



**Commonwealth Edison**  
Dresden Nuclear Power Station  
RR #1  
Morris, Illinois 60450  
Telephone 815-942-2920

February 22, 1993

CWS PMLTR #: 93-0085

Mr. A. Bert Davis  
Administrator  
U.S. Nuclear Regulatory Commission  
Region III  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

**Subject:** Dresden Nuclear Power Station  
Radioactive Effluent Report  
NRC Dockets 50-10, 50-237, 50-249

Enclosed is the corrected Radioactive Effluent Report for January through June, 1992 for Dresden Nuclear Power Station. Projected data for Sr-89, Sr-90, Fe-55, H-3, and Gross Alpha for April through June were replaced with actual sample results.

A copy of this report will be furnished to the NRC Resident Inspector.

Sincerely Yours,

Charles W. Schroeder  
Station Manager  
Dresden Station

CWS:MG:sib

Enclosure

cc: M. Gagnon  
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DRESDEN NUCLEAR POWER STATION DOCKET NOS. 50-10, 50-237, 50-249

REGULATORY LIMITS

Gaseous Effluents - Dose

This Specification is provided to ensure that the dose at the unrestricted area boundary from gaseous effluents from the units on site will be within the annual dose limits of 10 CFR Part 20 for unrestricted areas. The annual dose limits are the doses associated with the concentrations of 10 CFR Part 20, Appendix B, Table II. These limits provide reasonable assurance that radioactive material discharged in gaseous effluents will not result in the exposure of an individual in an unrestricted area to annual average concentrations exceeding the limits specified in Appendix B, Table II of 10 CFR Part 20 (10 CFR Part 20.106(b)). The specified release rate limits restrict, at all times, the corresponding gamma and beta dose rates above background to an individual at or beyond the unrestricted area boundary to less than or equal to 500 mrem/year to the total body or to less than or equal to 3000 mrem/year to the skin. These release rate limits also restrict, at all times, the corresponding thyroid dose rate above background to a child via the inhalation pathway to less than or equal to 1500 mrem/year. For purposes of calculating doses resulting from airborne releases, the main chimney is considered to be an elevated release point and the reactor building vent stack is considered to be a mixed mode release point.

Dose, Noble Gases

This Specification is provided to implement the requirements of Sections II.B, III.A and IV.A of Appendix I, 10 CFR Part 50. The Limiting Conditions For Operation implement the guides set forth in Section II.3 of Appendix I. The statements provide the required operating flexibility and at the same time implement the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive material in gaseous effluents will be kept "as low as is reasonably achievable." The Surveillance Requirements implement the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I is to be shown by calculational procedures based on models and data such that the actual exposure of an individual through the appropriate pathways is unlikely to be substantially underestimated. The dose calculations established in the ODCM for calculating the doses due to the actual release rates of radioactive noble gases in gaseous effluents will be consistent with the methodology provided in Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR

DRESDEN NUCLEAR POWER STATION DOCKET NOS. 50-10, 50-237, 50-249  
REGULATORY LIMITS

Dose, Noble Gases (continued)

Part 50, Appendix I," Revision 1, October 1977 and Regulatory Guide 1.111, "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water Cooled Reactors," Revision 1, July 1977. The ODCM equations provide for determining the air doses at the unrestricted boundary based upon the historical average atmospheric conditions. NUREG-0133 provides methods for dose calculations consistent with Regulatory Guides 1.109 and 1.111.

DRESDEN NUCLEAR POWER STATION DOCKET NOS. 50-10, 50-237, 50-249

Dose, Radioiodines, Radioactive Material in Particulate Form and Radionuclides Other than Noble Gases

This specification is provided to implement the requirements of Sections II.C, III.A and IV.A of Appendix I, 10 CFR Part 50. The Limiting Conditions for Operation are the guides set forth in Section II.C of Appendix I. The statements provide the required operating flexibility and at the same time implement the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive materials in gaseous effluents will be kept "as low as reasonably achievable." The ODCM calculational methods specified in the surveillance requirements implement the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I be shown by calculational procedures based on models and data such that the actual exposure of an individual through appropriate pathways is unlikely to be substantially underestimated. The ODCM calculational methods approved by NRC for calculating the doses due to the actual release rates of the subject materials are required to be consistent with the methodology provided in Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I", Revision 1, October 1977 and Regulatory Guide 1.111, "Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors," Revision 1, July 1977. These equations also provide for determining the actual doses based upon the historical average atmospheric conditions. The release rate specifications for radioiodines, radioactive material in particulate form and radionuclides other than noble gases are dependent on the existing radionuclide pathways to man, in the unrestricted area. The pathways which were examined in the development of these specifications were: 1) individual inhalation of airborne radionuclides, 2) deposition of radionuclides onto green leafy vegetation with subsequent consumption by man and 3) deposition onto grassy areas where milk animals graze with consumption of the milk by man.

Gaseous Waste Treatment

The OPERABILITY of the gaseous waste treatment which reduces amounts or concentrations of radioactive materials ensures that the system will be available for use whenever gaseous effluents require treatment prior to release to the environment. The requirement that the appropriate portions of this system be operable when specified provides reasonable assurance that the releases of radioactive materials in gaseous effluents will be kept "as low as reasonably achievable". This specification implements the requirements of 10 CFR Part 50.36a, General Design Criterion 60 of Appendix A to 10 CFR Part 50, and design objective Section II.D of Appendix I to 10 CFR Part 50.

DRESDEN NUCLEAR POWER STATION DOCKET NOS. 50-10, 50-237, 50-249

LIQUID EFFLUENTS

**Concentration**

This specification is provided to ensure the concentration of radioactive materials released in liquid waste effluents from the site to unrestricted areas will be less than the concentration levels specified in 10 CFR Part 20, Appendix B, Table II, Column 2. The concentration limit for noble gases, MPC in air (submersion), was converted to an equivalent concentration in water using the International Commission on Radiological Protection (ICRP) Publication 2.

**Dose**

This specification is provided to implement the requirements of Sections II.A, III.A and IV.A of Appendix I, 10 CFR Part 50. The Limiting Condition for Operation implements the guides set forth in Section II.A of Appendix I. The statements provide the required operating flexibility and at the same time implement the guides set forth in Section IV.A of Appendix I to assure that the releases of radioactive material in liquid effluents will be kept "as low as reasonably achievable". The dose calculations in the ODCM implement the requirements in Section III.A of Appendix I that conformance with the guides of Appendix I be shown by calculational procedures based on models and data such that the actual exposure of an individual through appropriate pathways is unlikely to be substantially underestimated. The equations specified in the ODCM for calculating the doses due to the actual release rates of radioactive materials in liquid effluents will be consistent with the methodology provided in Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I", Revision 1, October 1977 and Regulatory Guide 1.113, "Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I", April 1977. NUREG-0113 provides methods for dose calculations with Reg Guide 1.109 and 1.113.

**Liquid Waste Treatment**

The operability of the liquid radwaste treatment system ensures that this system will be available for use whenever liquid effluents require treatment prior to release to the environment. The requirement that the appropriate portions of this system be used when specified provides assurance that the releases of radioactive materials in liquid effluents will be kept "as low as reasonably achievable". This specification implements the requirements of 10 CFR Part 50 and design objective Section II.D of Appendix I to 10 CFR Part 50.

DRESDEN NUCLEAR POWER STATION DOCKET NOS. 50-10, 50-237, 50-249

MAXIMUM PERMISSIBLE CONCENTRATIONS (MPC)

The concentration of radioactive materials released in gaseous and liquid effluents from the site to unrestricted areas will be less than the concentration levels specified in 10 CFR Part 20, Appendix B, Table II, Columns 1 and 2. The concentration limit for noble gases, MPC in air (submersion), was converted to an equivalent concentration in water using the International Commission on Radiological Protection (ICRP) Publication 2.

MAXIMUM PERMISSIBLE CONCENTRATION OF DISSOLVED  
OR ENTRAINED NOBLE GASES RELEASED FROM  
THE SITE TO UNRESTRICTED AREAS  
IN LIQUID WASTE

NUCLIDE	MPC( $\mu\text{Ci}/\text{m}^3$ ) <sup>*</sup>
Kr-85 <sup>m</sup>	2.0E-04
Kr-85	5.0E-04
Kr-87	4.0E-05
Kr-88	9.0E-05
Ar-41	7.0E-05
Xe-131 <sup>m</sup>	7.0E-04
Xe-133 <sup>m</sup>	5.0E-04
Xe-133	6.0E-04
Xe-135 <sup>m</sup>	2.0E-04
Xe-135	2.0E-04

\* Computed from Equation 20 of ICRP Publication 2 (1959), adjusted for infinite cloud submersion in water, and  $R = 0.01 \text{ rem/week}$ , density = 1.0 g/cc and  $P_w/P_t = 1.0$ .

DRESDEN NUCLEAR POWER STATION DOCKET NOS. 50-10,  
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AVERAGE ENERGY

The average energy of fission and activation gases was calculated for the gaseous effluents released from the site. The average energy is based on the percentage of each fission gas nuclide present and its average energy per disintegration ( $E$  in MeV/dis) for gamma and beta emissions separately.

$$E_{\gamma} = 3.75E-01 \text{ MeV/dis}$$

$$E_{\beta} = 2.97E-01 \text{ MeV/dis}$$

EQUIPMENT OUT-OF-SERVICE:

The Units 2 and 3 Main Chimney SPING (System Particulate Iodine Noble Gas Monitor) was out of service from January 29, 1992 to February 29, 1992. This was due to calibration work on the SPING. The 2/3 Chimney G.E. Radiation Monitors and the respective Iodine and particulate samplers were utilized during this period.

DRESDEN NUCLEAR POWER STATION DOCKET NOS. 50-10,  
50-237, 50-249

MEASUREMENTS AND APPROXIMATIONS

- A. Fission and activation gases: The D-1 Chimney, D2/3 Chimney, and D2/3 Reactor Building Vent are sampled weekly via a grab sample. The samples are analyzed for specific isotopes present in the release using a Hyper-Pure Germanium (HP Ge) Spectrometry System. Tritium is sampled monthly via a grab sample on the D-1 Chimney, D2/3 Chimney, and D2/3 Reactor Building Vent and analyzed using a Liquid Scintillation Counter. Krypton-85 is estimated in the D2/3 Chimney using a recoil or non-recoil calculation using the fission per second plot and the sum of Kr-85m, Kr-87, Kr-88, Xe-133, Xe-135, and Xe 138 activities present in Reactor Off-Gas.
- B. Iodine and Particulate: Iodine and particulate samples from the D-1 Chimney, D2/3 Chimney and the D2/3 Reactor Building Vent are collected for a maximum seven day period. These samples are analyzed for specific nuclides present in the release using a HP Ge spectrometry system. When iodine or particulate samples are not used for reporting the release rate due to management decision that the sample may not be representative, an average of the preceding sample and the following sample is used to calculate the release. A monthly composite of the particulate samples is sent to a vendor to be analyzed for Fe-55, Sr-89, Sr-90, and Gross Alpha activity.
- C. Liquid Effluents: Analyzed for specific isotopes present in the release using a HP Ge spectrometry system. A composite of all batches for the month is sent to a vendor to be analyzed for Sr-89, Sr-90, Fe-55, H-3, and Gross Alpha activity. A sample of each Containment Cooling Service Water (CCSW) system is analyzed each month for specific isotopes present in the release using a HP Ge spectrometry system. A sample of each CCSW system is sent each month to a vendor to be analyzed for Sr-89, Sr-90, Fe-55, H-3, and Gross Alpha activity.
- D. Estimation of Overall Errors: The methods used for estimating overall errors associated with radioactivity measurements vary with discharge path and form of isotopes. Factors that contribute to the error include such items as calibration of counting equipment, counting statistics, sampling error, discharge volume, and flow rate monitors.

