicate challenge to core cooling.

The RPV water level threshold is the same as Fuel Clad ~~barrier-Barrier~~ **FC2** Potential Loss threshold ~~2.A~~. Thus, this threshold indicates a Loss of the RCS barrier and Potential Loss of the Fuel Clad barrier and that appropriately escalates the emergency classification level to a Site Area Emergency.

This threshold is considered to be exceeded when, as specified in the site-specific EOPs, RPV water **level** cannot be restored and maintained above the specified level following depressurization of the RPV (either manually, automatically or by failure of the RCS barrier) or when procedural guidance or a lack of low pressure RPV injection sources preclude Emergency RPV depressurization. EOPs allow the operator a wide choice of RPV injection sources to consider when restoring RPV water level to within prescribed limits. EOPs also specify depressurization of the RPV in order to facilitate RPV water level control with low-pressure injection sources. In some events, elevated RPV pressure may prevent restoration of RPV water level until pressure drops below the shutoff heads of available injection sources. Therefore, this RCS barrier Loss is met only after either: 1) the RPV has been depressurized, or required emergency RPV depressurization has been attempted, giving the operator an opportunity to assess the capability of low-pressure injection sources to restore RPV water level or 2) no low pressure RPV injection systems are available, precluding RPV depressurization in an attempt to minimize loss of RPV inventory.

The term, "cannot be restored and maintained above," means the value of RPV water level is not able to be brought above the specified limit (top of active fuel). The determination requires an evaluation of system performance and availability in relation to the RPV water level value and trend. A threshold prescribing declaration when a threshold value *cannot* be restored and maintained above a specified limit does not

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

require immediate action simply because the current value is below the top of active fuel, but does not permit extended operation beyond the limit; the threshold must be considered reached as soon as it is apparent that the top of active fuel cannot be attained.

In high-power ATWS/failure to scram events, EOPs may direct the operator to deliberately lower RPV water level ~~to the top of active fuel~~ in order to reduce reactor power. ~~RPV water level is then controlled between the top of active fuel and the Minimum Steam Cooling RPV Water Level (MSCRWL).~~ Although such action is a challenge to core cooling and the Fuel Clad barrier, the immediate need to reduce reactor power is the higher priority. For such events, ICs **SA5-MA3** or **SS5-MS3** will dictate the need for emergency classification.

There is no RCS Potential Loss threshold associated with RPV Water Level.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2
2. 2000-GLN-3200.01, Plant Specific Technical Guideline
3. 2000-BAS-3200.02, EOP Users Guide

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION

RC3

Initiating Condition:

Primary Containment Pressure

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:LOSS~~A. Primary containment pressure greater than (site specific value) due to RCS leakage.~~

1. Drywell pressure > 3.0 psig.

AND

2. Drywell pressure rise is due to RCS leakage

Basis:Loss 1.A

The ~~(site specific value)~~> 3.0 psig primary containment pressure is the ~~drywell~~Drywell high pressure setpoint which indicates a LOCA by automatically initiating ~~the ECCS or equivalent makeup system.~~

The second threshold condition focuses the fission product barrier loss threshold on a failure of the RCS instead of the non-LOCA malfunctions that may adversely affect primary containment pressure. Pressures of this magnitude can be caused by non-LOCA events such as a loss of Drywell cooling or inability to control primary containment vent/purge.

The release of mass from the RCS due to the as-designed/expected operation of any relief valve does not warrant an emergency classification.

A stuck-open Electromatic Relief Valve (EMRV) or EMRV leakage is not considered either identified or unidentified leakage by Technical Specifications and, therefore, is not applicable to this EAL.

There is no Potential Loss threshold associated with Primary Containment Pressure.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2
2. EMG-3200.01A, RPV Control – No ATWS
3. EMG-3200.02, Primary Containment Control
4. 2000-BAS-3200.02, EOP User's Guide

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

RC4**Initiating Condition:**

RCS Leak Rate

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:LOSS

A1. UNISOLABLE Main Steam Line (MSL), Isolation Condenser, Feedwater, or RWCU line break. in ~~ANY of the following: (site-specific systems with potential for high energy line breaks)~~

OR

B2. Emergency RPV Depressurization is required.

POTENTIAL LOSS

3A. UNISOLABLE primary system leakage that results in EITHER of the following:

a. Secondary Containment area temperature > EMG-3200.11 Max Normal (Table 11) operating level.

OR

b. Secondary Containment area radiation level > EMG-3200.11 Max Normal (Table 12) operating level.

~~1. Max Normal Operating Temperature~~

OR

~~2. Max Normal Operating Area Radiation Level.~~

Basis:

UNISOLABLE: An open or breached system line that cannot be isolated, remotely or locally.

Classification of a system break over system leakage is based on information available to the Control Room from the event. Indications that should be considered are:

- Reports describing magnitude of steam or water release.
- Use of system high flow alarms / indications, if available,
- Significant changes in makeup requirements,
- Abnormal reactor water level changes in response to the event.

The use of the above indications provides the Control Room the bases to determine that the on going event is more significant than the indications that would be expected from system leakage and therefore should be considered a system break.

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**Loss Threshold #1 Basis-3.A**

Large high-energy lines that rupture outside primary containment can discharge significant amounts of inventory and jeopardize the pressure-retaining capability of the RCS until they are isolated. If it is determined that the ruptured line cannot be promptly isolated ~~from the Control Room~~, the RCS barrier Loss threshold is met.

Loss Threshold #2 Basis-3.B

Emergency RPV Depressurization in accordance with the EOPs is indicative of a loss of the RCS barrier. If Emergency RPV Depressurization is performed, the plant operators are directed to open ~~safety-Electromatic~~ relief valves (SRVsEMRVs) and keep them open. Even though the RCS is being vented into the ~~suppression pooltorus~~, a Loss of the RCS barrier exists due to the diminished effectiveness of the RCS to retain fission products within its boundary.

Potential Loss Threshold- #3 Basis 3.A

Potential loss of RCS based on primary system leakage outside the primary containment is determined from EOP temperature or radiation Max Normal Operating values in areas such as ~~main steam line tunnel~~Trunnion room, Isolation Condenser, RWCU RCIC, HPCI, etc., which indicate a direct path from the RCS to areas outside primary containment.

A Max Normal Operating value is the highest value of the identified parameter expected to occur during normal plant operating conditions with all directly associated support and control systems functioning properly.

The indicators reaching the threshold barriers and confirmed to be caused by RCS leakage from a primary system warrant an Alert classification. A primary system is defined to be the pipes, valves, and other equipment which connect directly to the RPV such that a reduction in RPV pressure will effect a decrease in the steam or water being discharged through an unisolated break in the system.

