



R.E. Ginna Nuclear Power Plant

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April 9, 2014

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

R.E. Ginna Nuclear Power Plant
Renewed Facility Operating License No. DPR-18
NRC Docket No. 50-244

Subject: 10 CFR 50.46 Annual ECCS Report

Reference: (1) Westinghouse Letter LTR-LIS-14-90, "R. E. Ginna, 10 CFR 50.46 Annual Notification and Reporting for 2013," dated March 17, 2014.

In accordance with the requirements in 10 CFR 50.46 paragraph (a)(3)(ii), this annual Emergency Core Cooling System (ECCS) report is hereby submitted by R.E. Ginna Nuclear Power Plant (REG). Westinghouse, the provider of Loss of Coolant Accident (LOCA) analysis services for REG, has provided an update to peak cladding temperature (PCT) margin in Reference 1.

Attached please find Attachment 1, "R.E. Ginna Nuclear Power Plant, Westinghouse LOCA Evaluation Model Changes," which is the 2013 annual report of corrections to the REG ECCS Evaluation Models. This report summarizes changes made to both the Large-Break LOCA (LBLOCA) and Small-Break LOCA (SBLOCA) analyses. The "R.E. Ginna Nuclear Power Plant, LBLOCA and SBLOCA Peak Clad Temperature Assessment Sheets" are attached as Attachment 2.

There are no regulatory commitments contained in this submittal. Should you have any other questions regarding this submittal, please contact Thomas Harding at 585-771-5219.

Respectfully,

A handwritten signature in black ink, appearing to read "Tom Mogren".

Thomas G. Mogren
Manager – Engineering Services
R.E. Ginna Nuclear Power Plant

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NRR

April 9, 2014
U.S. Nuclear Regulatory Commission
Page 2

TGM/JPO

Attachments:

- 1) R.E. Ginna Nuclear Power Plant, Westinghouse LOCA Evaluation Model Changes
- 2) R.E. Ginna Nuclear Power Plant, LBLOCA and SBLOCA Peak Clad Temperature Assessment Sheets

cc: NRC Regional Administrator, Region I
NRC Project Manager, Ginna
NRC Senior Resident Inspector

ATTACHMENT 1

**R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES**

ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES

GENERAL CODE MAINTENANCE

Background

Various changes have been made to enhance the usability of codes and to streamline future analyses. Examples of these changes include modifying input variable definitions, units and defaults; improving the input diagnostic checks; enhancing the code output; optimizing active coding; and eliminating inactive coding. These changes represent Discretionary Changes that will be implemented on a forward-fit basis in accordance with Section 4.1.1 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

The nature of these changes leads to an estimated Peak Cladding Temperature (PCT) impact of 0°F.

ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES

BURST ELEVATION SELECTION

Background

It is stated on page 11-20 of WCAP-16009-P-A that the burst option is applied at the elevation corresponding to the (WCOBRA/TRAC) burst elevation for the hot assembly rod. This approach was modified to apply the burst option at the HOTSPOT predicted burst elevation as described on page 19 of Attachment 1 to LTR-NRC-06-8. The HOTSPOT code has been updated to incorporate the following changes to the burst elevation selection logic if multiple nodes burst at the same time: (1) the node that has the highest cladding temperature at the time of burst is selected; (2) if multiple nodes have the same burst time and cladding temperature at the time of burst, the lowest ordered elevation of those nodes is selected. These changes represent a closely-related group of Discretionary Changes in accordance with Section 4.1.1 of WCAP-13451.

Affected Evaluation Model(s)

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

This improvement in burst elevation selection is a forward-fit change, leading to an estimated Peak Cladding Temperature (PCT) impact of 0°F.

ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES

ELEVATIONS FOR HEAT SLAB TEMPERATURE INITIALIZATION

Background

An error was discovered in WCOBRA/TRAC whereby an incorrect value would be used in the initial fuel rod temperature calculation for a fuel rod heat transfer node if that node elevation was specified outside of the bounds of the temperature initialization table. This problem has been evaluated for impact on existing analyses and its resolution represents a Discretionary Change in accordance with Section 4.1.1 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

Based on inspection of plant analysis input, it was concluded that the input decks for existing analyses are not impacted by this error, leading to an estimated peak cladding temperature impact of 0°F.

ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES

HEAT TRANSFER LOGIC CORRECTION FOR ROD BURST CALCULATION

Background

A change was made to the WCOBRA/TRAC coding to correct an error which had disabled rod burst in separate effect test simulations. This change represents a Discretionary Change in accordance with Section 4.1.1 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

Based on the nature of the change and the evaluation model requirements for plant modeling in Westinghouse best estimate large break LOCA analyses with WCOBRA/TRAC, it is judged that existing analyses are not impacted by this change, leading to an estimated peak cladding temperature impact of 0°F.

**ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES**

WCOBRA/TRAC U19 FILE DIMENSION ERROR CORRECTION

Background

A problem was identified in the dimension of an array used to generate the u19 file in WCOBRA/TRAC. The u19 file is read during HSDRIVER execution and provides information needed to generate the HOTSPOT thermal-hydraulic history and user input files. The array used to write the desired information to the u19 file is dimensioned to 2000 in WCOBRA/TRAC. It is possible, however, for more than 2000 curves to be written to the u19 file. If that is the case, it is possible that the curves would not be stored correctly on the u19 file. A survey of current Best Estimate Large Break LOCA analyses indicated that the majority of plants had less than 2000 curves in their u19 files; therefore these plants are not affected by the change. For those plants with more than 2000 curves, plant-specific sensitivity calculations indicated that resolution of this issue does not impact the peak cladding temperature (PCT) calculation for prior analyses. This represents a Discretionary Change in accordance with Section 4.1.1 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

As discussed in the Background section, resolution of this issue does not impact the peak cladding temperature calculation for prior LBLOCA analyses, leading to an estimated peak cladding temperature impact of 0°F.

ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES

HEAT TRANSFER MODEL ERROR CORRECTIONS

Background

Several related changes were made to WCOBRA/TRAC to correct errors discovered which affected the heat transfer models. These errors included calculation of the entrained liquid fraction used in calculation of the drop wall heat flux, application of the grid enhancement factor for grid temperature calculation, calculation of the Reynold's number used in the Wong-Hochrieter correlation for the heat transfer coefficient from fuel rods to vapor, fuel rod initialization and calculation of cladding inner radius with creep, application of grid and two phase enhancement factors and radiation component in single phase vapor heat transfer, and reset of the critical heat flux temperature when J=2. These errors have been evaluated to estimate the impact on existing LBLOCA analysis results. Correction of these errors represents a closely-related group of Non-Discretionary Changes in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

Based on the results of representative plant calculations, separate effects and integral effects test simulations, it is concluded that the error corrections have a negligible local effect on heat transfer, leading to an estimated peak cladding temperature impact of 0°F.

**ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES**

CORRECTION TO HEAT TRANSFER NODE INITIALIZATION

Background

An error was discovered in the heat transfer node initialization logic in WCOBRA/TRAC whereby the heat transfer node center locations could be inconsistent with the geometric node center elevations. The primary effects of this issue are on the interpolated fluid properties and grid turbulent mixing enhancement at the heat transfer node. This problem has been evaluated for impact on existing analyses and its resolution represents a Non-Discretionary Change in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

Based on engineering judgment and the results from a matrix of representative plant calculations, it is concluded that the effect of this error is within the code resolution, leading to an estimated peak cladding temperature impact of 0°F.

ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES

MASS CONSERVATION ERROR FIX

Background

It was identified that mass was not conserved in WCOBRA/TRAC one-dimensional component cells when void fraction values were calculated to be slightly out of the physical range (greater than 1.0 or smaller than 0.0). This was observed to result in artificial mass generation on the secondary side of steam generator components. Correction of this problem represents a Non-Discretionary Change in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

This error was observed to primarily affect the mass on the secondary side of the steam generator. This issue was judged to have a negligible impact on existing LBLOCA analysis results, leading to an estimated peak cladding temperature impact of 0°F.

ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES

CORRECTION TO SPLIT CHANNEL MOMENTUM EQUATION

Background

An error was discovered in the momentum equation calculations for split channels in WCOBRA/TRAC. This error impacts the (1) continuity area of the phantom/boundary bottom cell; (2) bottom and top continuity area correction factors for the channel inlet at the bottom of a section and for the channel outlet at the top of a section; and (3) drop entrainment mass rate per unit volume and drop de-entrainment mass rate per unit volume contributions to the momentum calculations for split channels. This problem has been evaluated for impact on existing analyses and its resolution represents a Non-Discretionary Change in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

Based on the results from a matrix of representative plant calculations, it is concluded that the effect of this error on the quantities directly impacted by the momentum equation calculations for split channels (velocities, flows, etc.) is negligible, leading to an estimated peak cladding temperature impact of 0°F.

ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES

CHANGES TO VESSEL SUPERHEATED STEAM PROPERTIES

Background

Several related changes were made to the WCOBRA/TRAC coding for the vessel super-heated water properties, including updating the HGAS subroutine coding to be consistent with WCAP-12945-P-A Equation 10-6, updating the approximation of the enthalpy in the TGAS subroutine to be consistent with the HGAS subroutine coding, and updating the temperature iteration method and convergence criteria in the TGAS subroutine. These changes represent a closely-related group of Non-Discretionary Changes in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

The updates to the calculations of the superheated steam properties had generally less than 1°F impact on the resulting steam temperature values, leading to an estimated peak cladding temperature impact of 0°F.

**ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES**

UPDATE TO METAL DENSITY REFERENCE TEMPERATURES

Background

It was identified that for one-dimensional components in which heat transfer to stainless steel 304 or 316 is modeled, the reference temperature for the metal density calculation was allowed to vary; as a result the total metal mass was not preserved. Correction of this problem represents a Non-Discretionary Change in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

This change primarily impacts the reactor coolant system loop piping modeled in the large break loss-of-coolant accident (LBLOCA) WCOBRA/TRAC models. It was judged that the effect of this change on the peak cladding temperature results was negligible, leading to an estimated peak cladding temperature impact of 0°F.

ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES

DECAY HEAT MODEL ERROR CORRECTIONS

Background

The decay heat model in the WCOBRA/TRAC code was updated to correct the erroneously coded value of the yield fraction directly from fission for Group 19 of Pu-239, and to include the term for uncertainty in the prompt energy per fission in the calculation of the decay heat power uncertainty. Correction of these errors represents a closely-related group of Non-Discretionary Changes in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

These changes have a negligible impact on the calculated decay heat power, leading to an estimated peak cladding temperature impact of 0°F.

ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES

CORRECTION TO THE PIPE EXIT PRESSURE DROP ERROR

Background

An error was discovered in WCOBRA/TRAC whereby the frictional pressure drop at the split break TEE connection to the BREAK component was incorrectly calculated using the TEE hydraulic diameter instead of the BREAK component length input. This error has been evaluated for impact on existing analyses and its resolution represents a Non-Discretionary Change in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

Based on the results from a matrix of representative plant calculations, it is concluded that the effect of this error on the pressure at the break and the break flow is negligible, leading to an estimated peak cladding temperature impact of 0°F.

**ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES**

GRID HEAT TRANSFER ENHANCEMENT CALCULATION

Background

An issue was identified which could affect the calculation of the heat transfer at gridded elevations for Best-Estimate (BE) Large-Break Loss-of-Coolant Accident (LBLOCA) Evaluation Models (EMs). For a specific input condition, the grid heat transfer enhancement factor is calculated based on an erroneous core geometry, which can cause an over-prediction of the heat transfer coefficient at gridded elevations. This issue has been evaluated to estimate the impact on existing LBLOCA analysis results. The resolution of this issue represents a Non-Discretionary Change in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

The effect described above was judged to have a negligible effect on existing LBLOCA analysis results, leading to an estimated Peak Cladding Temperature (PCT) impact of 0°F.

ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES

REVISED HEAT TRANSFER MULTIPLIER DISTRIBUTIONS

Background

Several changes and error corrections were made to WCOBRA/TRAC and the impacts of these changes on the heat transfer multiplier uncertainty distributions were investigated. During this investigation, errors were discovered in the development of the original multiplier distributions, including errors in the grid locations specified in the WCOBRA/TRAC models for the G2 Refill and G2 Reflood tests, and errors in processing test data used to develop the reflood heat transfer multiplier distribution. Therefore, the blowdown heatup, blowdown cooling, refill, and reflood heat transfer multiplier distributions were redeveloped. For the reflood heat transfer multiplier development, the evaluation time windows for each set of test experimental data and each test simulation were separately defined based on the time at which the test or simulation exhibited dispersed flow film boiling heat transfer conditions characteristic of the reflood time period. The revised heat transfer multiplier distributions have been evaluated for impact on existing analyses. Resolution of these issues represents a closely related group of Non-Discretionary Changes in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

A plant transient calculation representative of R. E. Ginna transient behavior was performed with the latest version of WCOBRA/TRAC. Using this transient, a matrix of HOTSPOT calculations was performed to estimate the effect of the heat transfer multiplier distribution changes. Using these results and considering the heat transfer multiplier uncertainty attributes from limiting cases for R. E. Ginna, an estimated PCT effect of +2°F has been established for 10 CFR 50.46 reporting purposes for R. E. Ginna.

ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES

ERROR IN BURST STRAIN APPLICATION

Background

An error in the application of the burst strain was discovered in HOTSPOT. The equation for the application of the burst strain is given as Equation 7-69 in WCAP-16009-P-A and in WCAP-12945-P-A. The outer radius of the cladding after burst occurs should be calculated based on the burst strain, and the inner radius of the cladding should be calculated based on the outer radius. In HOTSPOT, the burst strain is applied to the calculation of the cladding inner radius. The cladding outer radius is then calculated based on the inner radius. As such, the burst strain is incorrectly applied to the inner radius rather than the outer radius, which impacts the resulting cladding geometry at the burst elevation after burst occurs. Correction of the erroneous calculation results in thinner cladding at the burst node and more fuel relocating into the burst node, leading to an increase in the Peak Cladding Temperature (PCT) at the burst node. This issue has been evaluated to estimate the impact on existing Best-Estimate (BE) Large-Break Loss-of-Coolant Accident (LBLOCA) analysis results. The resolution of this issue represents a Non-Discretionary Change in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

The issue described above was evaluated by executing the most limiting plant-specific HOTSPOT runs with a HOTSPOT version that includes the correction of this error. This plant-specific sensitivity study resulted in an estimated PCT impact of 0°F for R. E. Ginna.

**ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES**

CHANGES TO GRID BLOCKAGE RATIO AND POROSITY

Background

A change in the methodology used to calculate grid blockage ratio and porosity for Westinghouse fuel resulted in a change to the grid inputs for large break loss-of-coolant accident (LBLOCA) analyses. Grid inputs affect heat transfer in the core during a LBLOCA. This change represents a Non-Discretionary Change in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1996 Westinghouse Best Estimate Large Break LOCA Evaluation Model

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

The updates to the methodology to calculate grid blockage ratio and porosity used as input in Westinghouse LBLOCA models resulted in a negligible change to heat transfer in the core for the fuel type used in R. E. Ginna. The estimated penalty associated with the changes is 0°F for 10 CFR 50.46 reporting purposes.

**ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES**

INITIAL FUEL PELLET AVERAGE TEMPERATURE UNCERTAINTY CALCULATION

Background

In the Automated Statistical Treatment of Uncertainty Method (ASTRUM) Best-Estimate (BE) Large-Break Loss-of-Coolant Accident (LBLOCA) Evaluation Model (EM), uncertainties are applied to the gap heat transfer coefficient and pellet thermal conductivity to capture the uncertainty in the initial fuel pellet average temperature. This approach was compared to the initial fuel pellet average temperature uncertainties predicted by the PAD code at beginning-of-life conditions and found to be conservative in Section 25-4-2-4 of WCAP-12945-P-A. However, the initial fuel pellet average temperature uncertainty range analyzed at higher burnups in the ASTRUM EM is much wider than the uncertainty range predicted by the PAD code, which may result in excessively low or high analyzed initial fuel pellet average temperatures. This issue has been evaluated to estimate the impact on existing ASTRUM LBLOCA analysis results. The resolution of this issue represents a Non-Discretionary Change in accordance with Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

2004 Westinghouse Realistic Large Break LOCA Evaluation Model Using ASTRUM

Estimated Effect

The issue described above was evaluated with plant-specific sensitivity studies resulting in an estimated Peak Cladding Temperature (PCT) impact of 0°F.

**ATTACHMENT 1
R.E. GINNA NUCLEAR POWER PLANT
WESTINGHOUSE LOCA EVALUATION MODEL CHANGES**

SBLOCTA CLADDING STRAIN REQUIREMENT FOR FUEL ROD BURST

Background

An error was discovered in the minimum local strain required for burst for ZIRLO^{®1} cladding in the SBLOCTA code. The coding does not enforce reaching the minimum percent local strain threshold prior to calculating fuel rod burst. However, a review of licensing basis analyses revealed no instances of this error impacting calculated results. Resolution of this issue represents a Non-Discretionary Change to the Evaluation Model as described in Section 4.1.2 of WCAP-13451.

Affected Evaluation Model(s)

1985 Westinghouse Small Break LOCA Evaluation Model with NOTRUMP

Estimated Effect

Based on a review of current licensing basis analyses, and the phenomena and physics of a small break LOCA transient, it is concluded that this error has a negligible effect on small break LOCA analysis results, leading to an estimated Peak Cladding Temperature (PCT) impact of 0°F.

¹ ZIRLO is a registered trademark of Westinghouse Electric Company LLC, its affiliates and/or its subsidiaries in the United States of America and may be registered in other countries throughout the world. All rights reserved. Unauthorized use is strictly prohibited. Other names may be trademarks of their respective owners.

ATTACHMENT 2

R.E. GINNA NUCLEAR POWER PLANT

LBLOCA AND SBLOCA

PEAK CLAD TEMPERATURE

ASSESSMENT SHEETS

ATTACHMENT 2
R.E. GINNA NUCLEAR POWER PLANT
LBOCA AND SBLOCA PEAK CLAD TEMPERATURE ASSESSMENT SHEETS

Westinghouse LOCA Peak Clad Temperature Summary for ASTRUM Best Estimate Large Break

Plant Name: R. E. Ginna
Utility Name: Constellation Generation Group
Revision Date: 2/27/2014

Analysis Information

EM:	ASTRUM	Analysis	3/18/2005	Limiting Break	Split
FQ:	2.6	FdH:	1.72		
Fuel:	422 Vantage +	SGTP (%):	10		
Notes:	Uprate to 1811 MWt (inclusive of calorimetric uncertainty) Effective beginning Cycle 33, Mixed Core OFA & 422 V+				

	Clad Temp (°F)	Ref.	Notes
LICENSING BASIS			
Analysis-Of-Record PCT	1870	1	(a)
PCT ASSESSMENTS (Delta PCT)			
A. PRIOR ECCS MODEL ASSESSMENTS			
1 . HOTSPOT Fuel Relocation Error	37	2	
2 . Evaluation of Pellet Thermal Conductivity Degradation and Peaking Factor Burndown	230	3	(b)
B. PLANNED PLANT MODIFICATION EVALUATIONS			
1 . Evaluation of Design Input Changes	-96	3	(b, c)
2 . Evaluation of Elevated Initial Containment and Accumulator	75	4,6	(d)
C. 2013 ECCS MODEL ASSESSMENTS			
1 . Revised Heat Transfer Multiplier Distributions	2	5	
D. OTHER*			
1 . None	0		

LICENSING BASIS PCT + PCT ASSESSMENTS **PCT = 2118**

* It is recommended that the licensee determine if these PCT allocations should be considered with respect to 10 CFR 50.46 reporting requirements.