DRESDEN NUCLEAR POWER STATION  
UNITS 1,2, AND 3  
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
JANUARY Through June 1992

GASEOUS EFFLUENTS Docket Numbers: 50-10  
50-237  
SUMMATION OF ALL RELEASES 50-249

TYPE OF RELEASE	UNITS	EST. TOTAL	
		1st QUARTER	2nd QUARTER
<u>FISSION AND ACTIVATION GASES</u>			
1. Total Release	Ci	7.69E+00	1.09E+00
2. Average Release Rate for Period	uCi/sec	9.78E-01	1.39E-01
3. Percent of Technical Specification Limit	%	*	*
<u>IODINES</u>			
1. Total Iodine-131	Ci	2.18E-07	1.18E-04
2. Average Release Rate of I-131 for Period	uCi/sec	2.77E-08	1.50E-05
3. Percent of Technical Specification Limit	%	*	*
4. Total Iodine-131, Iodine-133, and Iodine-135	Ci	2.87E-04	1.39E-03
<u>PARTICULATES</u>			
1. Particulates with half-lives > 8 days	Ci	6.40E-03	9.56E-03
2. Average Release Rate for Period	uCi/sec	8.14E-04	1.22E-03
3. Percent of Technical Specification Limit	%	*	*
4. Gross Alpha Radioactivity	Ci	3.95E-06	LLD
<u>TRITIUM</u>			
1. Total Release	Ci	6.17E-01	1.51E+00
2. Average Release Rate for Period	uCi/sec	7.63E-02	1.92E-01
3. Percent of Technical Specification Limit	%	*	*

- \* The information is contained in the Radiological Impact on Man section of the report. Total airborne release data is provided which includes fission and activation gases, iodines, particulates, and tritium.

DRESDEN NUCLEAR POWER STATION  
UNITS 1, 2, AND 3  
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
January Through June 1992

GASEOUS EFFLUENTS      Docket Numbers: 50-10  
SUMMATION OF ALL RELEASES      50-237  
LLD (uCi/cc)      50-249

**1. FISSION GASES**

Xe-138	1.60E-07
Xe-135m	9.90E-08
Kr-87	4.40E-08
Kr-88	5.40E-08
Kr-85m	1.80E-08
Kr-85	5.30E-08
Xe-135	1.60E-08
Xe-133	4.40E-08
Ar-41	2.50E-08
Xe-133m	1.50E-07

**2. IODINES**

I-131	7.90E-11
I-133	3.40E-12
I-135	3.70E-10

**3. PARTICULATES**

Sr-89	1.40E-14
Sr-90	3.00E-15
Cs-51	5.40E-12
Mn-54	6.80E-11
Co-58	8.00E-11
Fe-55	3.00E-14
Fe-59	1.60E-12
Co-60	1.40E-12
Zr-95	1.50E-12
Nb-95	7.10E-13
Mo-99	7.40E-11
Ru-101	7.20E-11
Ag-110m	6.30E-11
Sb-124	5.20E-11
I-131	7.70E-11
Cn-135	6.80E-11
Cn-136	7.30E-11
Cn-137	7.60E-11
Ba-140	3.30E-12
La-140	1.10E-12
Cn-141	9.99E-11
Sc-144	3.80E-12
Zn-65	1.70E-13
Ba-133	7.20E-11
Sb-125	1.90E-12
Others:	
Gross Alpha	1.2E-14

DRESDEN NUCLEAR POWER STATION  
UNIT 1  
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
JANUARY Through June 1992

D1 Chimney GASEOUS EFFLUENTS

GROUND LEVEL RELEASES

Docket Number: 50-10

SEMI-ELEVATED RELEASES

XX ELEVATED RELEASES

CONTINUOUS MODE

BATCH MODE

NUCLIDES RELEASED	UNIT	1st QUARTER	2nd QUARTER	1st QUARTER	2nd QUARTER
<b>FISSION GASES</b>					
Xe-136	C1	•	•		
Xe-135m	C1	•	•		
Kr-87	C1	•	•		
Kr-88	C1	•	•		
Kr-85m	C1	•	•		
Kr-85	C1	•	•		
Xe-135	C1	•	•		
Xe-133	C1	•	•		
<b>TOTAL</b>	<b>C1</b>			<b>NONE</b>	<b>NONE</b>
<b>IODINES</b>					
I-131	C1	•	•		
I-132	C1	•	•		
I-133	C1	•	•		
<b>TOTAL</b>	<b>C1</b>			<b>NONE</b>	<b>NONE</b>
<b>PARTICULATES</b>					
Sr-89	C1	3.88E-07	1.75E-07		
Sr-90	C1	8.19E-04	•		
Cr-51	C1	•	•		
Mn-54	C1	•	•		
Co-58	C1	•	•		
Fe-59	C1	•	•		
Co-60	C1	2.79E-06	2.25E-06		
Zr-95	C1	•	•		
Nb-95	C1	•	•		
Mo-99	C1	•	•		
Pu-103	C1	•	•		
Ag-110m	C1	•	•		
Sb-124	C1	•	•		
I-131	C1	•	•		
Ca-134	C1	•	•		
Ca-136	C1	•	•		
Ca-137	C1	6.79E-06	3.81E-06		
Ba-140	C1	•	•		
La-140	C1	•	•		
Co-141	C1	•	•		
Co-144	C1	•	•		
Zn-65	C1	•	•		
Ba-133	C1	•	•		
Sb-125	C1	•	•		
Fe-55	C1	•	3.09E-06		
<b>TOTAL</b>	<b>C1</b>	<b>1.00E-05</b>	<b>9.13E-06</b>	<b>NONE</b>	<b>NONE</b>

\* The activity of this nuclide is less than the LLD listed on the appropriate table.

DRESDEN NUCLEAR POWER STATION  
UNIT 1 EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
JANUARY Through March 1992

PJ Cherry GASEOUS EFFLUENTS  
GROUNd LEVEL RELEASES  
SEMI-ELEVATED RELEASES

Docket Number: 10-10

XX ELEVATED RELEASES

NUCLIDES RELEASED	UNIT	CONTINUOUS MODE			1/4 QUARTER TOTAL
		JANUARY	FEBRUARY	MARCH	
<b>FISSION GASES</b>					
Xe-138	C1	.	.	.	.
Xe-135m	C1	.	.	.	.
Kr-87	C1	.	.	.	.
Kr-88	C1	.	.	.	.
Kr-85m	C1	.	.	.	.
Kr-85	C1	.	.	.	.
Xe-135	C1	.	.	.	.
Xe-133	C1	.	.	.	.
TOTAL	C1				
<b>IODINES</b>					
I-131	C1	.	.	.	.
I-133	C1	.	.	.	.
I-129	C1	.	.	.	.
TOTAL	C1				
<b>PARTICULATES</b>					
Sr-89	C1	6.29E-08	3.90E-08	2.86E-07	3.88E-07
Sr-90	C1	4.21E-09	4.40E-09	3.30E-08	8.19E-08
Cr-51	C1	.	.	.	.
Mn-54	C1	.	.	.	.
Co-58	C1	.	.	.	.
Fe-59	C1	.	.	.	.
Co-60	C1		1.53E-06	1.72E-06	2.75E-06
Zr-95	C1	.	.	.	.
Nh-95	C1	.	.	.	.
Mo-99	C1	.	.	.	.
Ru-103	C1	.	.	.	.
Ag-110m	C1	.	.	.	.
Sb-124	C1	.	.	.	.
I-131	C1	.	.	.	.
Ca-114	C1	.	.	.	.
Cn-116	C1	.	.	.	.
Ca-137	C1	7.17E-06	4.62E-06	.	6.79E-06
Ra-140	C1	.	.	.	.
La-140	C1	.	.	.	.
Ca-141	C1	.	.	.	.
Co-144	C1	.	.	.	.
Zn-65	C1	.	.	.	.
Ba-133	C1	.	.	.	.
Sb-125	C1	.	.	.	.
Fe-55	C1	.	.	.	.
TOTAL	C1	7.24E-06	6.23E-06	1.58E-06	1.00E-05

\* The activity of this nuclide is less than the LID listed on the appropriate table.

DRESDEN NUCLEAR POWER STATION  
UNIT 1  
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
April Through June 1992

Bl. Chimney GASEOUS EFFLUENTS

GROUND LEVEL RELEASES

Docket Number: 50-10

SEMI-ELEVATED RELEASES

XX ELEVATED RELEASES

CONTINUOUS MODE

NUCLIDES RELEASED	UNIT	APRIL	MAY	JUNE	2nd QUARTER TOTAL
FISSION GASES					
Xe-138	C1	•	•	•	•
Xe-135m	C1	•	•	•	•
Kr-87	C1	•	•	•	•
Kr-88	C1	•	•	•	•
Kr-85m	C1	•	•	•	•
Kr-85	C1	•	•	•	•
Xn-135	C1	•	•	•	•
Xe-133	C1	•	•	•	•
TOTAL	C1				
IODINES					
I-131	C1	•	•	•	•
I-133	C1	•	•	•	•
I-135	C1	•	•	•	•
TOTAL	C1				
PARTICULATES					
Sr-89	C1	1.46E-07	2.21E-06	•	1.75E-07
Sr-90	C1	•	•	•	•
Cr-51	C1	•	•	•	•
Mn-54	C1	•	•	•	•
Co-58	C1	•	•	•	•
Fe-59	C1	•	•	•	•
Co-60	C1	2.25E-06	•	•	2.25E-06
Zr-95	C1	•	•	•	•
Nb-95	C1	•	•	•	•
Pu-103	C1	•	•	•	•
Ag-110m	C1	•	•	•	•
Sb-124	C1	•	•	•	•
I-131	C1	•	•	•	•
Ca-134	C1	•	•	•	•
Ca-136	C1	•	•	•	•
Ca-137	C1	2.38E-06	•	1.43E-06	3.81E-06
Ra-140	C1	•	•	•	•
La-140	C1	•	•	•	•
Co-141	C1	•	•	•	•
Co-144	C1	•	•	•	•
Zn-69	C1	•	•	•	•
Ba-133	C1	•	•	•	•
Sb-125	C1	•	•	•	•
Fo-55	C1	9.13E-07	1.96E-06	2.16E-07	1.09E-06
TOTAL	C1	5.69E-06	1.99E-06	1.65E-06	9.33E-06

\* The activity of this nuclide is less than the LLD listed on the appropriate table.