In general, multiple indications should be used to determine if a primary system is discharging outside Primary Containment. For example, a high area radiation condition does not necessarily indicate that a primary system is discharging into the Reactor Building since this may be caused by radiation shine from nearby steam lines or the movement of radioactive materials. Conversely, a high area radiation condition in conjunction with other indications (e.g. room flooding, high area temperatures, reports of steam in the Reactor Building, an unexpected rise in Feedwater flowrate, or unexpected Main Turbine Control Valve closure) may indicate that a primary system is discharging into the Reactor Building.

An UNISOLABLE leak which is indicated by Max Normal Operating values escalates to a Site Area Emergency when combined with Containment Barrier CT6 Loss Tthreshold #13.A (after a containment isolation) and a General Emergency when the Fuel Clad Barrier criteria is also exceeded.

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2
2. EMG-3200.11, Secondary Containment Control
3. 2000-GLN-3200.01, Plant Specific Technical Guideline

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION

RC5

Initiating Condition:

Primary Containment radiation

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:LOSS

Containment Hi Range Radiation Monitoring System (CHRRMS) reading > 100R/hr.

~~A. Primary containment radiation reading greater than (site specific value).~~**Basis:**Loss 4.A

The radiation monitor reading corresponds to an instantaneous release of all reactor coolant mass into the primary containment, assuming that reactor coolant activity equals Technical Specification allowable limits. This value is lower than that specified for Fuel Clad Barrier FC5 Loss Threshold ~~4.A~~ since it indicates a loss of the RCS Barrier only.

There is no **Reactor Coolant System** Potential Loss threshold associated with Primary Containment Radiation.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2
2. EP-EAL-0611, Criteria for Choosing Containment Radiation Monitor Reading Indicative of Loss of RCS Barrier

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION

RC7

Initiating Condition:

Emergency Director Judgment.

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:LOSS

A1. Any condition in the opinion of the Emergency Director that indicates Loss of the RCS Barrier.

POTENTIAL LOSS

A2. Any condition in the opinion of the Emergency Director that indicates Potential Loss of the RCS Barrier.

Basis:**Loss 6.A Threshold #1 Basis:**

This threshold addresses any other factors that are to be used by the Emergency Director in determining whether the RCS Barrier is lost.

Potential Loss 6.A Threshold #2 Basis:

This threshold addresses any other factors that may be used by the Emergency Director in determining whether the RCS Barrier is potentially lost. The Emergency Director should also consider whether or not to declare the barrier potentially lost in the event that barrier status cannot be monitored.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION

CT2

Initiating Condition:

RPV Water Level

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:POTENTIAL LOSS

A. Plant conditions indicate **Primary-primary** containment flooding is required.

Basis:Potential Loss 2.A

The Potential Loss threshold is identical to the Fuel Clad **Barrier FC2** Loss threshold RPV Water Level **2.A**. The Potential Loss requirement for Primary Containment Flooding indicates adequate core cooling cannot be restored and maintained and that core damage is possible. BWR **EPGsEOPs/SAMGs** specify the conditions that require primary containment flooding. When primary containment flooding is required, the **EPGsEOPs** are exited and **SAMGs** are entered. Entry into **SAMGs** is a logical escalation in response to the inability to restore and maintain adequate core cooling.

PRA studies indicate that the condition of this Potential Loss threshold could be a core melt sequence which, if not corrected, could lead to RPV failure and increased potential for primary containment failure. In conjunction with the RPV water level Loss thresholds in the Fuel Clad and RCS barrier columns, this threshold results in the declaration of a General Emergency.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2
2. EMG-3200.01B, RPV Control – With ATWS
3. EMG-3200.08A, RPV Flooding – No ATWS
4. EMG-3200.08B, RPV Flooding – With ATWS
5. EMG-3200.02, Primary Containment Control6. EMG-3200.01A, RPV Control – No ATWS

Table OCGS 3-2 OCGS EAL Technical Basis

RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION

CT3

Initiating Condition:

Primary Containment Conditions

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:LOSS

A1. UNPLANNED rapid drop in ~~primary containment~~Drywell pressure following ~~primary containment pressure~~Drywell rise.

OR

B2. ~~Primary containment~~Drywell pressure response **not** consistent with LOCA conditions.

POTENTIAL LOSS

A3. ~~Primary c~~Drywell~~ontainment~~ pressure ~~greater than (site specific value)~~ **> 44 psig** and rising.

OR

B4. ~~(site specific explosive mixture) exists inside primary containment~~ a. Drywell or Torus Hydrogen concentration $\geq 6\%$.

AND

b. Drywell or Torus Oxygen concentration $\geq 5\%$.

OR

C5. HTLG-Heat Capacity Temperature Limit (EMG-3200.02 Fig. F) exceeded.

Basis:

UNPLANNED: A parameter change or an event that is not 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

Loss 1.A and 1.B Threshold #1 and #2 Basis

Rapid UNPLANNED loss of ~~primary containment~~Drywell pressure (i.e., not attributable to ~~drywell~~Drywell spray or condensation effects) following an initial pressure ~~increaserise~~ indicates a loss of ~~primary containment~~Drywell integrity. ~~Primary containment~~Drywell pressure should ~~increaserise~~ as a result of mass and energy release into the primary containment from a LOCA. Thus, ~~primary containment~~Drywell pressure not increasing under these conditions indicates a loss of ~~primary containment~~Drywell integrity.

These thresholds rely on operator recognition of an unexpected response for the condition and therefore a specific value is not assigned. The unexpected (UNPLANNED) response is important because it is the indicator for a containment

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

bypass condition. A pressure suppression bypass path would **not** be an indication of a containment breach.

Potential Loss ~~1.A~~Threshold #3 Basis

The threshold pressure is the ~~primary containment~~Drywell internal design pressure. Structural acceptance testing demonstrates the capability of the ~~primary containment~~Drywell to resist pressures greater than the internal design pressure. A pressure of this magnitude is greater than those expected to result from any design basis accident and, thus, represent a Potential Loss of the Containment barrier.

Potential Loss ~~1.B~~Threshold #4 Basis

If hydrogen concentration reaches or exceeds the lower flammability limit, as defined in plant EOPs, in an oxygen rich environment, a potentially explosive mixture exists. If the combustible mixture ignites inside the primary containment, loss of the Containment barrier could occur.

