## Advanced Passive 1000 (AP1000) Generic Technical Specification Traveler (GTST)

### Title: Removal of AP1000 GTS Subsection 3.9.5, Containment Penetration

# I. <u>Technical Specifications Task Force (TSTF) Travelers, Approved Since Revision 2 of</u> <u>STS NUREG-1431, and Used to Develop this GTST</u>

#### TSTF Number and Title:

None

#### STS NUREGs Affected:

Not applicable

#### NRC Approval Date:

Not applicable

#### **TSTF Classification:**

Not applicable

#### II. <u>Reference Combined License (RCOL) Standard Departures (Std. Dep.), RCOL COL</u> <u>Items, and RCOL Plant-Specific Technical Specifications (PTS) Changes Used to</u> <u>Develop this GTST</u>

#### RCOL Std. Dep. Number and Title:

None

### **RCOL COL Item Number and Title:**

Not applicable

#### **RCOL PTS Change Number and Title:**

The Vogtle Electric Generating Plant Units 3 and 4 License Amendment Request (VEGP LAR) proposed the following changes to the initial version of the PTS (referred to as the current TS by the VEGP LAR).

These changes include Relocation Changes (R) and are addressed in enumerated discussions of change (DOCs). These changes are discussed in Sections VI and VII of this GTST.

DOC R1: Relocate GTS 3.9.5

## III. <u>Comments on Relations Among TSTFs, RCOL Std. Dep., RCOL COL Items, and</u> <u>RCOL PTS Changes</u>

This section discusses changes: (1) that were applicable to previous designs, but are not to the current design; (2) that are already incorporated in the GTS; and (3) that are superseded by another change.

DOC R1 relocates Subsection 3.9.5 to a document that is required to be controlled in accordance with 10 CFR 52.98. Details of this change are discussed later in Section VI "Traveler Information."

## IV. <u>Additional Changes Proposed as Part of this GTST (modifications proposed by NRC</u> <u>staff and/or clear editorial changes or deviations identified by preparer of GTST)</u>

None

## V. Applicability

## Affected Generic Technical Specifications and Bases:

Section 3.9.5 Containment Penetrations

#### Changes to the Generic Technical Specifications and Bases:

DOC R1 removes Subsection 3.9.5 TS and Bases from the AP1000 GTS Rev. 19 and relocates them to a document that is committed to be controlled in accordance with 10 CFR 52.98.

## VI. <u>Traveler Information</u>

#### **Description of TSTF changes:**

None

#### Rationale for TSTF changes:

Not applicable

#### Description of changes in RCOL Std. Dep., RCOL COL Item(s), and RCOL PTS Changes:

DOC R1 removes Subsection 3.9.5 TS and Bases from the AP1000 GTS Rev. 19 and relocates them to a document that is committed to be controlled in accordance with 10 CFR 52.98.

#### Rationale for changes in RCOL Std. Dep., RCOL COL Item(s), and RCOL PTS Changes:

According to DOC R1 "SNC commits to relocate the Containment Penetrations during movement of irradiated fuel assemblies within containment Specification to a document that is controlled in accordance with 10 CFR 52.98."

The rational given in the DOC states that "current TS 3.9.5 provides requirements for the containment penetrations during movement of irradiated fuel assemblies within containment. The containment serves to contain fission product radioactivity that may be released from the reactor core following an accident, such that offsite radiation exposures are maintained within the requirements. However, there are no safety analyses that assume containment closure during movement of irradiated fuel assemblies within containment, other than those discussed in current TS 3.6.8, "Containment Penetrations," which is applicable in Modes 5 and 6 and which is being maintained in the TS. Fuel handling accidents, analyzed in Final Safety Analysis Report (FSAR) Section 15.7.4, include dropping a single irradiated fuel assembly and handling tool or a heavy object onto other irradiated fuel assemblies. After the activity escapes from the water pool, it is assumed that it is released directly to the environment within a 2-hour period. Closure of the containment purge lines can occur on detection of high radioactivity; however, no credit is taken for this in the analysis. The requirements of current TS 3.9.4, "Refueling Cavity Water Level," and current TS 3.9.7, "Decay Time," ensure that the release of fission product radioactivity, subsequent to a fuel handling accident, results in doses that are well within the dose acceptance limit."

"While the containment design will mitigate the consequences of a fuel handling accident, it would have no effect on core damage frequency, since it does not prevent fuel failure, but only reduces the dose if it occurs. The dose rates associated with the spent fuel handling accidents are not enough to result in large early release, since a delay time to allow fission products to decay is required before fuel can be placed in and therefore moved within the spent fuel pool. As such, while the VFS exhaust subsystem is modeled in the PRA, its importance in limiting the likelihood of a severe accident sequence that has shown to be significant to public health and safety is not significant (i.e., is less than 1.3% of the AP1000 Shutdown Large Release Frequency). Therefore, the requirements of current TS 3.9.5 are relocated to a document that is controlled in accordance with 10 CFR 52.98."