DRESDEN NUCLEAR POWER STATION  
 UNITS 2 AND 3 EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
January Through June 1992

D2/3 CHIMNEY GASEOUS EFFLUENTS

GROUND LEVEL RELEASES

Docket Numbers: 50-237

SEMI-ELEVATED RELEASES

50-249

XX ELEVATED RELEASES

NUCLIDES RELEASED	UNIT	CONTINUOUS MODE		BATCH MODE	
		1st QUARTER	2nd QUARTER	1st QUARTER	2nd QUARTER
<b>FISSION GASES</b>					
Xe-138	C1	•	•	—	—
Xe-135m	C1	3.75E+00	•	—	—
Kr-87	C1	7.22E-01	•	—	—
I-135	C1	6.92E-02	•	—	—
Kr-85m	C1	1.31E-01	•	—	—
Kr-85	C1	1.15E-04	4.69E-04	—	—
Xe-133	C1	2.64E+00	1.09E+00	—	—
Xe-131	C1	3.81E-01	•	—	—
<b>TOTAL</b>	C1	<b>7.69E+00</b>	<b>1.09E+00</b>	None	None
<b>IODINES</b>					
I-131	C1	•	9.95E-05	—	—
I-133	C1	1.95E-04	1.05E-03	—	—
I-135	C1	•	•	—	—
<b>TOTAL</b>	C1	<b>1.95E-04</b>	<b>1.15E-03</b>	None	None
<b>PARTICULATES</b>					
Sr-89	C1	1.16E-05	3.42E-05	—	—
Sr-90	C1	5.94E-07	7.66E-07	—	—
Cr-51	C1	•	•	—	—
Mn-54	C1	1.28E-04	1.23E-04	—	—
Co-58	C1	•	•	—	—
Fe-59	C1	•	•	—	—
Co-60	C1	2.67E-04	2.79E-04	—	—
Zr-95	C1	•	•	—	—
Nb-95	C1	•	•	—	—
Ru-103	C1	•	•	—	—
Ag-110m	C1	•	•	—	—
Sb-124	C1	•	•	—	—
I-131	C1	•	•	—	—
Ca-134	C1	•	•	—	—
Cr-136	C1	•	•	—	—
Co-137	C1	2.05E-05	2.04E-05	—	—
Ba-140	C1	•	•	—	—
Li-140	C1	•	•	—	—
Ce-141	C1	•	•	—	—
Co-134	C1	•	•	—	—
An-65	C1	•	•	—	—
Ba-133	C1	•	•	—	—
Sr-125	C1	•	•	—	—
Pt-95	C1	9.56E-04	3.47E-03	—	—
<b>TOTAL</b>	C1	<b>1.38E-03</b>	<b>1.93E-03</b>	None	None

\* The activity of this nuclide is less than the LLD listed on the appropriate table.

DRESDEN NUCLEAR POWER STATION  
UNITS 2 AND 3  
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
JANUARY Through March 1992

07/3 Chimney Gaseous Effluents

GROUND LEVEL RELEASES

Docket Numbers: SD-237

SEMI-ELEVATED RELEASES

50-249

XX ELEVATED RELEASES

CONTINUOUS MODE

NUCLIDES RELEASED	UNIT	JANUARY	FEBRUARY	MARCH	1ST QUARTER TOTAL
<b>FISSION GASES</b>					
Xe-138	C1	•	•	•	•
Xe-135m	C1	•	•	3.75E+00	3.75E+00
Kr-87	C1	•	7.22E-01	•	7.22E-01
Kr-88	C1	•	6.93E-02	•	6.92E-02
Kr-85m	C1	•	1.31E-01	•	1.31E-01
Kr-85	C1	•	1.86E-05	7.61E-05	1.15E-04
Xe-135	C1	•	1.87E+00	7.71E-01	2.64E+00
Xe-133	C1	•	3.46E-02	3.46E-01	3.81E-01
TOTAL	C1		2.43E+00	4.87E+00	7.69E+00
<b>IODINES</b>					
I-131	C1	•	•	•	•
I-133	C1	•	7.75E-05	1.17E-04	1.55E-04
I-135	C1	•	•	•	•
TOTAL	C1		7.75E-05	1.17E-04	1.95E-04
<b>PARTICULATES</b>					
Sr-89	C1	6.22E-07	6.10E-06	4.77E-06	1.10E-05
Sr-90	C1	1.20E-07	2.08E-07	1.37E-07	5.95E-07
Cr-51	C1	•	•	•	•
Mn-54	C1	1.54E-05	6.15E-05	5.10E-05	1.28E-04
Co-58	C1	•	•	•	•
Fe-59	C1	•	•	•	•
Co-60	C1	3.27E-05	8.77E-05	1.47E-04	2.67E-04
Zr-95	C1	•	•	•	•
Nd-95	C1	•	•	•	•
Pu-103	C1	•	•	•	•
Aq-110m	C1	•	•	•	•
Sb-124	C1	•	•	•	•
I-131	C1	•	•	•	•
Ga-134	C1	•	•	•	•
Cm-136	C1	•	•	•	•
Ga-137	C1	1.11E-05	7.44E-06	•	2.05E-05
Ba-140	C1	•	•	•	•
La-140	C1	•	•	•	•
Ce-141	C1	•	•	•	•
Co-144	C1	•	•	•	•
Zn-65	C1	•	•	•	•
Ba-133	C1	•	•	•	•
Sb-125	C1	•	•	•	•
Fr-90	C1	1.38E-04	3.92E-04	4.10E-04	9.40E-04
TOTAL	C1	1.95E-04	5.55E-04	6.33E-04	1.38E-03

\* The activity of this nuclide is less than the LLD listed on the appropriate table.

DRESDEN NUCLEAR POWER STATION  
 UNITS 2 and 3 EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
April Through June 1992

P2A3 Chimney

GASEOUS EFFLUENTS

GROUND LEVEL RELEASES

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SEMI-ELEVATED RELEASES

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XX

ELEVATED RELEASES

CONTINUOUS MODE

NUCLIDES RELEASED	UNIT	APRIL	MAY	JUNE	2nd QUARTER TL
FISSION GASES					
Xe-138	C1	•	•	•	•
Xe-135m	C1	•	•	•	•
Kr-87	C1	•	•	•	•
Kr-88	C1	•	•	•	•
Kr-85m	C1	•	•	•	•
Kr-95	C1	1.12E-04	1.84E-04	1.73E-04	4.69E-04
Xe-135	C1	1.09E+00	•	•	1.09E+00
Xe-123	C1	•	•	•	
TOTAL	C1	1.09E+00	1.84E-04	1.73E-04	1.09E+00
IODINES					
I-131	C1	1.46E-05	4.73E-05	4.25E-05	9.95E-05
I-133	C1	1.00E-04	2.99E-04	4.45E-04	1.05E-03
I-135	C1	•	•	•	•
TOTAL	C1	3.15E-04	3.41E-04	3.89E-04	1.15E-03
PARTICULATES					
Sr-89	C1	1.59E-05	3.60E-06	3.47E-05	3.42E-05
Sr-90	C1	2.87E-07	4.79E-07	•	7.66E-07
Cr-51	C1	•	•	•	•
Mn-54	C1	7.79E-05	4.53E-05	•	1.23E-04
Co-58	C1	•	•	•	•
Fe-59	C1	•	•	•	•
Co-60	C1	1.19E-04	1.08E-04	5.23E-05	2.79E-04
Zr-95	C1	•	•	•	•
Nb-95	C1	•	•	•	•
Ru-103	C1	•	•	•	•
An-110m	C1	•	•	•	•
Sh-124	C1	•	•	•	•
T-131	C1	•	•	•	•
Ca-134	C1	•	•	•	•
Ca-136	C1	•	•	•	•
Ca-137	C1	1.74E-05	8.00E-06	•	2.04E-05
Ba-140	C1	•	•	•	•
La-140	C1	•	•	•	•
Co-141	C1	•	•	•	•
Ce-144	C1	•	•	•	•
Zn-65	C1	•	•	•	•
Ba-133	C1	•	•	•	•
Sh-125	C1	•	•	•	•
Fe-55	C1	8.51E-04	5.37E-04	5.37E-05	1.47E-03
TOTAL	C1	1.09E-03	7.02E-04	1.52E-04	1.93E-03

\* The activity of this nuclide is less than the LLD listed on the appropriate table.

DRESDEN NUCLEAR POWER STATION  
UNITS 2 AND 3 EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
JANUARY Through June 1992

02/1 PA Building Vent GASEOUS EFFLUENTS

GROUND LEVEL RELEASES

Docket Number: 50-237

XX SEMI-ELEVATED RELEASES 50-249

ELEVATED RELEASES

CONTINUOUS MODE

BATCH MODE

NUCLIDES RELEASED	UNIT	1st QUARTER	2nd QUARTER	1st QUARTER	2nd QUARTER
FISSION GASES					
Xe-138	C1	•	•		
Xe-135m	C1	•	•		
Kr-87	C1	•	•		
Kr-88	C1	•	•		
Fr-85m	C1	•	•		
Kr-85	C1	•	•		
Xe-135	C1	•	•		
Xe-133	C1	•	•		
TOTAL	C1			NONE	NONE
IODINES					
I-131	C1	2.14E-07	1.84E-05		
I-133	C1	9.19E-05	2.20E-04		
I-135	C1	•	•		
TOTAL	C1	9.23E-05	2.14E-04	NONE	NONE
PARTICULATES					
Sr-89	C1	6.54E-06	3.41E-06		
Sr-90	C1	5.24E-08	1.83E-07		
Cr-53	C1	1.66E-04	2.11E-04		
Mn-59	C1	4.12E-04	5.82E-03		
Co-58	C1	1.18E-04	3.42E-04		
Fr-59	C1	4.57E-05	1.24E-04		
Co-60	C1	1.51E-03	2.07E-03		
Zr-95	C1	•	•		
Nh-95	C1	•	•		
Ru-103	C1	•	•		
Ag-110m	C1	•	•		
Sb-124	C1	•	•		
I-131	C1	•	•		
Ca-134	C1	•	•		
Ca-136	C1	•	•		
Co-137	C1	8.49E-06	•		
Rn-140	C1	•	•		
La-140	C1	•	•		
Ce-141	C1	•	•		
Co-144	C1	•	•		
Zn-65	C1	3.18E-05	•		
Ba-133	C1	•	•		
Mo-99	C1	8.51E-05	•		
Sb-125	C1	•	•		
Po-231	C1	2.93E-03	4.34E-03		
TOTAL	C1	5.01E-03	7.62E-03	NONE	NONE

\* The activity of this nuclide is less than the LLD listed on the appropriate table.