Potential Loss ~~1.C~~Threshold #5 Basis

~~The Heat Capacity Temperature Limit (HCTL) is the highest suppression pool temperature from which Emergency RPV Depressurization will not raise:~~

- ~~• Suppression chamber temperature above the maximum temperature capability of the suppression chamber and equipment within the suppression chamber which may be required to operate when the RPV is pressurized,~~

~~OR~~

- ~~• Suppression chamber pressure above Primary Containment Pressure Limit A, while the rate of energy transfer from the RPV to the containment is greater than the capacity of the containment vent.~~

The HCTL is a function of RPV pressure, ~~suppression pool~~torus temperature and ~~suppression pool~~torus water level. It is utilized to preclude failure of the containment and equipment in the containment necessary for the safe shutdown of the plant and therefore, the inability to maintain plant parameters below the limit constitutes a potential loss of containment.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2
2. FSAR Update 6.2.1.1.3
3. Technical Specifications 5.2 Basis
4. EMG-3200.02 Primary Containment Control

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION

CT5

Initiating Condition:

Primary Containment Radiation

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:POTENTIAL LOSS

~~A. Primary containment radiation monitor reading greater than (site specific value)~~
Containment Hi Range Radiation Monitoring System (CHRRMS) reading > 1210 R/hr.

Basis:

There is no Loss threshold associated with Primary Containment Radiation.

Potential Loss 4.A

The radiation monitor reading corresponds to an instantaneous release of all reactor coolant mass into the primary containment, assuming that 20% of the fuel cladding has failed. This level of fuel clad failure is well above that used to determine the analogous Fuel Clad Barrier Loss and RCS Barrier Loss thresholds.

NUREG-1228, *Source Estimations During Incident Response to Severe Nuclear Power Plant Accidents*, indicates the fuel clad failure must be greater than approximately 20% in order for there to be a major release of radioactivity requiring offsite protective actions. For this condition to exist, there must already have been a loss of the RCS Barrier and the Fuel Clad Barrier. It is therefore prudent to treat this condition as a potential loss of containment which would then escalate the emergency classification level to a General Emergency.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2
2. Core Damage Assessment Methodology

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

CT6

Initiating Condition:

Primary Containment Isolation Failure

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:LOSS

A1. UNISOLABLE direct downstream pathway to the environment exists after primary containment isolation signal.

OR

B2. Intentional Primary Containment venting/purging per EOP's or SAGs due to accident conditions.

OR

C3. UNISOLABLE primary system leakage that results in **EITHER** of the following:

4a. Secondary Containment area temperature > **EMG-3200.11 Max Safe** (Table 11) operating level. ~~Max Safe Operating Temperature~~

OR

2b. Secondary Containment area radiation level > **EMG-3200.11 Max Safe** (Table 12) operating level. ~~Max Safe Operating Radiation Level.~~

Basis:

UNISOLABLE: An open or breached system line that cannot be isolated, remotely or locally.

These thresholds address incomplete containment isolation that allows an UNISOLABLE direct release to the environment.

Loss 3.A Threshold #1 Basis

The use of the modifier "direct" in defining the release path discriminates against release paths through interfacing liquid systems or minor release pathways, such as instrument lines, not protected by the Primary Containment Isolation System (PCIS). Leakage into a closed system is to be considered only if the closed system is breached and thereby creates a significant pathway to the environment. Examples include unisolable Main Steamline, Isolation Condenser line breaks, unisolable RWCU system breaks, and unisolable containment atmosphere vent paths.

Examples of "downstream pathway to the environment" could be through the Turbine/Condenser, or direct release to the Turbine or Reactor Building.

The existence of a filter is not considered in the threshold assessment. Filters do not remove fission product noble gases. In addition, a filter could become ineffective due to iodine and/or particulate loading beyond design limits (i.e., retention ability has been exceeded) or water saturation from steam/high humidity in the release stream.

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

Following the leakage of RCS mass into primary containment and a rise in primary containment pressure, there may be minor radiological releases associated with allowable primary containment leakage through various penetrations or system components. Minor releases may also occur if a primary containment isolation valve(s) fails to close but the primary containment atmosphere escapes to an enclosed system. These releases do not constitute a loss or potential loss of primary containment but should be evaluated using the Recognition Category **A-R** ICs.

Loss 3.B Threshold #2 Basis

EOPs may direct primary containment isolation valve logic(s) to be intentionally bypassed, even if offsite radioactivity release rate limits will be exceeded. Under these conditions with a valid primary containment isolation signal, the containment should also be considered lost if primary containment venting is actually performed.

Intentional venting of primary containment for primary containment pressure or combustible gas control to the secondary containment and/or the environment is a Loss of the Containment. Venting for primary containment pressure control when not in an accident situation (e.g., to control pressure below the **drywell/Drywell** high pressure scram setpoint) does not meet the threshold condition.

Loss 3.C Threshold #3 Basis

The Max Safe Operating Temperature and the Max Safe Operating Radiation Level are each the highest value of these parameters at which neither: (1) equipment necessary for the safe shutdown of the plant will fail, nor (2) personnel access necessary for the safe shutdown of the plant will be precluded. EOPs utilize these temperatures and radiation levels to establish conditions under which RPV depressurization is required.

The temperatures and radiation levels should be confirmed to be caused by RCS leakage from a primary system. A primary system is defined to be the pipes, valves, and other equipment which connect directly to the RPV such that a reduction in RPV pressure will effect a decrease in the steam or water being discharged through an unisolated break in the system.

In general, multiple indications should be used to determine if a primary system is discharging outside Primary Containment. For example, a high area radiation condition does not necessarily indicate that a primary system is discharging into the Reactor Building since this may be caused by radiation shine from nearby steam lines or the movement of radioactive materials. Conversely, a high area radiation condition in conjunction with other indications (e.g. room flooding, high area temperatures, reports of steam in the Reactor Building, an unexpected rise in Feedwater flowrate, or unexpected Main Turbine Control Valve closure) may indicate that a primary system is discharging into the Reactor Building.

In combination with RCS **Barrier RC4 pPotential Loss Threshold #33-A** this threshold would result in a Site Area Emergency.

There is no Potential Loss threshold associated with Primary Containment Isolation Failure.

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2
2. 2000-GLN-3200.01, Plant Specific Technical Guideline
3. EMG-3200.02, Primary Containment Control
4. Support Procedures -32, -34, -41, -44
5. 2000-GLN-3200.03, OCGS Plant Specific Technical Guidelines for Severe Accident Guidelines
6. EMG-3200.11, Secondary Containment Control

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION

CT7

Initiating Condition:

Emergency Director Judgment.

Operating Mode Applicability:

1, 2

Fission Product Barrier (FPB) Threshold:LOSS

A1. Any condition in the opinion of the Emergency Director that indicates Loss of the Containment Barrier.

POTENTIAL LOSS

A2. Any condition in the opinion of the Emergency Director that indicates Potential Loss of the Containment Barrier.

Basis:**Loss 6.A Threshold #1 Basis:**

This threshold addresses any other factors that are to be used by the Emergency Director in determining whether the Containment Barrier is lost.

