
1.6 Material Referenced

Table 1.6-1—Reports Referenced, contains a list of AREVA reports submitted to the NRC that are incorporated by reference in the U.S. EPR FSAR. A COL applicant that references the US EPR design certification will include any site-specific topical reports that are incorporated by reference as part of the COL application in Table 1.6-1.

**Table 1.6-1—Reports Referenced
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Report No. (See Notes 1, 2, and 3)	Title	Date Submitted to NRC	FSAR Section Number(s)
ANF-89-060P-A ANF-89-060NP-A Supplement 1	Generic Mechanical Design Report High Thermal Performance Spacer and Intermediate Flow Mixer	3/28/91	4.2
ANP-10263P-A ANP-10263NP-A	Codes and Methods Applicability Report for the U.S. EPR	11/06/07	4, 5.2, 15, and 16
ANP-10266 Revision 4	AREVA NP Inc. Quality Assurance Plan (QAP) for Design Certification of the U.S. EPR Topical Report	6/18/07	7.1, 9.5, 15.0, 17.0, 17.4, 17.5, 18.1, 18.7, and 18.11
ANP-10268P-A ANP-10268NP-A	U.S. EPR Severe Accident Evaluation Topical Report	2/26/08	6.2.5, 19.1, and 19.2
ANP-10269P-A ANP-10269NP-A	The ACH-2 CHF Correlation for the U.S. EPR Topical Report	3/10/08	4.4, 15, and 16
ANP-10272 -A Revision 3	Software Program Manual TELEPERM XSTM Safety Systems Topical Report	7/11	7.1 and 7.6
ANP-10275P-A ANP-10275NP-A	U.S. EPR Instrument Setpoint Methodology Topical Report	2/26/08	1.9, 7.1, 7.2, 7.3 and 16
ANP-10278P-A ANP-10278NP-A Revision 1	U.S. EPR Realistic Large Break Loss of Coolant Accident Topical Report	3/08/12	15 and 16
ANP-10282P ANP-10282NP	POWERTRAX/E Online Core Monitoring Software for the U.S. EPR Technical Report	11/27/07	4.4
ANP-10283P-A Revision 2 ANP-10283NP-A Revision 2	U.S. EPR Pressure-Temperature Limits Methodology for RCS Heat-Up and Cool-Down Topical Report	8/12	5.3 and 16
ANP-10285P Rev 1 ANP-10285NP Rev 1	U.S. EPR Fuel Assembly Mechanical Design Topical Report	5/31/2013	4.1,4.2, 4.3, and 16
ANP-10286P-A ANP-10286NP-A	U.S. EPR Rod Ejection Accident Methodology Topical Report	3/08/12	4.3, 15 and 16
ANP-10287P ANP-10287NP Revision 1	Incore Trip Setpoint and Transient Methodology for U.S. EPR Topical Report	4/12	4.3, 4.4, 7.1, 7.2, 15.0, 15.1, 15.2,15.3,15.4,15.6, 16
ANP-10288P ANP-10288NP Revision 2	U.S. EPR Post-LOCA Boron Precipitation and Boron Dilution Technical Report	6/13	1.9 and 15.6

**Table 1.6-1—Reports Referenced
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Report No. (See Notes 1, 2, and 3)	Title	Date Submitted to NRC	FSAR Section Number(s)
ANP-10290 Revision 1	AREVA NP Environmental Report Standard Design Certification	9/11/09	19.1 and 19.2
ANP-10291P ANP-10291NP	Small Break LOCA and Non-LOCA Sensitivity Studies and Methodology Technical Report	5/09	15 and 16
ANP-10292 Revision 1	U.S. EPR Conformance with Standard Review Plan (NUREG-0800) Technical Report	5/09	1.9
ANP-10293P, Revision 4	U.S. EPR Design Features to Address GSI-191 Technical Report	11/11	6.1, 6.3, and 15.6
ANP-10294 Revision 1	U.S. EPR Reactor Coolant Pump Motor Flywheel Structural Analysis Technical Report	3/09	5.4
ANP-10295P ANP-10295NP Revision 4	U.S. EPR Security Design Features Technical Report	4/13	13.6 and 19.2
ANP-10296 Revision 2	U.S. EPR Design Features that Enhance Security	6/13	13.6 and 19.2
ANP-10299P Revision 2	Applicability of AREVA NP Containment Response Evaluation Methodology to the U.S. EPR for Large Break LOCA Analysis, including Supplement 1, August 2011.	12/09	6.2
<i>[ANP-10304 Revision 6</i>	<i>U.S. EPR Diversity and Defense in Depth Assessment Technical Report]*</i>	5/13	1.9, 7.1, 7.2, 7.3, 7.8, 18.7, 19.1
ANP-10306P, Revision 1	Comprehensive Vibration Assessment Program for U.S. EPR Reactor Internals Technical Report	1/13	3.9.2 and 3.9.5
<i>[ANP-10309P ANP-10309NP Revision 5</i>	<i>U.S. EPR Protection System Technical Report]*</i>	5/13	4.6, 7.1, 7.2, 7.3 and 16
<i>[ANP-10310P Revision 2</i>	<i>Methodology for 100% Combinatorial Testing of the U.S. EPR Priority Module Technical Report]*</i>	5/13	7.1 and 7.8
ANP-10314	The Operating Strategies for Severe Accidents Methodology for the U.S. EPR Technical Report	7/10	19.2