DRESDEN NUCLEAR POWER STATION  
 UNITS 2 and 3 EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
January Through March 1992

D2/J Rx Building Vent      GASEOUS EFFLUENTS

GROUND LEVEL RELEASES

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XX      SEMI-ELEVATED RELEASES

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ELEVATED RELEASES

CONTINUOUS MODE

BATCH MODE

NUCLIDES RELEASED	UNIT	JANUARY	FEBRUARY	MARCH	1st QUARTER
<u>FISSION GASES</u>					
Xe-138	C1	*	*	*	*
Xe-135m	C1	*	*	*	*
Kr-87	C1	*	*	*	*
Kr-88	C1	*	*	*	*
Kr-85m	C1	*	*	*	*
Kr-85	C1	*	*	*	*
Xe-135	C1	*	*	*	*
Xe-133	C1	*	*	*	*
TOTAL	C1	*	*	*	*
TOPINES					
I-131	C1	*	-	2.18E-07	2.18E-07
I-132	C1	*	7.03E-05	7.15E-05	9.19E-05
I-135	C1	*	*	*	*
TOTAL	C1	*	7.03E-05	7.17E-05	9.21E-05
<u>PARTICULATES</u>					
Sr-89	C1	1.85E-07	7.51E-07	5.46E-06	6.56E-06
Sr-90	C1	3.92E-06	1.96E-06	*	5.93E-06
Cr-51	C1	*	3.10E-05	1.35E-04	1.66E-04
Mn-59	C1	1.81E-05	1.97E-04	1.97E-04	3.12E-04
Co-58	C1	*	4.71E-05	7.05E-05	1.18E-04
Fe-59	C1	*	1.45E-05	2.73E-05	4.57E-05
Co-60	C1	1.29E-04	6.85E-04	6.92E-04	1.51E-03
Zr-95	C1	*	*	*	*
Nb-95	C1	*	*	*	*
Ru-103	C1	*	*	*	*
Ag-110m	C1	*	*	*	*
Sb-124	C1	*	*	*	*
I-131	C1	*	*	*	*
Cn-134	C1	*	*	*	*
Cn-136	C1	*	*	*	*
Cn-137	C1	7.23E-06	*	1.48E-06	8.89E-06
Ba-140	C1	*	*	*	*
La-140	C1	*	*	*	*
Cr-141	C1	*	*	*	*
Cr-144	C1	*	*	*	*
Sn-113	C1	*	3.97E-05	1.33E-05	1.16E-05
Ba-133	C1	*	*	*	*
Kr-99	C1	*	*	8.51E-05	8.53E-05
Sb-125	C1	*	*	*	*
Fr-55	C1	3.37E-04	1.04E-03	1.26E-03	2.63E-03
TOTAL	C1	4.80E-04	2.04E-03	2.49E-03	5.31E-03

\* The activity of this nuclide is less than the LLD listed on the appropriate table.

DRESDEN NUCLEAR POWER STATION  
 UNITS 2 and 3 EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
April Through June 1992

D-2/2 Rx Building Vent      GASEOUS EFFLUENTS

GROUND LEVEL RELEASES

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XX      SEMI-ELEVATED RELEASES

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ELEVATED RELEASES

CONTINUOUS MODE

NUCLIDES RELEASED	UNIT	APRIL	MAY	JUNE	2nd QUARTER TOTAL
<b>FISSION GASES</b>					
Xe-138	CL	•	•	•	•
Xe-135m	CL	•	•	•	•
Kr-87	CL	•	•	•	•
Kr-88	CL	•	•	•	•
Kr-85m	CL	•	•	•	•
Kr-85	CL	•	•	•	•
Xe-125	CL	•	•	•	•
Xe-133	CL	•	•	•	•
TOTAL	CL				
<b>ISOTOPES</b>					
I-121	CL	6.21E-06	2.67E-06	9.50E-06	1.84E-05
I-131	CL	9.07E-05	1.91E-05	1.10E-04	2.20E-04
J-135	CL	•	•	•	•
TOTAL	CL	9.69E-05	2.18E-05	1.20E-04	2.38E-04
<b>PARTICULATES</b>					
Sr-89	CL	1.25E-06	1.21E-06	1.35E-06	3.81E-06
Sr-90	CL	•	•	1.81E-07	1.81E-07
Cr-51	CL	1.19E-04	•	9.42E-05	2.13E-04
Mn-54	CL	2.59E-04	7.34E-05	2.56E-04	5.89E-04
Co-58	CL	6.70E-05	5.49E-05	1.20E-04	2.43E-04
Fe-59	CL	6.19E-05	•	6.70E-05	1.24E-04
Co-60	CL	6.96E-05	1.39E-04	1.02E-03	2.07E-03
Zr-95	CL	•	•	•	•
Nb-95	CL	•	•	•	•
Ru-103	CL	•	•	•	•
Ag-110m	CL	•	•	•	•
Sb-124	CL	•	•	•	•
I-131	CL	•	•	•	•
Co-113	CL	•	•	•	•
Cn-136	CL	•	•	•	•
Cs-137	CL	•	•	•	•
Ba-140	CL	•	•	•	•
La-140	CL	•	•	•	•
Ce-141	CL	•	•	•	•
Cu-64	CL	•	•	•	•
Zn-65	CL	•	•	•	•
Ba-133	CL	•	•	•	•
Mo-99	CL	•	•	•	•
Sb-125	CL	•	•	•	•
Fe-55	CL	1.33E-03	6.97E-04	2.35E-03	4.28E-03
TOTAL	CL	2.53E-03	1.18E-03	3.40E-03	7.62E-03

\* The activity of this nuclide is less than the LLD listed on the appropriate table.

DRESDEN NUCLEAR POWER STATION  
 UNITS 1, 2, and 3  
 EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
JANUARY Through June 1992  
 LIQUID EFFLUENTS

SUMMATION OF ALL RELEASES Docket Numbers: 50-10  
 50-237  
 50-249

EST. TOTAL  
 UNITS 1st QUARTER 2nd QUARTER ERRCR, %

A. FISSION AND ACTIVATION PRODUCTS

1. Total Release (not incl. tritium, gases, alpha)	cI	1.47E-02	6.31E-03	5.58
2. Average Diluted Conc. During Period	uCi/mL	6.52E-09	1.98E-09	
3. Percent of Applicable Limit	*	*	*	

B. TRITIUM

1. Total Release	cI	1.24E-00	1.54E+00	7.75
2. Average Diluted Conc. During Period	uCi/mL	5.50E-07	4.83E-07	
3. Percent of Applicable Limit	*	*	*	

C. DISSOLVED AND ENTRAINED GASES

1. Total Release	cI	1.64E-05	< LLD	5.58
2. Average Diluted Conc. During Period	uCi/mL	7.29E-12	< LLD	
3. Percent of Applicable Limit	*	*	*	

D. GROSS ALPHA RADIOACTIVITY

1. Total Release	cI	< LLD	< LLD	15.1
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E. VOLUME OF WASTE RELEASED (prior to dilution)	Liters	4.36E+06	3.09E+06	5.00
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F. VOLUME OF DILUTION WATER USED DURING PERIOD	Liters	2.25E+09	3.19E+09	5.00
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\* The information is contained in the Radiological Impact on Man section of this report. Total liquid release data is provided which includes fission and activation products, tritium, and dissolved and entrained gases.

DRESDEN NUCLEAR ENERGY STATION  
 UNITS 1, 2, & 3  
 EFFLUENT AND WASTE DISCHARGE SEMIANNUAL REPORT  
JANUARY Through JULY, 1992

TABLE OF LOWER LIMITS OF DETECTABILITY  
 FOR LIQUID EFFLUENTS

Docket Number: 50-10  
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NUCLIDE	LID (uCi/mL)
Sr-89	7.00E-06
Sr-90	5.02E-05
Mn-54	3.14E-05
Co-58	4.39E-05
Fe-59	9.39E-05
Co-60	1.23E-07
Tl-205	1.20E-07
Sb-124	4.50E-08
I-131	4.50E-08
Cs-134	4.60E-08
Cs-137	7.22E-08
Ra-140	1.00E-07
La-140	4.60E-08
Cr-141	4.90E-08
Xe-133	1.20E-07
Xe-135	1.14E-08
Cr-51	1.70E-07
Fe-55	1.50E-07
Ca-40	1.10E-07
H-3	3.00E-07
Gross Alpha	8.00E-08
Zr-95	9.50E-08
Kr-87	1.50E-07
Kr-88	1.70E-07
I-135	1.10E-07
I-132	6.70E-08
Aq-110m	5.90E-08
Pa-133	6.00E-08
Ce-144	3.10E-07
Cs-136	5.50E-08
I-133	6.30E-08
I-134	1.50E-07
Kr-85	1.10E-05
Mn-99	3.90E-08
Nb-95	5.00E-08
Fr-239	1.50E-07
Pu-103	5.00E-08
St-125	1.50E-07
Xe-131m	1.60E-06
Xe-133m	1.60E-07
Te-138	4.10E-06

\*This limit was reported as an MDL by the offsite vendor for CCSW samples for discharges from 06-13-92 to 06-16-92. All other discharge samples had MDAs reported at less than or equal to 1.00E-08 uCi/mL.

DRESDEN NUCLEAR POWER STATION  
UNITS 1, 2, AND 3  
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
JANUARY Through JUNE 1992

Radiation LIQUID EFFLUENTS Docket Number: 50-10

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1. Number of Batch Releases: 71
2. Total Time Period for Batch Releases:  $1.83E+04$  min
3. Maximum Time Period for a Batch Release:  $3.74E+02$  min
4. Average Time Period for Batch Releases:  $2.57E+02$  min
5. Minimum Time Period for a Batch Release:  $1.52E+02$  min
6. Average Stream Flow During Periods of Release of Effluent into a Flowing Stream:  $3.00E+05$  L/min

CONTINUOUS MODE      BATCH MODE

NUCLIDES RELEASED	UNIT	1st QUARTER	2nd QUARTER	1st QUARTER	2nd QUARTER
Sc-89	C1			•	$7.81E-06$
Sr-90	C1			•	$7.39E-06$
Mn-54	C1			$3.60E-03$	$9.36E-04$
Co-58	C1			$3.94E-06$	•
Fe-59	C1			$8.15E-05$	•
Co-60	C1			$7.69E-03$	$2.52E-03$
Zn-65	C1			•	•
Ru-103	C1			•	•
Ag-119m	C1			•	•
Sb-124	C1			•	•
I-131	C1			•	•
Ca-43	C1			•	•
Ca-45	C1			$2.16E-03$	$2.08E-03$
Ba-130	C1			•	•
La-140	C1			•	•
Co-147	C1			•	•
Cr-138	C1			•	•
Fr-59	C1			$1.01E-03$	$7.44E-04$
Zr-95	C1			•	•
I-132	C1			•	•
I-134	C1			•	•
Bl-214	C1			$3.60E-05$	•
(above)					
TOTAL FOR Period	C1	NONE	NONE	$1.47E-02$	$6.30E-03$
Xe-133	C1			•	•
Xe-135	C1			$1.64E-05$	•
Kr-88	C1			•	•

\* The activity of this nuclide is less than the LLD listed on the appropriate table.