Potential Loss 6.A Threshold #2 Basis:

This threshold addresses any other factors that may be used by the Emergency Director in determining whether the Containment Barrier is potentially lost. The Emergency Director should also consider whether or not to declare the barrier potentially lost in the event that barrier status cannot be monitored.

Basis Reference(s):

1. NEI 99-01 Rev 6, Table 9-F-2

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
FISSION PRODUCT BARRIER DEGRADATION**

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**MSG1****Initiating Condition:**

Prolonged loss of all Off-site and all On-Site AC power to emergency busses.

Operating Mode Applicability:

1, 2

Emergency Action Level (EAL):

Note: The Emergency Director should declare the **General Emergency event** promptly upon determining that ~~(site-specific hours)~~ the applicable time has been exceeded, or will likely be exceeded.

1-a. Loss of **ALL** offsite ~~and ALL onsite~~ AC power to 4160V Buses 1C and 1D. ~~(site-specific emergency buses).~~

AND

2. Failure of EDG-1 and EDG-2 Emergency Diesel Generators to supply power to 4160V Buses 1C and 1D.

AND

3b. **EITHER** of the following:

a. Restoration of at least one 4160V ~~emergency bus~~ Bus (1C or 1D) -in **< 1 hours is not less than** ~~(site-specific hours)~~ is not likely.

OR

b. RPV water level **cannot** be restored and maintained **> -20 inches TAF.**
~~• (Site-specific indication of an inability to adequately remove heat from the core)~~

Basis:

SAFETY SYSTEM: A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related.

This IC addresses a prolonged loss of all power sources to AC emergency buses. A loss of all AC power compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink. A prolonged loss of these buses will lead to a loss of ~~one or more~~ any fission product

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**

barriers. In addition, fission product barrier monitoring capabilities may be degraded under these conditions.

The EAL should require declaration of a General Emergency prior to meeting the thresholds for IC FG1. This will allow additional time for implementation of offsite protective actions.

Escalation of the emergency classification from Site Area Emergency will occur if it is projected that power cannot be restored to at least one AC emergency bus by the end of the analyzed station blackout coping period. Beyond this time, plant responses and event trajectory are subject to greater uncertainty, and there is an increased likelihood of challenges to multiple fission product barriers.

The estimate for restoring at least one emergency bus should be based on a realistic appraisal of the situation. Mitigation actions with a low probability of success should not be used as a basis for delaying a classification upgrade. The goal is to maximize the time available to prepare for, and implement, protective actions for the public.

The EAL will also require a General Emergency declaration if the loss of AC power results in parameters that indicate an inability to adequately remove decay heat from the core.

Basis Reference(s):

1. NEI 99-01 Rev 6, SG1
2. UFSAR Section 8.2, Offsite Power System
3. ABN-37, Station Blackout
4. ABN-60, Grid Emergency
5. Regulatory Guide 1.155, Station Blackout
6. TDR-1099, "Station Blackout Evaluation Report"
7. 2000-BAS-3200.02, EOP User's Guide
8. 2000-GLN-3200.01, Plant Specific Technical Guideline
9. OCGS Drawing BR 3000
10. ABN-36, Loss of Off-Site Power

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**MSS1****Initiating Condition:**

Loss of all offsite and all onsite AC power to emergency busses for 15 minutes or longer.

Operating Mode Applicability:

1, 2

Emergency Action Level (EAL):

Note: The Emergency Director should declare the ~~Site Area Emergency event~~ promptly upon determining that ~~the applicable time 15 minutes~~ has been exceeded, or will likely be exceeded.

1. Loss of **ALL** offsite ~~and ALL onsite~~ AC Power to ~~(site specific emergency busses)~~ 4160V Buses 1C and 1D ~~for 15 minutes or longer.~~

AND

2. Failure of EDG-1 and EDG-2 Emergency Diesel Generators to supply power to 4160V Buses 1C and 1D.

AND

3. Failure to restore power to at least one 4160V Bus (1C or 1D) in **< 15 minutes** from the time of loss of both offsite and onsite AC power

Basis:

SAFETY SYSTEM: A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related.

This IC addresses a total loss of AC power that compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink. In addition, fission product barrier monitoring capabilities may be degraded under these conditions. This IC represents a condition that involves actual or likely major failures of plant functions needed for the protection of the public.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Escalation of the emergency classification level would be via ICs RAG1, FG1, ~~or~~ MSG1, or MG2.

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**

Basis Reference(s):

1. NEI 99-01 Rev 6, SS1
2. UFSAR Section 8.2, Offsite Power System
3. OCNGS Drawing BR 3000
4. ABN-36, Loss of Off-Site Power
5. ABN-37, Station Blackout
6. ABN-60, Grid Emergency

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**MSA1****Initiating Condition:**

Loss of all but one AC power source to emergency buses for 15 minutes or longer.

Operating Mode Applicability:

1, 2

Emergency Action Level (EAL):

Note: The Emergency Director should declare the ~~eventAlert~~ promptly upon determining that ~~the applicable time 15 minutes~~ has been exceeded, or will likely be exceeded.

1. ~~AC power capability to 4160V Buses 1C and 1D reduced to only one of the following power sources for ≥ 15 minutes.~~
 - Startup Transformer SA
 - Startup Transformer SB
 - EDG-1 Emergency Diesel Generator
 - EDG-2 Emergency Diesel Generator

~~a. AC power capability to (site specific emergency buses) is reduced to a single power source for 15 minutes or longer.~~

AND

- 2b. Any additional single power source failure will result in a loss of all AC power to SAFETY SYSTEMS.

Basis:

SAFETY SYSTEM: A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related.

This IC describes a significant degradation of offsite and onsite AC power sources such that any additional single failure would result in a loss of all AC power to SAFETY SYSTEMS. In this condition, the sole AC power source may be powering one, or more than one, train of safety-related equipment. This IC provides an escalation path from IC **MSU1**.

An "AC power source" is a source recognized in ~~AOPs-ABNs~~ and EOPs, and capable of supplying required power to an emergency bus. Some examples of this condition are presented below.

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**

- A loss of all offsite power with a concurrent failure of all but one emergency power source (e.g., an onsite diesel generator).
- A loss of all offsite power and loss of all emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being back-fed from the unit main generator.
- A loss of emergency power sources (e.g., onsite diesel generators) with a single train of emergency buses being back-fed from an offsite power source.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of power.

Escalation of the emergency classification level would be via IC **MSS1**.

Basis Reference(s):

1. NEI 99-01 Rev 6, SA1
2. UFSAR Section 8.2, Offsite Power System
2. OCGS Drawing BR 3000
3. ABN-36, Loss of Off-Site Power
4. ABN-37, Station Blackout
5. ABN-60, Grid Emergency

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**MSU1****Initiating Condition:**

Loss of all offsite AC power capability to emergency buses for 15 minutes or longer.