References

1. RGE-05-32, "Transmittal of Input to Boron Concentration Increase and LOCA Methodology Change Tech Spec Amendment Submittal," April 2005.
2. LTR-LIS-07-388, "10 CFR 50.46 Reporting Text for HOTSPOT Fuel Relocation Error and Revised PCT Rackup Sheets for R. E. Ginna," June 2007.
3. NF-RG-12-45, "Information Regarding the R.E. Ginna Evaluation of Fuel Pellet Thermal Conductivity Degradation and Peaking Factor Burndown Including Analysis Input Changes," July 2012.
4. LTR-LIS-13-31, "LBLOCA PCT Rackup Sheet Update for the R. E. Ginna Evaluation of Elevated Initial Containment and Accumulator Temperature," January 2013.
5. LTR-LIS-13-367, "R. E. Ginna 10 CFR 50.46 Report for Revised Heat Transfer Multiplier Distributions," July 2013.
6. Letter from J.E. Pacher (REG) to NRC Document Control Desk, "License Amendment Request, Revise Section 3.6.5 of the Technical Specifications, "Containment Air Temperature,"" dated February 28, 2013 (ML13067A328)

ATTACHMENT 2
R.E. GINNA NUCLEAR POWER PLANT
LBOCA AND SBLOCA PEAK CLAD TEMPERATURE ASSESSMENT SHEETS

Notes:

- (a) Transition cycles containing OFA fuel are bounded by the analysis for 422 V+ fuel.
- (b) These assessments are coupled via an evaluation of burnup effects which include thermal conductivity degradation, peaking factor burndown, and design input changes. These assessments explicitly include the HOTSPOT Fuel Relocation Error correction.
- (c) Design input change was a reduction in steady-state FQ from 2.1 to 2.0.
- (d) Note that the LBLOCA analysis-of-record is not impacted by the increase in initial containment temperature. Therefore, the PCT assessment reflects the increase in initial accumulator temperature only.

**ATTACHMENT 2
R.E. GINNA NUCLEAR POWER PLANT
LBOCA AND SBLOCA PEAK CLAD TEMPERATURE ASSESSMENT SHEETS**

Westinghouse LOCA Peak Clad Temperature Summary for Appendix K Small Break

Plant Name: R. E. Ginna
Utility Name: Constellation Generation Group
Revision Date: 2/27/2014

Analysis Information

EM: NOTRUMP **Analysis** 4/21/2005 **Limiting Break** 2 inch, Hi Tavg
FQ: 2.6 **FdH:** 1.72
Fuel: 422 Vantage + **SGTP (%):** 10
Notes: Uprate to 1811 MWt (inclusive of calorimetric uncertainty), Effective beginning Cycle 33,
Mixed Core OFA & 422 V+

	Clad Temp (°F)	Ref.	Notes
LICENSING BASIS			
Analysis-Of-Record PCT	1167	1	(a)
PCT ASSESSMENTS (Delta PCT)			
A. PRIOR ECCS MODEL ASSESSMENTS			
1 . None	0		
B. PLANNED PLANT MODIFICATION EVALUATIONS			
1 . None	0		
C. 2013 ECCS MODEL ASSESSMENTS			
1 . None	0		
D. OTHER*			
1 . None	0		
LICENSING BASIS PCT + PCT ASSESSMENTS	PCT = 1167		

* It is recommended that the licensee determine if these PCT allocations should be considered with respect to 10 CFR 50.46 reporting requirements.

Reference

- 1 . RGE-05-32, "Transmittal of Input to Boron Concentration Increase and LOCA Methodology Change Tech Spec Amendment Submittal," April 2005.

Notes:

- (a) Transition cycles containing OFA fuel are bounded by the analysis for 422 V+ fuel.