**DRESDEN NUCLEAR POWER STATION**  
**UNITS 1, 2, AND 3**  
**EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT**  
JANUARY Through MARCH 1982

Radiation LIQUID EFFLUENTS Docket Numbers: SO-10  
 SO-237  
 SO-249

NUCLIDES RELEASED	UNIT	BATCH MODE			1st QUARTER TOTAL
		JANUARY	FEBRUARY	MARCH	
Sc-99	C1	•	•	•	•
St-90	C1	•	•	•	•
Mn-54	C1	5.79E-04	1.43E-03	1.79E-03	3.80E-03
Co-58	C1	•	•	3.94E-06	3.94E-06
Fe-59	C1	•	•	8.15E-05	8.15E-05
Co-60	C1	2.20E-03	2.60E-03	2.89E-03	7.69E-03
Zn-65	C1	•	•	•	•
Ru-193	C1	•	•	•	•
Az-110m	C1	•	•	•	•
Sb-124	C1	•	•	•	•
I-131	C1	•	•	•	•
Cr-134	C1	•	•	•	•
Co-137	C1	1.24E-03	4.64E-04	4.32E-04	2.14E-03
Ra-140	C1	•	•	•	•
La-140	C1	•	•	•	•
Ca-138	C1	•	•	•	•
Fe-55	C1	6.10E-04	2.74E-04	3.23E-04	1.01E-03
I-132	C1	•	•	•	•
I-133	C1	•	•	•	•
Ba-714	C1	•	1.60E-05	•	1.60E-05
(above)					
Total For Period	C1	4.63E-03	4.74E-03	5.32E-03	1.47E-02
Xe-133	C1	•	•	•	•
Xe-135	C1	•	6.02E-06	1.04E-05	1.64E-05

\* The activity of this nuclide is less than the LLD listed on the appropriate table.

DRESDEN NUCLEAR POWER STATION  
 UNITS 1, 2, AND 3  
 EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
April Through June 1992

Radwaste LIQUID EFFLUENTS Docket Numbers: 50-10  
 50-237  
 50-249

NUCLIDES RELEASED	UNIT	BATCH MODE			2nd QUARTER TOTAL
		APRIL	MAY	JUNE	
Sr-89	C1	•	•	7.81E-06	7.81E-06
Sr-90	C1	6.09E-06	5.81E-07	7.23E-07	7.39E-06
Mn-54	C1	4.97E-04	2.10E-04	2.34E-04	9.36E-04
Co-58	C1	•	•	•	•
Fe-59	C1	•	•	•	•
Co-60	C1	1.16E-03	5.86E-04	7.69E-04	2.52E-03
Zn-65	C1	•	•	•	•
Sb-124	C1	•	•	•	•
I-131	C1	•	•	•	•
Cr-134	C1	•	•	•	•
Cs-137	C1	1.27E-03	2.88E-04	5.24E-04	2.08E-03
Ra-140	C1	•	•	•	•
La-140	C1	•	•	•	•
Se-141	C1	•	•	•	•
Ca-138	C1	•	•	•	•
Fe-55	C1	1.99E-04	1.27E-04	4.18E-04	7.44E-04
I-132	C1	•	•	•	•
	C1	•	•	•	•
(above)					
Total For Period	C1	3.13E-03	3.21E-03	1.95E-03	6.30E-03
Xe-133	C1	•	•	•	•
Xe-135	C1	•	•	•	•

\* The activity of this nuclide is less than the LLD listed on the appropriate table.

DRESDEN NUCLEAR POWER STATION  
UNITS 2 AND 3  
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
JANUARY Through June 1992

CCSW LIQUID EFFLUENTS Docket Numbers: SD-217  
SD-249

1. Number of Batch Releases: 72
2. Total Time Period for Batch Releases: 8.93E+01 min
3. Maximum Time Period for a Batch Release: 1.24E+00 min
4. Average Time Period for Batch Releases: 1.24E+00 min
5. Minimum Time Period for a Batch Release: 1.24E+00 min
6. Average Stream Flow During Periods of Release of Effluent into a Flowing Stream: 1.94E+06 L/min

NUCLIDES RELEASED	UNIT	CONTINUOUS MODE		BATCH MODE	
		1st QUARTER	2nd QUARTER	1st QUARTER	2nd QUARTER
Sr-89	C1			•	•
Sr-90	C1			1.80E-06	7.75E-07
Mn-54	C1			•	•
Co-58	C1			•	•
Fr-59	C1			•	•
Co-60	C1			1.11E-05	2.91E-06
Zn-65	C1			•	•
Sn-112	C1			•	•
Sb-124	C1			•	•
I-131	C1			•	•
I-132	C1			•	•
I-135	C1			•	•
Cs-136	C1			•	•
Cs-137	C1			2.60E-06	6.39E-06
Ra-140	C1			•	•
La-140	C1			•	•
Ca-138	C1			•	•
Fe-55	C1			•	•
	C1				
(above)					
Total For Period	C1	NONE	NONE	1.55E-05	1.01E-05
Xe-133	C1			•	•
Xe-135	C1			•	•

\* The activity of this nuclide is less than the LLD listed on the appropriate table.

**DRESDEN NUCLEAR POWER STATION  
UNITS 2 AND 3  
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
January Through March 1991**

CCSW LIQUID EFFLUENTS Docket Numbers: 50-237  
50-249

NUCLIDES RELEASED	UNIT	BATCH MODE			1st QUARTER TOTAL
		JANAUARY	FEBRUARY	MARCH	
Sr-89	C1	•	•	•	•
Sr-90	C1	•	5.11E-07	1.29E-06	1.80E-06
Mn-54	C1	•	•	•	•
Co-58	C1	•	•	•	•
Fe-59	C1	•	•	•	•
Co-60	C1	5.31E-06	2.07E-06	3.68E-06	1.11E-05
Zn-65	C1	•	•	•	•
Sb-122	C1	•	•	•	•
Sb-124	C1	•	•	•	•
I-131	C1	•	•	•	•
I-122	C1	•	•	•	•
I-135	C1	•	•	•	•
Cr-134	C1	•	•	•	•
Ca-137	C1	•	6.60E-07	1.94E-06	2.60E-06
Ra-130	C1	•	•	•	•
La-130	C1	•	•	•	•
Ca-138	C1	•	•	•	•
Fe-55	C1	•	•	•	•
	C1				
(above)					
Total For Period	C1	5.31E-06	1.24E-06	6.91E-06	1.55E-05
Xe-133	C1	•	•	•	•
Xe-135	C1	•	•	•	•

- The activity of this nuclide is less than the LLD listed on the appropriate table.

**DRESDEN NUCLEAR POWER STATION  
UNITS 2 AND 3  
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT  
APRIL Through JUNE 1972**

CCSW LIQUID EFFLUENTS Docket Numbers: 50-237  
50-249

NUCLIDES RELEASED	UNIT	BATCH MODE			2nd QUARTER TOTAL
		APRIL	MAY	JUNE	
Sr-89	C1	•	•	•	•
Sr-90	C1	6.76E-07	9.80E-08	•	7.75E-07
Mn-54	C1	•	•	•	•
Co-58	C1	•	•	•	•
Fe-59	C1	•	•	•	•
Co-60	C1	2.91E-06	•	•	2.91E-06
Zn-65	C1	•	•	•	•
Sb-122	C1	•	•	•	•
Sb-124	C1	•	•	•	•
I-131	C1	•	•	•	•
I-132	C1	•	•	•	•
I-135	C1	•	•	•	•
Cs-134	C1	•	•	•	•
Cs-137	C1	1.37E-05	2.30E-06	2.66E-06	6.39E-06
Ba-140	C1	•	•	•	•
Ka-140	C1	•	•	•	•
Cr-138	C1	•	•	•	•
Fe-55	C1	•	•	•	•
	C1				
(above)					
Total For Period	C1	4.96E-06	2.46E-06	2.66E-06	1.01E-05
Xe-133	C1	•	•	•	•
Xe-135	C1	•	•	•	•

- The activity of this nuclide is less than the LLD listed on the appropriate table.

**DRESDEN NUCLEAR POWER STATION  
UNITS 1, 2, AND 3  
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT**

January Through June 1992

Docket Numbers: 50-10  
50-237  
50-249

## SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

Est Tot.  
Error

**A. SOLID WASTE SHIPPED OFFSITE FOR BURIAL OR DISPOSAL (NOT IRRADIATED FUEL)**

1. Type of Waste	Unit: 6-month period
a. Spent resins, filter sludges, evaporator bottoms, etc.	mJ 2.04E+02 Cl 1.70E+03 12.4
b. Dry compressible waste, contaminated equip., etc.	mJ 9.95E+02 Cl 2.99E+01 16.6
c. Irradiated components, control rods, etc.	mJ 0.00E+00 Cl 0.00E+00 16.6
d. Other (describe)	mJ 0.00E+00 Cl 0.00E+00

#### 2. Estimate of Major Nuclide Composition (by type of waste)

			C1
a.	Co-60	66.6	1.13E+03
	Fe-55	1.74	2.28E+03
	Mn-54	19.2	2.41E+02
	Ni-63	11.6	1.97E+02
	Co-137	3.02	5.13E+01
b.	Co-60	21.2	6.34E+00
	Fe-55	64.4	1.93E+01
	Mn-54	6.15	1.84E+00
	Ni-59	1.32	4.01E-01
	Ni-63	1.02	3.05E-01
	Co-137	5.14	1.55E+00

### 3. Solid Waste Disposition

<u>NUMBER OF SHIPMENTS</u>	<u>MODE OF TRANSPORTATION</u>	<u>DESTINATION</u>
43	Motor freight (exclusive use only)	CNSI, Barnwell, SC
4	Motor freight (exclusive use only)	Cuadrex, Oak Ridge, TN
9	Motor freight (exclusive use only)	CNSI, Channahon, IL
7	Motor freight (exclusive use only)	SEG, Oak Ridge, TN
23	Motor freight (exclusive use only)	US Ecology, Beatty, NV

**B. IRRADIATED FUEL SHIPMENTS (Disposition)**

<u>NUMBER OF SHIPMENTS</u>	<u>MODE OF TRANSPORTATION</u>	<u>DESTINATION</u>
:	Motor Freight (Highway Route Controlled)	Batcock and Wilcox Lynchburg, VA.