Operating Mode Applicability:

1, 2

Emergency Action Level (EAL):

Note: The Emergency Director should declare the ~~Unusual Event~~ promptly upon determining that ~~the applicable time 15 minutes~~ has been exceeded, or will likely be exceeded.

~~1. Loss of ALL offsite AC power capability to 4160V Buses 1C and 1D (site-specific emergency buses) for \geq 15 minutes or longer.~~

Basis:

This IC addresses a prolonged loss of offsite power. The loss of offsite power sources renders the plant more vulnerable to a complete loss of power to AC emergency buses. This condition represents a potential reduction in the level of safety of the plant.

For emergency classification purposes, "capability" means that an offsite AC power source(s) is available to the emergency buses, whether or not the buses are powered from it.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of offsite power.

Escalation of the emergency classification level would be via IC **MSA1**.

Basis Reference(s):

1. NEI 99-01 Rev 6, SU1
2. UFSAR Section 8.2, Offsite Power System
3. OCGS Drawing BR 3000
4. ABN-36, Loss of Off-Site Power
5. ABN-60, Grid Emergency

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**MSG28****Initiating Condition:**

Loss of all AC and Vital DC power sources for 15 minutes or longer.

Operating Mode Applicability:

1, 2

Emergency Action Level (EAL):

Note: The Emergency Director should declare the ~~General Emergency event~~ promptly upon determining that ~~the applicable time 15 minutes~~ has been exceeded, or will likely be exceeded.

1. Loss of **ALL** offsite AC power to 4160V Buses 1C and 1D.

AND

2. Failure of EDG-1 and EDG-2 Emergency Diesel Generators to supply power to 4160V Buses 1C and 1D.

AND

3. Voltage is **< 115 VDC** on 125 VDC battery busses B and C.

AND

4. **ALL** AC and Vital DC power sources have been lost for **≥ 15 minutes**.

~~1. a. Loss of **ALL** offsite and **ALL** onsite AC power to (site-specific emergency buses) for 15 minutes or longer.~~

AND

~~b. Indicated voltage is less than (site-specific bus voltage value) on **ALL** (site-specific Vital DC busses) for 15 minutes or longer.~~

Basis:

SAFETY SYSTEM: A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related.

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**

This IC addresses a concurrent and prolonged loss of both AC and Vital DC power. A loss of all AC power compromises the performance of all SAFETY SYSTEMS requiring electric power including those necessary for emergency core cooling, containment heat removal/pressure control, spent fuel heat removal and the ultimate heat sink. A loss of Vital DC power compromises the ability to monitor and control SAFETY SYSTEMS. A sustained loss of both AC and DC power will lead to multiple challenges to fission product barriers.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses. The 15-minute emergency declaration clock begins at the point when all EAL **conditions** are met.

Basis Reference(s):

1. NEI 99-01 Rev 6, SG8
2. UFSAR Section 8.3.2, DC Power Systems
3. UFSAR Section 8.2, Offsite Power System
4. OCGS Drawing BR 3000
5. ABN-36, Loss of Off-Site Power
6. ABN-37, Station Blackout
7. ABN-60, Grid Emergency
8. ABN-54, Loss of DC Distribution Center B
9. ABN-55, Loss of DC Distribution Center C

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**MSS28****Initiating Condition:**

Loss of all vital DC power for 15 minutes or longer.

Operating Mode Applicability:

1, 2

Emergency Action Level (EAL):

Note: The Emergency Director should declare the ~~Site Area Emergency event~~ promptly upon determining that ~~the applicable time 15 minutes~~ has been exceeded, or will likely be exceeded.

~~Indicated voltage is < 115 VDC less than (site-specific bus voltage value) on 125 VDC battery busses B and C ALL (site-specific Vital DC busses) for ≥ 15 minutes or longer.~~

Basis:

SAFETY SYSTEM: A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related.

This IC addresses a loss of Vital DC power which compromises the ability to monitor and control SAFETY SYSTEMS. In modes above Cold Shutdown, this condition involves a major failure of plant functions needed for the protection of the public.

Fifteen minutes was selected as a threshold to exclude transient or momentary power losses.

Escalation of the emergency classification level would be via ICs RAG1, FG1 or **MSG28**.

Basis Reference(s):

1. NEI 99-01 Rev 6, SS8
2. OCGS Drawing BR 3000
3. ABN-54, Loss of DC Distribution Center B
4. ABN-55, Loss of DC Distribution Center C

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**MSS35****Initiating Condition:**

Inability to shutdown the reactor causing a challenge to RPV water level or RCS heat removal.

Operating Mode Applicability:

1

Emergency Action Level (EAL):

1. Automatic scram did **not** shutdown the reactor **as indicated by Reactor Power > 2%**.

AND

2. **ALL** manual / **ARI** actions to shutdown the reactor have been unsuccessful **as indicated by Reactor Power > 2%**.

AND

3. EITHER of the following conditions exist:

- RPV water level **cannot** be restored and maintained **> -20 inches TAF**.
- OR**
- Heat Capacity Temperature Limit (EMG-3200.02 Fig. F) exceeded.

~~(Site-specific indication of an inability to adequately remove heat from the core)~~

~~(Site-specific indication of an inability to adequately remove heat from the RCS)~~

Basis:

This IC addresses a failure of the RPS to initiate or complete an automatic or manual reactor scram that results in a reactor shutdown, **all subsequent operator manual actions, both inside and outside the Control Room including driving in control rods and boron injection, all subsequent operator actions to manually shutdown the reactor** are unsuccessful, and continued power generation is challenging the capability to adequately remove heat from the core and/or the RCS. This condition will lead to fuel damage if additional mitigation actions are unsuccessful and thus warrants the declaration of a Site Area Emergency.

In some instances, the emergency classification resulting from this IC/EAL may be higher than that resulting from an assessment of the plant responses and symptoms against the Recognition Category F ICs/EALs. This is appropriate in that the Recognition Category F ICs/EALs do not address the additional threat posed by a failure to shutdown the reactor. The inclusion of this IC and EAL ensures the timely declaration of a Site Area Emergency in response to prolonged failure to shutdown the reactor.

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**

A reactor shutdown is determined in accordance with applicable Emergency Operating Procedure criteria.

EAL #3 is considered to be exceeded when, as specified in the site-specific EOPs, RPV water level cannot be restored and maintained above the specified level.

RPV values are actual levels, not indicated levels. Therefore, they may need level compensation depending on conditions.

Escalation of the emergency classification level would be via IC RAG1 or FG1.