VEGP LAR for TSU DOC concludes that "This change is acceptable because current TS 3.9.5 does not meet the 10 CFR 50.36(c)(2)(ii) criteria for inclusion into the TS, as discussed in the Applicable Regulatory Requirements."

However, as described in Reference (4), NRC staff raised concerns regarding the removal of Subsection 3.9.5 TS and Bases from the AP1000 GTS Rev. 19 and relocating them in another document. These concerns are documented in RAI 16-32 of Reference (4). NRC staff requested Southern Nuclear Operating Company (SNC) either (1) to withdraw the proposed relocation of the existing TS 3.9.5 from the LAR, or (2) to provide additional justification to support the relocation of this subsection.

In response to this RAI 16-32, SNC chose the latter option (2) as documented in Reference (5). SNC provided further justification for the relocation of TS 3.9.5 to a Technical Requirement Manual (TRM) with changes controlled by 10 CFR 52.98. This TRM will be incorporated by reference into FSAR Chapter 16 as a site-specific portion of the FSAR.

The staff's rationale for approving the relocation of Subsection 3.9.5 is provided in Reference (7), VEGP Unit's 3 and 4 Safety Evaluation for Amendment 13.

## Description of additional changes proposed by NRC staff/preparer of GTST:

None

## Rationale for additional changes proposed by NRC staff/preparer of GTST:

Not applicable

## VII. GTST Safety Evaluation

## **Technical Analysis:**

Technical discussion of the proposed removal of Subsection 3.9.5 by DOC R1 is covered in section VI.

# References to Previous NRC Safety Evaluation Reports (SERs):

None

## VIII. <u>Review Information</u>

#### **Evaluator Comments:**

None

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#### **Review Information:**

Availability for public review and comment on Revision 0 of this traveler approved by NRC staff on Friday, May 23, 2014.

## **NRC Final Approval Date:**

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## IX. <u>Evaluator Comments for Consideration in Finalizing Technical Specifications and</u> <u>Bases</u>

None

## X. <u>References Used in GTST</u>

- 1. AP1000 DCD, Revision 19, Section 16, "Technical Specifications," June 2011 (ML11171A500).
- 2. Vogtle Electric Generating Plant (VEGP), Units 3 &4 COL Application, Part 4, Technical Specifications, Revision 3 (ML11180A102, 07/01/2011).
- 3. Vogtle Electric Generating Plant (VEGP) Units 3 and 4 Final Safety Evaluation Report (ML110450302, 08/10/2011)
- Southern Nuclear Operating Company, Vogtle Electric Generating Plant, Unit 3 and 4, Technical Specifications Upgrade License Amendment Request, February 24, 2011 (ML12065A057).
- 5. RAI Letter No. 01 Related to License Amendment Request (LAR) 12-002 for the Vogtle Electric Generating Plant Units 3 and 4 Combined Licenses, September 07, 2012 (ML12251A355).
- Southern Nuclear Operating Company, Vogtle Electric Generating Plant, Units 3 and 4, Response to Request for Additional Information Letter No. 01 Related to License Amendment Request LAR-12-002, ND-12-2015, October 04, 2012 (ML12286A363 and ML12286A360)
- NRC Safety Evaluation (SE) for Amendment No. 13 to Combined License (COL) No. NPF- 91 for Vogtle Electric Generating Plant (VEGP) Unit 3, and Amendment No. 13 to COL No. NPF-92 for VEGP Unit 4, September 9, 2013 (ADAMS Package Accession No. ML13238A337), which contains:

ML13238A355	Cover Letter - Issuance of License Amendment No. 13 for Vogtle Units 3 and 4 (LAR 12-002).
ML13238A359	Enclosure 1 - Amendment No. 13 to COL No. NPF-91
ML13239A256	Enclosure 2 - Amendment No. 13 to COL No. NPF-92
ML13239A284	Enclosure 3 - Revised plant-specific TS pages (Attachment to
	Amendment No. 13)
ML13239A287	Enclosure 4 - Safety Evaluation (SE), and Attachment 1 - Acronyms
ML13239A288	SE Attachment 2 - Table A - Administrative Changes
ML13239A319	SE Attachment 3 - Table M - More Restrictive Changes
ML13239A333	SE Attachment 4 - Table R - Relocated Specifications
ML13239A331	SE Attachment 5 - Table D - Detail Removed Changes
ML13239A316	SE Attachment 6 - Table L - Less Restrictive Changes
The following documents were subsequently issued to correct an administrative error in Enclosure 3:	
ML13277A616	Letter - Correction To The Attachment (Replacement Pages) - Vogtle

 Electric Generating Plant Units 3 and 4- Issuance of Amendment Re: Technical Specifications Upgrade (LAR 12-002) (TAC No. RP9402)
ML13277A637 Enclosure 3 - Revised plant-specific TS pages (Attachment to Amendment No. 13) (corrected)