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From: Whited, Jeffrey
Sent: Thursday, May 22, 2014 9:20 AM
To: talentz@firstenergycorp.com
Cc: phlashley@firstenergycorp.com
Subject: BEAVER VALLEY POWER STATION, UNITS 1 AND 2 – ACCEPTANCE OF REQUESTED LICENSING ACTION RE: REQUEST TO EXTEND CONTAINMENT LEAKAGE TEST FREQUENCY (TAC NOS. MF3985 AND MF3986)

SUBJECT: BEAVER VALLEY POWER STATION, UNITS 1 AND 2 – ACCEPTANCE OF REQUESTED LICENSING ACTION RE: REQUEST TO EXTEND CONTAINMENT LEAKAGE TEST FREQUENCY (TAC NOS. MF3985 AND MF3986)

Dear Mr. Thomas Lentz:

By letter dated April 16, 2014, (Agencywide Document Access and Management System Accession No. ML14111A291) FirstEnergy Nuclear Operating Company submitted a license amendment request for Beaver Valley Power Station Units 1 and 2 (BVPS). The proposed amendment changes BVPS Technical Specification (TS) 5.5.12, "Containment Leakage Rate Testing Program," by deleting reference to the BVPS-1 exemption letter dated December 5, 1984, and replacing the reference to Regulatory Guide (RG) 1.163 with a reference to Nuclear Energy Institute (NEI) topical report NEI 94-01, Revision 3-A, "Industry Guideline for Implementing Performance Based Option of 10 CFR Part 50, Appendix J," in order to develop the BVPS performance-based leakage testing program in accordance with Option B of 10 CFR 50, Appendix J. The purpose is to extend the Type A primary containment Integrated Leak Rate Test (ILRT) intervals to fifteen years and the Type C local leak rate test intervals to 75 months, and incorporate the regulatory positions stated in RG 1.163.

Consistent with Section 50.90 of Title 10 of the *Code of Federal Regulations* (10 CFR), an amendment to the license (including the TS) must fully describe the changes requested, and following as far as applicable, the form prescribed for original applications. Section 50.34 of 10 CFR addresses the content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

The NRC staff has reviewed your application and concluded that it does provide technical information in sufficient detail to enable the NRC staff to complete its detailed technical review and make an independent assessment regarding the acceptability of the proposed license amendment request in terms of regulatory requirements and the protection of public health and safety and the environment. Given the lesser scope and depth of the acceptance review as compared to the detailed technical review, there may be instances in which issues that impact the NRC staff's ability to complete the detailed technical review are identified despite completion of an adequate acceptance review. You will be advised of any further information needed to support the NRC staff's detailed technical review by separate correspondence.

If you have any questions, please contact me at 301-415-4090.

Jeffrey Whited

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