Basis Reference(s):

1. NEI 99-01 Rev 6, SS5
2. EMG-3200.01B, RPV Control – with ATWS
3. EMG-3200.02, Primary Containment Control
4. 2000-BAS-3200.02, EOP User's Guide
5. 2000-GLN-3200.01, Plant Specific Technical Guideline
6. EMG-3200.01A, RPV Control – no ATWS

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**MSA35****Initiating Condition:**

Automatic or manual scram fails to shutdown the reactor, and subsequent manual actions taken at the reactor control consoles are not successful in shutting down the reactor.

Operating Mode Applicability:

1

Emergency Action Level (EAL):

Note: A manual action is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does not include manually driving in control rods or implementation of boron injection strategies.

1. ~~An~~ Automatic or manual scram did **not** shutdown the reactor **as indicated by Reactor Power > 2%**.

AND

2. Manual / ~~ARI~~ actions taken at the ~~reactor control consoles~~ **Reactor Console** are **not** successful in shutting down the reactor **as indicated by Reactor Power > 2%**.

Basis:

This IC addresses a failure of the RPS to initiate or complete an automatic or manual reactor scram that results in a reactor shutdown, and subsequent operator manual actions taken at the reactor ~~control~~-consoles to shutdown the reactor are also unsuccessful. This condition represents an actual or potential substantial degradation of the level of safety of the plant. An emergency declaration is required even if the reactor is subsequently shutdown by an action taken away from the reactor ~~control~~ consoles since this event entails a significant failure of the RPS.

A manual action at the reactor ~~control~~-consoles is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core (e.g., initiating a manual reactor scram. This action does not include manually driving in control rods or implementation of boron injection strategies. If this action(s) is unsuccessful, operators would immediately pursue additional manual actions at locations away from the reactor ~~control~~-consoles (e.g., locally opening breakers). Actions taken at back-panels or other locations within the Control Room, or any location outside the Control Room, are not considered to be "at the reactor ~~control~~-consoles".

Taking the Reactor Mode Switch to ~~SHUTDOWN~~ **Shutdown** is considered to be a manual scram action.

The plant response to the failure of an automatic or manual reactor scram will vary based upon several factors including the reactor power level prior to the event, availability of the condenser, performance of mitigation equipment and actions, other

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**

concurrent plant conditions, etc. If the failure to shutdown the reactor is prolonged enough to cause a challenge to the RPV water level or RCS heat removal safety functions, the emergency classification level will escalate to a Site Area Emergency via IC **MSS35**. Depending upon plant responses and symptoms, escalation is also possible via IC FS1. Absent the plant conditions needed to meet either IC **MSS35** or FS1, an Alert declaration is appropriate for this event.

It is recognized that plant responses or symptoms may also require an Alert declaration in accordance with the Recognition Category F ICs; however, this IC and EAL are included to ensure a timely emergency declaration.

A reactor shutdown is determined in accordance with applicable Emergency Operating Procedure criteria.

Basis Reference(s):

1. NEI 99-01 Rev 6, SA5
2. EMG-3200.01A, RPV Control – no ATWS
3. EMG-3200.01B, RPV Control – with ATWS
4. 2000-BAS-3200.02, EOP User's Guide
5. 2000-GLN-3200.01, Plant Specific Technical Guideline

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**MSU35****Initiating Condition:**

Automatic or manual scram fails to shutdown the reactor.

Operating Mode Applicability:

1

Emergency Action Level (EAL):

Note: A manual action is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core, and does not include manually driving in control rods or implementation of boron injection strategies.

1.
 - a. ~~An~~a Automatic scram -did not shutdown the reactor as indicated by Reactor Power > 2%.
 - .AND
 - b. ~~A~~s Subsequent manual / ARI action taken at the ~~reactor control consoles~~ Reactor Console is successful in shutting down the reactor.
- OR
2.
 - a. ~~A~~m Manual scram -did not shutdown the reactor as indicated by Reactor Power > 2%.
 - AND
 - b. EITHER of the following:
 1. ~~A~~s Subsequent manual / ARI action taken at the ~~reactor control consoles~~ Reactor Console is successful in shutting down the reactor.
 - OR
 2. ~~A~~s Subsequent automatic scram / ARI is successful in shutting down the reactor.

Basis:

This IC addresses a failure of the RPS to initiate or complete an automatic or manual reactor scram that results in a reactor shutdown, and either a subsequent operator manual action taken at the reactor ~~control~~ consoles or an automatic scram is successful in shutting down the reactor. This event is a precursor to a more significant condition and thus represents a potential degradation of the level of safety of the plant.

EAL #1 Basis

Following the failure on an automatic reactor scram, operators will promptly initiate manual actions at the reactor ~~control~~ consoles to shutdown the reactor (e.g., initiate a manual reactor scram). If these manual actions are successful in shutting down the reactor, core heat generation will quickly fall to a level within the capabilities of the plant's decay heat removal systems.

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS****EAL #2 Basis**

If an initial manual reactor trip is unsuccessful, operators will promptly take manual action at another location(s) on the reactor ~~control~~-consoles to shutdown the reactor (e.g., initiate a manual reactor scram / ARI using a different switch). Depending upon several factors, the initial or subsequent effort to manually scram the reactor, or a concurrent plant condition, may lead to the generation of an automatic reactor scram signal. If a subsequent manual or automatic scram / ARI is successful in shutting down the reactor, core heat generation will quickly fall to a level within the capabilities of the plant's decay heat removal systems.

A manual action at the reactor ~~control~~-consoles is any operator action, or set of actions, which causes the control rods to be rapidly inserted into the core (e.g., initiating a manual reactor scram). This action does not include manually driving in control rods or implementation of boron injection strategies. Actions taken at back-panels or other locations within the Control Room, or any location outside the Control Room, are not considered to be "at the reactor ~~control~~-consoles".

Taking the Reactor Mode Switch to Shutdown is considered to be a manual scram action.

The plant response to the failure of an automatic or manual reactor ~~scram~~ will vary based upon several factors including the reactor power level prior to the event, availability of the condenser, performance of mitigation equipment and actions, other concurrent plant conditions, etc. If subsequent operator manual actions taken at the reactor ~~control~~-consoles are also unsuccessful in shutting down the reactor, then the emergency classification level will escalate to an Alert via IC MSA35. Depending upon the plant response, escalation is also possible via IC FA1. Absent the plant conditions needed to meet either IC MSA35 or FA1, an Unusual Event declaration is appropriate for this event.

A reactor shutdown is determined in accordance with applicable Emergency Operating Procedure criteria.

Should a reactor scram signal be generated as a result of plant work (e.g., RPS setpoint testing), the following classification guidance should be applied.

- If the signal **generated as a result of plant work** causes a plant transient that **creates a real condition that** should have included an automatic reactor scram and the RPS fails to automatically shutdown the reactor, then this IC and the EALs are applicable, and should be evaluated.
- If the signal **generated as a result of plant work** does not cause a plant transient **but should have generated an RPS scram signal** and the scram failure is determined through other means (e.g., assessment of test results), then this IC and the EALs are not applicable and no classification is warranted.