DRESDEN NUCLEAR POWER STATION  
UNITS 1, 2, AND 3  
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT

JANUARY Through JUNE 1992

ABNORMAL RELEASES

A. LIQUID

1. Number of Releases: 0
2. Total Activity Released: 0

B. GASEOUS

1. Number of Releases: 4
2. Total Activity Released: 2.45E-03 Ci

TOTAL 4

1. On January 9, 1992, a backup sample pump was placed in service for Unit 1 chimney at 13:30. The sampler was discovered with a blown fuse on January 10, 1992 at 08:00. Sampling was re-established at 09:30. The estimated activity released is:

Cs-137	1.10E-06 Ci
Sr-90	4.09E-10 Ci
Gross Alpha	1.95E-08 Ci
H-3	5.99E-05 Ci

2. On February 11, 1992, the Units 2/3 Reactor Building Ventilation Stack Samples were not analyzed for the period from 08:25 to 16:15. The primary sampler (SPING) was out of service for modification work on a motor control center during this period. At the filter pull on February 18, 1992, the backup filters were not identified as the proper samples and therefore they were not analyzed. Corrective actions entail analyzing primary and backup samples at each filter pull (LER 92-09). The estimated activity released is:

I-133	5.22E-07 Ci
Mn-54	1.93E-06 Ci
Co-58	5.38E-07 Ci
Co-60	7.06E-06 Ci
Zn-65	1.04E-07 Ci
Fe-55	6.39E-06 Ci
Sr-89	4.27E-09 Ci
Sr-90	9.20E-10 Ci
Gross Alpha	7.22E-09 Ci
H-3	9.43E-04 Ci

DRESDEN NUCLEAR POWER STATION  
UNITS 1, 2 AND 3  
EFFLUENT AND WASTE DISPOSAL SEMI-ANNUAL REPORT  
January Through June 1992  
ABNORMAL RELEASES

3. The Unit 1 Chimney Backup Sampler was found with a blown fuse on April 9, 1992 at 07:15. The sampler was last known to be running at 15:45 on April 8, 1992. The blown fuse was discovered to be the wrong size. All samplers of this type were then checked for proper fuses. Correct size fuses are currently installed. (DVR #12-1-92-04) The estimated activity during the unsampled period is:

Fe-55	2.11E-08 Ci
Co-60	1.50E-06 Ci
Sr-89	2.69E-09 Ci
H-3	4.65E-05 Ci

4. On March 22, 1992 at 01:00, the Unit 2/3 Chimney G.E. System (backup sampler) flow was restricted by a failed fuse for a solenoid, causing a valve to close. The 2/3 Chimney SPING was placed in service at 03:35. The estimated activity during this period is:

Fe-55	1.16E-06 Ci
Sr-89	2.88E-05 Ci
H-3	1.35E-03 Ci

DRESDEN NUCLEAR POWER STATION DOCKET NOS. 50-10, 50-237, 50-249

RADIOLOGICAL IMPACT ON MAN

DRESDEN UNIT ONE

1992 ANNUAL REPORT  
 MAXIMUM DOSES RESULTING FROM AIRBORNE RELEASES  
 PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/22/93  
 INFANT RECEPTOR

TYPE	1ST QUARTER JAN-MAR	2ND QUARTER APR-JUN	3RD QUARTER JUL-SEP	4TH QUARTER OCT-DEC	ANNUAL
GAMMA AIR (MRAD)	0.00E+00 ( )	0.00E+00 ( )	0.00E+00 ( )	0.00E+00 ( )	0.00E+00 ( )
BETA AIR (MRAD)	0.00E+00 ( )	0.00E+00 ( )	0.00E+00 ( )	0.00E+00 ( )	0.00E+00 ( )
TOT. BODY (MREM)	0.00E+00 ( )	0.00E+00 ( )	0.00E+00 ( )	0.00E+00 ( )	0.00E+00 ( )
SKIN (MREM)	0.00E+00 ( )	0.00E+00 ( )	0.00E+00 ( )	0.00E+00 ( )	0.00E+00 ( )
ORGAN (MREM)	1.07E-05 (SE )	7.68E-06 (SE )	1.14E-06 (N )	1.11E-06 (N )	2.04E-05 (SE )
	LIVER	LIVER	LUNG	LIVER	LIVER

THIS IS A REPORT FOR THE CALENDAR YEAR 1992

COMPLIANCE STATUS - 10 CFR 50 APP. I  
 INFANT RECEPTOR

QTRLY OBJ	% OF APP I.				YRLY OBJ	% OF APP.I
	1ST QTR JAN-MAR	2ND QTR APR-JUN	3RD QTR JUL-SEP	4TH QTR OCT-NOV		
GAMMA AIR (MRAD)	5.0	0.00	0.00	0.00	10.0	0.00
BETA AIR (MRAD)	10.0	0.00	0.00	0.00	20.0	0.00
TOT. BODY (MREM)	2.5	0.00	0.00	0.00	5.0	0.00
SKIN (MREM)	7.5	0.00	0.00	0.00	15.0	0.00
ORGAN (MREM)	7.5	0.00	0.00	0.00	15.0	0.00
	LIVER	LIVER	LUNG	LIVER	LIVER	

RESULTS BASED UPON  
 ODCM ANNEX  
 REVISION 0  
 MARCH 1989

DRESDEN UNIT ONE

1992 ANNUAL REPORT  
 MAXIMUM DOSES RESULTING FROM AIRBORNE RELEASES  
 PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/22/93  
 ADULT RECEPTOR

TYPE	1ST QUARTER JAN-MAR	2ND QUARTER APR-JUN	3RD QUARTER JUL-SEP	4TH QUARTER OCT-DEC	ANNUAL
GAMMA AIR (MRAD)	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
BETA AIR (MRAD)	( )	( )	( )	( )	( )
TOT. BODY (MREM)	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
SKIN (MREM)	0.00E+00	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ORGAN (MREM)	9.44E-06 (SE )	1.32E-05 (SE )	6.78E-06 (SE )	4.40E-06 (SE )	3.39E-05 (SE )
	LIVER	LIVER	LIVER	LIVER	LIVER

THIS IS A REPORT FOR THE CALENDAR YEAR 1992

COMPLIANCE STATUS - 10 CFR 50 APP. I  
 ADULT RECEPTOR

QTRLY OBJ	% OF APP I.				YRLY OBJ	% OF APP.I
	1ST QTR JAN-MAR	2ND QTR APR-JUN	3RD QTR JUL-SEP	4TH QTR OCT-NOV		
GAMMA AIR (MRAD)	5.0	0.00	0.00	0.00	0.00	10.0 0.00
BETA AIR (MRAD)	10.0	0.00	0.00	0.00	0.00	20.0 0.00
TOT. BODY (MREM)	2.5	0.00	0.00	0.00	0.00	5.0 0.00
SKIN (MREM)	7.5	0.00	0.00	0.00	0.00	15.0 0.00
ORGAN (MREM)	7.5	0.00	0.00	0.00	0.00	15.0 0.00
	LIVER	LIVER	LIVER	LIVER	LIVER	

RESULTS BASED UPON  
 ODCM ANNEX  
 REVISION 0  
 MARCH 1989

l:p

DRESDEN UNIT TWO

1992 ANNUAL REPORT  
 MAXIMUM DOSES RESULTING FROM AIRBORNE RELEASES  
 PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/26/93  
 INFANT RECEPTOR

TYPE	1ST QUARTER JAN-MAR	2ND QUARTER APR-JUN	3RD QUARTER JUL-SEP	4TH QUARTER OCT-DEC	ANNUAL
GAMMA AIR (MRAD)	1.47E-04 (NE )	5.95E-06 (NE )	3.83E-06 (NE )	1.05E-05 (NE )	1.68E-04 (NE )
BETA AIR (MRAD)	2.00E-05 (N )	1.25E-06 (N )	8.07E-07 (N )	2.20E-06 (N )	2.43E-05 (N )
TOT. BODY (MREM)	7.20E-05 (HE )	2.50E-06 (NE )	1.61E-06 (NE )	4.40E-06 (HE )	8.05E-05 (NE )
SKIN (MREM)	1.30E-04 (NE )	5.47E-06 (NE )	3.52E-06 (NE )	9.61E-06 (NE )	1.48E-04 (NE )
ORGAN (MREM)	1.47E-02 (NNE )	8.65E-03 (NNE )	8.59E-04 (NNE )	1.72E-03 (NNE )	2.58E-02 (NNE )
	LUNG	LUNG	THYROID	LUNG	LUNG

THIS IS A REPORT FOR THE CALENDAR YEAR 1992

COMPLIANCE STATUS - 10 CFR 50 APP. I  
 INFANT RECEPTOR

QTRLY OBJ	1 OF APP I.				YRLY OBJ	1 OF APP.I
	1ST QTR JAN-MAR	2ND QTR APR-JUN	3RD QTR JUL-SEP	4TH QTR OCT-NOV		
GAMMA AIR (MRAD)	5.0	0.00	0.00	0.00	0.00	10.0 0.00
BETA AIR (MRAD)	10.0	0.00	0.00	0.00	0.00	20.0 0.00
TOT. BCDY (MREM)	2.5	0.00	0.00	0.00	0.00	5.0 0.00
SKIN (MREM)	7.5	0.00	0.00	0.00	0.00	15.0 0.00
ORGAN (MREM)	7.5	0.20	0.12	0.01	0.02	15.0 0.17
	LUNG	LUNG	THYROID	LUNG	LUNG	

RESULTS BASED UPON  
 ODCM ANNEX  
 REVISION 0  
 MARCH 1989

DRESDEN UNIT TWO

1992 ANNUAL REPORT  
 MAXIMUM DOSES RESULTING FROM AIRBORNE RELEASES  
 PERIOD OF RELEASE - 01/01/92 TO 12/31/92    CALCULATED 02/26/93  
 ADULT RECEPTOR

TYPE	1ST QUARTER JAN-MAR	2ND QUARTER APR-JUN	3RD QUARTER JUL-SEP	4TH QUARTER OCT-DEC	ANNUAL
GAMMA AIR (MRAD)	1.47E-04 (NE )	5.95E-06 (NE )	3.83E-06 (NE )	1.05E-05 (NE )	1.68E-04 (NE )
BETA AIR (MRAD)	2.00E-05 (N )	1.25E-06 (N )	8.07E-07 (N )	2.20E-06 (N )	2.43E-05 (N )
TOT. BODY (MREM)	7.20E-05 (NE )	2.50E-06 (NE )	1.61E-06 (NE )	4.40E-06 (NE )	8.05E-05 (NE )
SKIN (MREM)	1.30E-04 (NE )	5.47E-06 (NE )	3.52E-06 (NE )	9.51E-06 (NE )	1.48E-04 (NE )
ORGAN (MREM)	1.50E-02 (NHE )	8.98E-03 (NHE )	1.01E-03 (NHE )	1.89E-03 (NHE )	2.67E-02 (NHE )
	LUNG	LUNG	THYROID	LUNG	LUNG