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Report No. (See Notes 1, 2, and 3)	Title	Date Submitted to NRC	FSAR Section Number(s)
[ANP-10315P Revision 2	<i>U.S. EPR Protection System Surveillance Testing and Teleperm XS Self-Monitoring Technical Report]*</i>	5/13	7.1, 7.3, 7.8 and 16
ANP-10317 Revision 3	Design Requirements for the U.S. EPR Aircraft Hazard Protection Structures	7/13	19.2.7.4
ANP-10318P Revision 2	Pipe Rupture External Loading Effects on U.S. EPR Essential Structures, Systems, and Components Technical Report	12/13	3.6.2
ANP-10322P	Qualification and Testing of the U.S. EPR Passive Autocatalytic Recombiner	6/12	6.2.5 and 6.2.6
[ANP-10324P	<i>U.S. EPR Implementation Plan for the Integration of Human Reliability Analysis (HRA) with the Human Factors Engineering (HFE) Program]*</i>	1/13	18.6
ANP-10325P ANP-10325NP	U.S. EPR Fuel Assembly - Faulted Condition Analysis	5/23/2013	4, 15.6
[ANP-10327P	<i>U.S. EPR HFE Program Management Plan]*</i>	4/13	18
[ANP-10328P	<i>U.S. EPR Human System Interface Design Implementation Plan]*</i>	4/13	18
ANP-10329	U.S. EPR Mitigation Strategies for Extended Loss of AC Power Event Technical Report	5/13	6.3, 7.1, 8.3, 19.2
BAW-10132-A	Analytical Methods Description – Reactor Coolant System Hydrodynamic Loadings During a Loss-of-Coolant Accident	7/20/79	App. 3C
BAW-10133P-A BAW-10133-A Revision 1, Addendum 1 and 2	Mark-C Fuel Assembly LOCA-Seismic Analysis	12/00	3.7.1
BAW-10147P-A BAW-10147-A Revision 1	Fuel Rod Bowing in Babcock & Wilcox Fuel Designs	6/28/83	4.2, 4.4
BAW-10156-A Revision 1	LYNXT, Core Transient Thermal-Hydraulic Program	8/18/93	4

Table 1.6-1—Reports Referenced
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Report No. (See Notes 1, 2, and 3)	Title	Date Submitted to NRC	FSAR Section Number(s)
BAW-10163P-A BAW-10163-A	Core Operating Limit Methodology for Westinghouse Designed PWRs	6/2/89	4.3
BAW-10164P-A BAW-10164NP-A Revision 6	RELAP5/MOD2-B&W - An Advanced Computer Program for Light Water Reactor LOCA and Non-LOCA Transient Analysis	11/20/07	3.9.1, 6.2, 6.3, and 8.4
BAW-10168P-A BAW-10168-A Revision 3	BWNT Loss-of-Coolant Accident Model for Recirculating Steam Generator Plants	1/31/97	6.2
BAW-10169P-A BAW-10169-A	B&W Safety Analysis Methodology for Recirculating Steam Generator Plants	10/89	6.2
BAW-10172P-A BAW-10172NP-A	Mark-BW Mechanical Design Report	12/19/89 (Note 4)	4.2
BAW-10183P-A BAW-10183-A	Fuel Rod Gas Pressure Criterion (FRGPC)	7/24/95	4.2
BAW-10186P-A BAW-10186NP-A Revision 2	Extended Burnup Evaluation	1/04	4.2 and 4.4
BAW-10199P-A BAW-10199-A	The BWU Critical Heat Flux Correlations	8/19/96	4.4
BAW-10221P-A BAW-10221-A	NEMO-K A Kinetics Solution in NEMO	9/17/88	4.3 and 16
BAW-10227P-A BAW-10227NP-A Revision 1	Evaluation of Advanced Cladding and Structural Material (M5) in PWR Reactor Fuel	6/18/03 (Note 4)	4.1, 4.2, and 15
BAW-10231P-A BAW-10231NP-A Revision 1	COPERNIC Fuel Rod Design Computer Code	9/30/04	4 and 16
BAW-10239P-A BAW-10239NP-A	Advanced Mark-BW Fuel Assembly Mechanical Design Topical Report	10/5/04	4.2
BAW-10240P-A BAW-10240NP-A	Incorporation of M5™ Properties in Framatome ANP Approved Methods	8/17/04	4.2
BAW-10243P-A BAW-10243NP-A	Statistical Fuel Assembly Hold Down Methodology	12/21/05	4.4
BAW-10252P-A BAW-10252NP-A	Analysis of Containment Response to Postulated Pipe Ruptures using GOTHIC	12/21/05	6.3

**Table 1.6-1—Reports Referenced
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Report No. (See Notes 1, 2, and 3)	Title	Date Submitted to NRC	FSAR Section Number(s)
BAW-1810	Nuclear Model Development and Critical Experiment Benchmark	4/84	4.3
BAW-2241P-A BAW-2241NP-A Revision 2	Fluence and Uncertainty Methodologies	11/20/07	1.8, 4.1, and 5.3
EMF-2110(NP)(A) Revision 1	TELEPERM XS: A Digital Reactor Protection System	7/12/00	7.1, 7.2, 7.3, and 19.1
EMF-2310(P)(A) Revision 1	SRP Chapter 15 Non-LOCA Methodology for Pressurized Water Reactors	5/04	16
EMF-96-029(P)(A) EMF-96-029(NP)(A)	Reactor Analysis Systems for PWRs, Volume 1 -Methodology Description, Volume 2 -Benchmarking Results	1/13/97	4.3 and 16
TN-Rack 0101(P)	U.S. EPR New and Spent Fuel Storage Rack Technical Report	12/09	9.1.1, and 9.1.2

Notes:

1. P – Denotes document is proprietary.
2. NP – Denotes the document is non-proprietary. Any reports that do not have a “P” or “NP” after the numerical designation are also non-proprietary.
3. “-A” denotes the published version of the report following issuance of the NRC Safety Evaluation Report approving the topical report.
4. This is the date that the NRC approved this topical report. The letter transmitting the approved version to NRC is not publicly available.