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**

Basis Reference(s):

1. NEI 99-01 Rev 6, SU5
2. EMG-3200.01A, RPV Control – no ATWS
3. EMG-3200.01B, RPV Control – with ATWS
4. 2000-BAS-3200.02, EOP User's Guide
5. 2000-GLN-3200.01, Plant Specific Technical Guideline

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**

MSA42

Initiating Condition:

UNPLANNED loss of Control Room indications for 15 minutes or longer with a significant transient in progress.

Operating Mode Applicability:

1, 2

Emergency Action Level (EAL):

Note: The Emergency Director should declare the **eventAlert** promptly upon determining that **the applicable time 15 minutes** has been exceeded, or will likely be exceeded.

1. a. ~~An~~ UNPLANNED event results in the inability to monitor **ANYone or more Table M1 of the following** parameters from within the Control Room for **≥ 15 minutes or longer**.

[see table below]

{BWR parameter list}
Reactor Power
RPV Water Level
RPV Pressure
Primary Containment Pressure
Suppression Pool Level
Suppression Pool Temperature

Table M1 Control Room Parameters
<ul style="list-style-type: none"> • Reactor Power • RPV Water Level • RPV Pressure • Drywell Pressure • Torus Water Level • Torus Water Temperature

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
SYSTEM MALFUNCTIONS

AND

b. Any ~~Table M2 of the following~~ transient **events** in progress.

- ~~• Automatic or Manual runback greater than 25% thermal reactor power~~
- ~~• Electrical load rejection greater than 25% full electrical load~~
- ~~• Reactor trip~~
- ~~• ECCS (SI) actuation~~

Table M2 Significant Transients

- Turbine Trip
- Reactor Scram
- ECCS Actuation
- Thermal power change >25%
- Thermal Power oscillations > 10%

Basis:

UNPLANNED: A parameter change or an event that is not 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

SAFETY SYSTEM: A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related.

This IC addresses the difficulty associated with monitoring rapidly changing plant conditions during a transient without the ability to obtain SAFETY SYSTEM parameters from within the Control Room. During this condition, the margin to a potential fission product barrier challenge is reduced. It thus represents a potential substantial degradation in the level of safety of the plant.

As used in this EAL, an "inability to monitor" means that values for ~~one or more~~ any of the listed parameters cannot be determined from within the Control Room. This situation would require a loss of all of the Control Room sources for the given parameter(s). For example, the reactor power level cannot be determined from any analog, **computer point**, digital and recorder source within the Control Room.

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**

An event involving a loss of plant indications, annunciators and/or display systems is evaluated in accordance with 10 CFR 50.72 (and associated guidance in NUREG-1022) to determine if an NRC event report is required. The event would be reported if it significantly impaired the capability to perform emergency assessments. In particular, emergency assessments necessary to implement abnormal operating procedures, emergency operating procedures, and emergency plan implementing procedures addressing emergency classification, accident assessment, or protective action decision-making.

This EAL is focused on a selected subset of plant parameters associated with the key safety functions of reactivity control, ~~RPV level~~RPV water level and RCS heat removal. The loss of the ability to determine ~~one or more~~any of these parameters from within the Control Room is considered to be more significant than simply a reportable condition. In addition, if all indication sources for ~~one or more~~any of the listed parameters are lost, then the ability to determine the values of other SAFETY SYSTEM parameters may be impacted as well. For example, if the value for RPV water level cannot be determined from the indications and recorders on a main control board, the SPDS or the plant computer, the availability of other parameter values may be compromised as well.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of indication.

Escalation of the emergency classification level would be via ICs FS1 or IC RAS1.

Basis Reference(s):

1. NEI 99-01 Rev 6, SA2

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**

MSU42

Initiating Condition:

UNPLANNED loss of Control Room indications for 15 minutes or longer.

Operating Mode Applicability:

1, 2

Emergency Action Level (EAL):

Note: The Emergency Director should declare the ~~Unusual Event~~ promptly upon determining that ~~the applicable time 15 minutes~~ has been exceeded, or will likely be exceeded.

~~a. An UNPLANNED event results in the inability to monitor one or more ANY Table M1 parameters from within the Control Room for ≥ 15 minutes.~~

Table M1 Control Room Parameters
<ul style="list-style-type: none"> • Reactor Power • RPV Water Level • RPV Pressure • Drywell Pressure • Torus Water Level • Torus Water Temperature

~~1. of the following parameters from within the Control Room for 15 minutes or longer.~~

2. {BWR parameter list}	3. {PWR parameter list}
4. Reactor Power	6. Reactor Power
5.	7.
8. RPV Water Level	9. RCS Level
10. RPV Pressure	11. RCS Pressure
12. Primary Containment Pressure	13. In-Core/Core Exit Temperature
14. Suppression Pool Level	15. Levels in at least (site-specific number) steam generators
16. Suppression Pool Temperature	17. Steam Generator Auxiliary or Emergency Feed Water Flow

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**

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Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS****Basis:**

UNPLANNED: A parameter change or an event that is not 1) the result of an intended evolution or 2) an expected plant response to a transient. The cause of the parameter change or event may be known or unknown.

SAFETY SYSTEM: A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related.

This IC addresses the difficulty associated with monitoring normal plant conditions without the ability to obtain SAFETY SYSTEM parameters from within the Control Room. This condition is a precursor to a more significant event and represents a potential degradation in the level of safety of the plant.

As used in this EAL, an "inability to monitor" means that values for ~~one or more~~ any of the listed parameters cannot be determined from within the Control Room. This situation would require a loss of all of the Control Room sources for the given parameter(s). For example, the reactor power level cannot be determined from any analog, digital and recorder source within the Control Room.

An event involving a loss of plant indications, annunciators and/or display systems is evaluated in accordance with 10 CFR 50.72 (and associated guidance in NUREG-1022) to determine if an NRC event report is required. The event would be reported if it significantly impaired the capability to perform emergency assessments. In particular, emergency assessments necessary to implement abnormal operating procedures, emergency operating procedures, and emergency plan implementing procedures addressing emergency classification, accident assessment, or protective action decision-making.

This EAL is focused on a selected subset of plant parameters associated with the key safety functions of reactivity control, core cooling and RCS heat removal. The loss of the ability to determine ~~one or more~~ any of these parameters from within the Control Room is considered to be more significant than simply a reportable condition. In addition, if all indication sources for ~~one or more~~ any of the listed parameters are lost, then the ability to determine the values of other SAFETY SYSTEM parameters may be impacted as well. For example, if the value for reactor vessel level cannot be determined from the indications and recorders on a main control board, the SPDS or the plant computer, the availability of other parameter values may be compromised as well.