THIS IS A REPORT FOR THE CALENDAR YEAR 1992

COMPLIANCE STATUS - 10 CFR 50 APP. I  
 ADULT RECEPTOR

QTRLY OBJ	----- 1 OF APP I. -----				YRLY OBJ	1 OF APP.I
	1ST QTR JAN-MAR	2ND QTR APR-JUN	3RD QTR JUL-SEP	4TH QTR OCT-NOV		
GAMMA AIR (MRAD)	5.0	0.00	0.00	0.00	0.00	10.0
BETA AIR (MRAD)	10.0	0.00	0.00	0.00	0.00	20.0
TOT. BODY (MREM)	2.5	0.00	0.00	0.00	0.00	5.0
SKIN (MREM)	7.5	0.00	0.00	0.00	0.00	15.0
ORGAN (MREM)	7.5	0.20	0.12	0.01	0.03	15.0
	LUNG	LUNG	THYROID	LUNG	LUNG	

RESULTS BASED UPON:  
 ODCH ANNEX  
 REVISION 0  
 MARCH 1989

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DRESDEN UNIT THREE

1992 ANNUAL REPORT  
 MAXIMUM DOSES RESULTING FROM AIRBORNE RELEASES  
 PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/26/93  
 INFANT RECEPTOR

TYPE	1ST QUARTER JAN-MAR	2ND QUARTER APR-JUN	3RD QUARTER JUL-SEP	4TH QUARTER OCT-DEC	ANNUAL
GAMMA AIR (MRAD)	0.00E+00	9.59E-06	1.45E-05	3.35E-05	5.75E-05
BETA AIR (MRAD)	( )	(NE )	(NE )	(NE )	(NE )
TOT. BODY (MREM)	0.00E+00	2.02E-06	3.05E-06	7.05E-06	1.21E-05
SKIN (MREM)	( )	(N )	(N )	(N )	(N )
ORGAN (MREM)	0.00E+00	4.03E-06	6.08E-06	1.41E-05	2.42E-05
	( )	(NE )	(NE )	(NE )	(NE )
	0.00E+00	8.81E-06	1.33E-05	3.07E-05	5.28E-05
	( )	(NE )	(NE )	(NE )	(NE )
	6.82E-04	1.21E-02	4.32E-03	9.86E-03	2.66E-02
	(NNE )	(NNE )	(NNE )	(NNE )	(NNE )
	LUNG	LUNG	THYROID	LUNG	THYROID

THIS IS A REPORT FOR THE CALENDAR YEAR 1992

COMPLIANCE STATUS - 10 CFR 50 APP. I  
 INFANT RECEPTOR

QTRLY OBJ	1 OF APP I.				YRLY OBJ	1 OF APP.I
	1ST QTR JAN-MAR	2ND QTR APR-JUN	3RD QTR JUL-SEP	4TH QTR OCT-NOV		
GAMMA AIR (MRAD)	5.0	0.00	0.00	0.00	0.00	10.0
BETA AIR (MRAD)	10.0	0.00	0.00	0.00	0.00	20.0
TOT. BODY (MREM)	2.5	0.00	0.00	0.00	0.00	5.0
SKIN (MREM)	7.5	0.00	0.00	0.00	0.00	15.0
ORGAN (MREM)	7.5	0.01	0.16	0.06	0.13	15.0
	LUNG	LUNG	THYROID	LUNG		THYROID

RESULTS BASED UPON  
 ODCM ANNEX  
 REVISION 0  
 MARCH 1989

DRESDEN UNIT THREE

1992 ANNUAL REPORT  
 MAXIMUM DOSES RESULTING FROM AIRBORNE RELEASES  
 PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/26/93  
 ADULT RECEPTOR

TYPE	1ST QUARTER JAN-MAR	2ND QUARTER APR-JUN	3RD QUARTER JUL-SEP	4TH QUARTER OCT-DEC	ANNUAL
GAMMA AIR (MRAD)	0.00E+00	9.59E-06	1.45E-05	3.35E-05	5.75E-05
BETA AIR (MRAD)	( )	(NE )	(NE )	(NE )	(NE )
TOT. BODY (MREM)	0.00E+00	2.02E-06	3.05E-06	7.05E-06	1.21E-05
SKIN (MREM)	( )	(N )	(N )	(N )	(N )
ORGAN (MREM)	0.00E+00	4.03E-06	6.08E-06	1.41E-05	2.42E-05
	( )	(NE )	(NE )	(NE )	(NE )
	0.00E+00	8.81E-06	1.33E-05	3.07E-05	5.28E-05
	( )	(NE )	(NE )	(NE )	(NE )
	6.92E-04	1.37E-02	5.04E-03	1.05E-02	2.89E-02
	(NNE )	(NNE )	(NNE )	(NNE )	(NNE )
	LUNG	GI-LLI	THYROID	LUNG	GI-LLI

THIS IS A REPORT FOR THE CALENDAR YEAR 1992

COMPLIANCE STATUS - 10 CFR 50 APP. I  
 ADULT RECEPTOR

QTRLY OBJ	% OF APP I.				YRLY OBJ	% OF APP.I
	1ST QTR JAN-MAR	2ND QTR APR-JUN	3RD QTR JUL-SEP	4TH QTR OCT-NOV		
GAMMA AIR (MRAD)	5.0	0.00	0.00	0.00	0.00	10.0 0.00
BETA AIR (MRAD)	10.0	0.00	0.00	0.00	0.00	20.0 0.00
TOT. BODY (MREM)	2.5	0.00	0.00	0.00	0.00	5.0 0.00
SKIN (MREM)	7.5	0.00	0.00	0.00	0.00	15.0 0.00
ORGAN (MREM)	7.5	0.01	0.18	0.07	0.14	15.0 0.19
	LUNG	GI-LLI	THYROID	LUNG		GI-LLI

RESULTS BASED UPON:  
 ODCM ANNEX  
 REVISION 0  
 MARCH 1989

11P

DRESDEN UNIT TWO  
INFANT RECEPTOR

1992 ANNUAL REPORT  
MAXIMUM DOSES (MREM) RESULTING FROM LIQUID EFFLUENTS  
PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/24/93

DOSE TYPE	1ST QUARTER JAN-MAR	2ND QUARTER APR-JUN	3RD QUARTER JUL-SEP	4TH QUARTER OCT-DEC	ANNUAL
TOTAL BODY INTERNAL ORGAN	1.09E-05 2.67E-05	9.95E-06 2.65E-05	5.41E-06 6.43E-06	2.21E-06 3.22E-05	2.85E-05 6.29E-05
	LIVER	LIVER	LIVER	LIVER	LIVER

THIS IS A REPORT FOR THE CALENDAR YEAR 1992  
COMPLIANCE STATUS - 10 CFR 50 APP. I

QTRLY OBJ	----- % OF APP I. -----				YRLY OBJ	% OF APP.I	
	1ST QTR JAN-MAR	2ND QTR APR-JUN	3RD QTR JUL-SEP	4TH QTR OCT-NOV			
TOTAL BODY (MREM)	1.5	0.00	0.00	0.00	0.00	3.0	0.00
CRIT. ORGAN(MREM)	5.0	0.00	0.00	0.00	0.00	10.0	0.00
	LIVER	LIVER	LIVER	LIVER	LIVER	LIVER	

RESULTS BASED UPON  
ODCM ANNEX  
REVISION 0  
MARCH 1989

DRESDEN UNIT TWO  
INFANT RECEPTOR

1992 ANNUAL REPORT  
PROJECTED DOSE AT NEAREST COMMUNITY WATER SYSTEM \*  
PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/24/93

DOSE TYPE	1ST QUARTER JAN-MAR	2ND QUARTER APR-JUN	3RD QUARTER JUL-SEP	4TH QUARTER OCT-DEC	ANNUAL
TOTAL BODY INTERNAL ORGAN	1.09E-05 2.67E-05 LIVER	9.95E-06 2.65E-05 LIVER	5.41E-06 6.43E-06 LIVER	2.21E-06 3.22E-06 LIVER	2.85E-05 6.29E-05 LIVER

THIS IS A REPORT FOR THE CALENDAR YEAR 1992

COMPLIANCE STATUS - 40 CFR 141

TYPE	ANNUAL LIMIT	% OF LIMIT
TOTAL BODY INTERNAL ORGAN	4.0 (MREM) 4.0 (MREM)	0.001 0.002 LIVER

\* THIS CALCULATION OF DOSE IS BASED ON TECHNIQUES DESCRIBED IN THE COMMONWEALTH EDISON OFFSITE DOSE CALCULATION MANUAL. THESE TECHNIQUES DIFFER FROM THOSE DESCRIBED IN 40 CFR 141.

RESULTS BASED UPON  
OLCM ANNEX  
REVISION 0  
MARCH 1989

DRESDEN UNIT TWO  
ADULT RECEPTOR

1992 ANNUAL REPORT  
MAXIMUM DOSES (MREM) RESULTING FROM LIQUID EFFLUENTS  
PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/24/93

DOSE TYPE	1ST QUARTER JAN-MAR	2ND QUARTER APR-JUN	3RD QUARTER JUL-SEP	4TH QUARTER OCT-DEC	ANNUAL
TOTAL BODY	3.51E-05	3.70E-05	3.91E-06	2.76E-06	7.87E-05
INTERNAL ORGAN	5.23E-05	5.47E-05	5.01E-06	3.83E-06	1.16E-04
	LIVER	LIVER	LIVER	LIVER	LIVER

THIS IS A REPORT FOR THE CALENDAR YEAR 1992

COMPLIANCE STATUS - 10 CFR 50 APP. I

QTRLY OBJ	1 OF APP I.					YRLY OBJ	% OF APP.
	1ST QTR JAN-MAR	2ND QTR APR-JUN	3RD QTR JUL-SEP	4TH QTR OCT-NOV			
TOTAL BODY (MREM)	1.5	0.00	0.00	0.00	0.00	3.0	0.0
CRIT. ORGAN(MREM)	5.0	0.00	0.00	0.00	0.00	10.0	0.0
	LIVER	LIVER	LIVER	LIVER	LIVER		

RESULTS BASED UP  
ODCM ANNEX  
REVISION 0  
MARCH 1989

DRESDEN UNIT TWO  
ADULT RECEPTOR

1992 ANNUAL REPORT  
PROJECTED DOSE AT NEAREST COMMUNITY WATER SYSTEM \*  
PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/24/93

DOSE TYPE	1ST QUARTER JAN-MAR	2ND QUARTER APR-JUN	3RD QUARTER JUL-SEP	4TH QUARTER OCT-DEC	ANNUAL
TOTAL BODY	4.94E-06	5.00E-06	1.93E-06	8.46E-07	1.27E-05
INTERNAL ORGAN	7.03E-06	5.90E-06	2.03E-06	9.11E-07	1.48E-05
	GI-LLI	LIVER	GI-LLI	LIVER	LIVER

THIS IS A REPORT FOR THE CALENDAR YEAR 1992

COMPLIANCE STATUS - 40 CFR 141

TYPE	ANNUAL LIMIT	% OF LIMIT
TOTAL BODY	4.0 (MREM)	0.000
INTERNAL ORGAN	4.0 (MREM)	0.000
		LIVER

\* THIS CALCULATION OF DOSE IS BASED ON TECHNIQUES DESCRIBED IN THE COMMONWEALTH EDISON OFFSITE DOSE CALCULATION MANUAL. THESE TECHNIQUES DIFFER FROM THOSE DESCRIBED IN 40 CFR 141.