Fifteen minutes was selected as a threshold to exclude transient or momentary losses of indication.

Escalation of the emergency classification level would be via IC **MSA42**.

Basis Reference(s):

Table OCGS 3-2 OCGS EAL Technical Basis

**RECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**

1. NEI 99-01 Rev 6, SU2

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**MSA59****Initiating Condition:**

Hazardous event affecting a SAFETY SYSTEM ~~needed~~required for the current operating mode.

Operating Mode Applicability:

1, 2

Emergency Action Level (EAL):

(4) 1. ~~a.~~ The occurrence of **ANY** of the following hazardous events:

- Seismic event (earthquake)
- Internal or external flooding event
- High winds or tornado strike
- FIRE
- EXPLOSION
- ~~(site specific hazards)~~
- Other events with similar hazard characteristics as determined by the Shift Manager

AND

2. ~~b.~~ **EITHER** of the following:

a.1. Event damage has caused indications of degraded performance in at least one train of a SAFETY SYSTEM ~~needed~~required by **Technical Specifications** for the current operating mode.

OR

b.2. The event has caused **VISIBLE DAMAGE** to a SAFETY SYSTEM component or structure ~~needed~~required by **Technical Specifications** for the current operating mode.

OR

c. A seismic event required a manual reactor scram per ABN-38, Station Seismic Event.

Basis:

FIRE: Combustion characterized by heat and light. Sources of smoke such as slipping drive belts or overheated electrical equipment do not constitute FIRES. Observation of flame is preferred but is NOT required if large quantities of smoke and heat are observed.

EXPLOSION: A rapid, violent and catastrophic failure of a piece of equipment due to combustion, chemical reaction or overpressurization. A release of steam (from high

Table OCGS 3-2 OCGS EAL Technical Basis**RECOGNITION CATEGORY
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energy lines or components) or an electrical component failure (caused by short circuits, grounding, arcing, etc.) should not automatically be considered an explosion. Such events may require a post-event inspection to determine if the attributes of an explosion are present.

SAFETY SYSTEM: A system required for safe plant operation, cooling down the plant and/or placing it in the cold shutdown condition, including the ECCS. These are typically systems classified as safety-related.

VISIBLE DAMAGE: Damage to a component or structure that is readily observable without measurements, testing, or analysis. The visual impact of the damage is sufficient to cause concern regarding the operability or reliability of the affected component or structure.

This IC addresses a hazardous event that causes damage to a SAFETY SYSTEM, or a structure containing SAFETY SYSTEM components, ~~needed~~ required for the current operating mode, "required", i.e. required to be operable by Technical Specifications for the current operating mode. This condition significantly reduces the margin to a loss or potential loss of a fission product barrier, and therefore represents an actual or potential substantial degradation of the level of safety of the plant. Manual or automatic electrical isolation of safety equipment due to flooding, in and of itself, does not constitute degraded performance and is classified under HU6.

~~EAL 1.b.1~~ EAL #2.a addresses damage to a SAFETY SYSTEM train that is required to be operable by Technical Specifications for the current operating mode, and is in ~~service~~/operation since indications for it will be readily available. The indications of degraded performance should be significant enough to cause concern regarding the operability or reliability of the SAFETY SYSTEM train.

~~EAL 1.b.2~~ EAL #2.b addresses damage to a SAFETY SYSTEM component that is required to be operable by Technical Specifications for the current operating mode, and is not in ~~service~~/operation or readily apparent through indications alone, ~~or as well as~~ damage to a structure containing SAFETY SYSTEM components. Operators will make this determination based on the totality of available event and damage report information. This is intended to be a brief assessment not requiring lengthy analysis or quantification of the damage.

Escalation of the emergency classification level would be via IC FS1 or RAS1.

If the EAL conditions of MA5 are not met then assess the event via HU3, HU4, or HU6.

Basis Reference(s):

1. NEI 99-01, Rev 6 SA9

Table OCGS 3-2 OCGS EAL Technical BasisRECOGNITION CATEGORY
SYSTEM MALFUNCTIONS**MSU64****Initiating Condition:**

RCS leakage for 15 minutes or longer.

Operating Mode Applicability:

1, 2

Emergency Action Level (EAL):

Note: The Emergency Director should declare the ~~Unusual Event~~ promptly upon determining that ~~the applicable time 15 minutes~~ has been exceeded, or will likely be exceeded.

1. RCS unidentified or pressure boundary leakage ~~in the Drywell greater than~~ **> 10 gpm for \geq 15 minutes.** ~~(site-specific value) for 15 minutes or longer.~~

OR

2. RCS identified leakage ~~in the Drywell greater than~~ **>25 gpm for \geq 15 minutes.** ~~(site-specific value) for 15 minutes or longer.~~

OR

3. Leakage from the RCS to a location outside ~~containment the Drywell~~ **>25 gpm for \geq 15 minutes.** ~~greater than 25 gpm for 15 minutes or longer.~~

Basis:

UNISOLABLE: An open or breached system line that cannot be isolated, remotely or locally.

This IC addresses RCS leakage which may be a precursor to a more significant event. In this case, RCS leakage has been detected and operators, following applicable procedures, have been unable to promptly isolate the leak. This condition is considered to be a potential degradation of the level of safety of the plant.

EAL #1 and EAL #2 Basis

These EALs are focused on a loss of mass from the RCS due to "unidentified leakage", "pressure boundary leakage" or "identified leakage" (as these leakage types are defined in the plant Technical Specifications).

EAL #3 Basis

This EAL addresses a RCS mass loss caused by an UNISOLABLE leak through an interfacing system.

These EALs thus apply to leakage into the containment, a secondary-side system ~~(e.g., steam generator tube leakage in a PWR)~~ or a location outside of containment.

The leak rate values for each EAL were selected because they are usually observable with normal Control Room indications. Lesser values typically require time-consuming

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calculations to determine (e.g., a mass balance calculation). EAL #1 uses a lower value that reflects the greater significance of unidentified or pressure boundary leakage.

The release of mass from the RCS due to the as-designed/expected operation of **any** relief valve does not warrant an emergency classification.

~~For BWR's, Aa~~ stuck-open **Safety-Electromatic** Relief Valve (**SRVEMRV**) or **SRV-EMRV** leakage is not considered either identified or unidentified leakage by Technical Specifications and, therefore, is not applicable to this EAL.

The 15-minute threshold duration allows sufficient time for prompt operator actions to isolate the leakage, if possible.

Escalation of the emergency classification level would be via ICs of Recognition Category **RA** or F.

Basis Reference(s):

1. NEI 99-01 Rev 6, SU4
2. Technical Specifications 3.3.D, Reactor Coolant System Leakage