RESULTS BASED UPON:  
ODCM ANNEX  
REVISION 0  
MARCH 1989

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DRESDEN UNIT THREE  
INFANT RECEPTOR

1992 ANNUAL REPORT  
MAXIMUM DOSES (MREM) RESULTING FROM LIQUID EFFLUENTS  
PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/24/93

DOSE TYPE	1ST QUARTER	2ND QUARTER	3RD QUARTER	4TH QUARTER	ANNUAL
	JAN-MAR	APR-JUN	JUL-SEP	OCT-DEC	
TOTAL BODY	1.19E-05	1.00E-05	5.38E-06	2.19E-06	2.95E-05
INTERNAL ORGAN	3.03E-05	2.66E-05	6.31E-06	2.69E-06	6.60E-05
	LIVER	LIVER	LIVER	LIVER	LIVER

THIS IS A REPORT FOR THE CALENDAR YEAR 1992  
COMPLIANCE STATUS - 10 CFR 50 APP. I

QTRLY OBJ	----- % OF APP I. -----				YRLY OBJ	% OF APP.I	
	1ST QTR JAN-MAR	2ND QTR APR-JUN	3RD QTR JUL-SEP	4TH QTR OCT-NOV			
TOTAL BODY (MREM)	1.5	0.00	0.00	0.00	0.00	3.0	0.00
CRIT. ORGAN(MREM)	5.0	0.00	0.00	0.00	0.00	10.0	0.00
	LIVER	LIVER	LIVER	LIVER		LIVER	

RESULTS BASED UPON  
ODCM ANNEX  
REVISION 0  
MARCH 1989

DRESDEN UNIT THREE  
INFANT RECEPTOR

1992 ANNUAL REPORT  
PROJECTED DOSE AT NEAREST COMMUNITY WATER SYSTEM •  
PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/24/93

DOSE TYPE	1ST QUARTER JAN-MAR	2ND QUARTER APR-JUN	3RD QUARTER JUL-SEP	4TH QUARTER OCT-DEC	ANNUAL
TOTAL BODY	1.19E-05	1.00E-05	5.38E-06	2.19E-06	2.95E-05
INTERNAL ORGAN	3.03E-05	2.66E-05	6.31E-06	2.69E-06	6.60E-05
	LIVER	LIVER	LIVER	LIVER	LIVER

THIS IS A REPORT FOR THE CALENDAR YEAR 1992

COMPLIANCE STATUS - 40 CFR 141

TYPE	ANNUAL LIMIT	% OF LIMIT
TOTAL BODY	4.0 (MREM)	0.001
INTERNAL ORGAN	4.0 (MREM)	0.002
		LIVER

\* THIS CALCULATION OF DOSE IS BASED ON TECHNIQUES DESCRIBED IN THE COMMONWEALTH EDISON OFFSITE DOSE CALCULATION MANUAL. THESE TECHNIQUES DIFFER FROM THOSE DESCRIBED IN 40 CFR 141.

RESULTS BASED UPON  
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MARCH 1989

DRESDEN UNIT THREE  
ADULT RECEPTOR

1992 ANNUAL REPORT  
MAXIMUM DOSES (MREM) RESULTING FROM LIQUID EFFLUENTS  
PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/24/93

DOSE TYPE	1ST QUARTER	2ND QUARTER	3RD QUARTER	4TH QUARTER	ANNUAL
	JAN-MAR	APR-JUN	JUL-SEP	OCT-DEC	
TOTAL BODY	4.13E-05	3.71E-05	3.71E-06	1.78E-06	8.39E-05
INTERNAL ORGAN	6.16E-05	5.49E-05	4.72E-06	2.34E-06	1.24E-04
	LIVER	LIVER	LIVER	LIVER	LIVER

THIS IS A REPORT FOR THE CALENDAR YEAR 1992

COMPLIANCE STATUS - 10 CFR 50 APP. I

QTRLY OBJ	1 OF APP I.				YRLY OBJ	% OF APP.I	
	1ST QTR JAN-MAR	2ND QTR APR-JUN	3RD QTR JUL-SEP	4TH QTR OCT-NOV			
TOTAL BODY (MREM)	1.5	0.00	0.00	0.00	0.00	3.0	0.00
CRIT. ORGAN(MREM)	5.0	0.00	0.00	0.00	0.00	10.0	0.00
	LIVER	LIVER	LIVER	LIVER	LIVER		

RESULTS BASED UPON  
ODCM ANNEX  
REVISION 0  
MARCH 1989

DRESDEN UNIT THREE  
ADULT RECEPTOR

1992 ANNUAL REPORT  
PROJECTED DOSE AT NEAREST COMMUNITY WATER SYSTEM \*  
PERIOD OF RELEASE - 01/01/92 TO 12/31/92 CALCULATED 02/26/93

DOSE TYPE	1ST QUARTER JAN-MAR	2ND QUARTER APR-JUN	3RD QUARTER JUL-SEP	4TH QUARTER OCT-DEC	ANNUAL
TOTAL BODY INTERNAL ORGAN	5.44E-06 8.49E-06	5.03E-06 5.91E-06	1.91E-06 2.00E-06	7.86E-07 8.56E-07	1.32E-05 1.55E-05
	GI-LLI	LIVER	GI-LLI	GI-LLI	GI-LLI

THIS IS A REPORT FOR THE CALENDAR YEAR 1992

COMPLIANCE STATUS - 40 CFR 141

TYPE	ANNUAL LIMIT	% OF LIMIT
TOTAL BODY INTERNAL ORGAN	4.0 (MREM)	0.000
	4.0 (MREM)	0.000
		GI-LLI

\* THIS CALCULATION OF DOSE IS BASED ON TECHNIQUES DESCRIBED IN THE COMMONWEALTH EDISON OFFSITE DOSE CALCULATION MANUAL. THESE TECHNIQUES DIFFER FROM THOSE DESCRIBED IN 40 CFR 141.

RESULTS BASED UPON  
ODCM ANNEX  
REVISION 0  
MARCH 1989

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DRESDEN NUCLEAR POWER STATION DOCKET NOS. 50-10, 50-237, 50-249

METEOROLOGICAL DATA

**ELITE CLASSIC STATION**

Jacoby-Barca 1992  
132-33 SL 215720101 TERRITORY

NUMBER OF OBSERVATIONS : 2100  
NUMBER AND PERCENT ACCURACY

**CITIC OVERSEAS STATECO**

January-February 1912

#### Our Strategies to Stabilize

## **First Directions to the Seats**

Wolfgang Müller für SIEBEL  
WOLFGANG MÜLLER für SIEBEL

MINBAN NUMBER 18-01-01  
MAY 2010 EDITION

WILSON COUNCIL FOR CHILDREN WITH DISABILITIES

CELESTIAL STATION  
333 ft. HIGH SPEED ONE WAY SECTION

**Assay-Bars 1882**  
**123-35 ft. DIFFERENTIAL TERRACE**

### **THE EFFECTS OF STABILITY**

1122 Questions at 1122 Books

**WIND SPEED AND DIRECTION**

April-June 1932  
193-35 FL. DIFFERENTIAL THERMISTERS

**NUMBER OF OBSERVATIONS = 200**

~~CECO DRESSED STATION~~

April-June 1912  
100-34 ft. DIFFERENTIAL TEMPERATURE

181 1-81 1-23 1-31 1-33 1-35 1-37 1-39 1-41 1-43 1-45 1-47 1-49 1-51 1-53 1-55 1-57 1-59 1-61 1-63 1-65 1-67 1-69 1-71 1-73 1-75 1-77 1-79 1-81

#### Wet Distinct De Stabilite

### The Structure of the Society

THE PRESENT STATION  
IS IN THE CITY OF NEW YORK.

April-June 1982  
320-18 G. DIFFERENTIAL TEMPERATURE

**NUMBER OF OBSERVATIONS : 200**

DECA DRESSED STATION  
120 FT. HIGH SPEED AND DRY BURNER

April-June 1917  
110-11 St. differential thermometer

#### Use Functions to Simplify

R	ME	NE	DE	E	EE	SE	SEE	S	SSI	N	NSI	T	NTU	W	WTU	STABILITY CLASS
.00	1.01	1.02	.03	.01	.03	.00	.03	.03	.03	.03	.03	.03	.03	.01	1.01	1.02
.01	.02	1.04	.03	.01	.04	.02	.03	.03	.02	.03	.02	.04	.01	.01	.02	Federately Unstable
.02	.03	.03	.04	.01	.04	.02	.03	.03	.02	.03	.02	.04	.01	.01	.02	Slightly Unstable
.03	.04	.03	.03	.03	.03	.03	.04	.04	.03	.03	.03	.02	.03	.01	.01	Extremely Unstable
.04	.03	.03	.03	.03	.03	.03	.03	.03	.03	.03	.03	.02	.03	.01	.01	Extremely Stable
.05	.02	.03	.03	.03	.03	.03	.03	.03	.03	.03	.03	.02	.03	.01	.01	Federately Stable
.06	.01	.03	.03	.03	.03	.03	.03	.03	.03	.03	.03	.02	.03	.01	.01	Slightly Stable
.07	.00	.03	.03	.03	.03	.03	.03	.03	.03	.03	.03	.02	.03	.01	.01	Extremely Stable
.08	.00	.03	.03	.03	.03	.03	.03	.03	.03	.03	.03	.02	.03	.01	.01	Federately Stable
.09	.00	.03	.03	.03	.03	.03	.03	.03	.03	.03	.03	.02	.03	.01	.01	Slightly Stable
.10	.00	.03	.03	.03	.03	.03	.03	.03	.03	.03	.03	.02	.03	.01	.01	Extremely Stable

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