

#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

February 27, 2014

Mr. Edward D. Halpin Senior Vice President and Chief Nuclear Officer Pacific Gas and Electric Company Diablo Canyon Power Plant P.O. Box 56, Mail Code 104/6 Avila Beach, CA 93424

### SUBJECT: DIABLO CANYON POWER PLANT, UNIT NOS. 1 AND 2 - ISSUANCE OF AMENDMENTS RE: EDITORIAL REVISIONS TO TECHNICAL SPECIFICATIONS 3.7.10 AND 5.6.5 (TAC NOS. MF1959 AND MF1960)

Dear Mr. Halpin:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 217 to Facility Operating License No. DPR-80 and Amendment No. 219 to Facility Operating License No. DPR-82 for the Diablo Canyon Power Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated June 6, 2013.

The amendments revise Technical Specification (TS) 3.7.10, "Control Room Ventilation System (CRVS)," and TS 5.6.5, "Core Operating Limits Report (COLR)," to incorporate editorial changes. Specifically, the amendments delete footnote (1) from the TS 3.7.10 Condition A Completion Time, and revise inconsistent wording in TS 5.6.5a.4, TS 5.6.5a.5, and TS 5.6.5a.9.

E. Halpin

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

James Ki

James Kim, Project Manager Plant Licensing IV-2 and Decommissioning Transition Branch Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-275 and 50-323

Enclosures:

- 1. Amendment No. 217 to DPR-80
- 2. Amendment No. 219 to DPR-82
- 3. Safety Evaluation

cc w/enclosures: Distribution via Listserv



#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

# PACIFIC GAS AND ELECTRIC COMPANY

# DOCKET NO. 50-275

## DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 1

### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 217 License No. DPR-80

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Pacific Gas and Electric Company (the licensee), dated June 6, 2013, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. DPR-80 is hereby amended to read as follows:
  - (2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 217, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

CEDAD for

Michael T. Markley, Chief Plant Licensing IV-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Attachment: Changes to the Facility Operating License No. DPR-80 and Technical Specifications

Date of Issuance: February 27, 2014



#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

## PACIFIC GAS AND ELECTRIC COMPANY

### DOCKET NO. 50-323

### DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 2

### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 219 License No. DPR-82

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Pacific Gas and Electric Company (the licensee), dated June 6, 2013, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. DPR-82 is hereby amended to read as follows:
  - (2) <u>Technical Specifications (SSER 32, Section 8)\* and Environmental</u> <u>Protection Plan</u>

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 219, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

CFJyonfor

Michael T. Markley, Chief Plant Licensing IV-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Attachment: Changes to the Facility Operating License No. DPR-82 and Technical Specifications

Date of Issuance: February 27, 2014

### ATTACHMENT TO LICENSE AMENDMENT NO. 217

### TO FACILITY OPERATING LICENSE NO. DPR-80

#### AND AMENDMENT NO. 219 TO FACILITY OPERATING LICENSE NO. DPR-82

### DOCKET NOS. 50-275 AND 50-323

Replace the following pages of the Facility Operating License Nos. DPR-80 and DPR-82, and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Operating License Nos. DPR-80 and DPR-82

REMOVE INSERT

-3-

**Technical Specifications** 

-3-

REMOVE	INSERT		
3.7-18	3.7-18		
5.0-19	5.0-19		

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This License shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
  - (1) Maximum Power Level

The Pacific Gas and Electric Company is authorized to operate the facility at reactor core power levels not in excess of 3411 megawatts thermal (100% rated power) in accordance with the conditions specified herein.

(2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 217, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

#### (3) Initial Test Program

The Pacific Gas and Electric Company shall conduct the post-fuel-loading initial test program (set forth in Section 14 of Pacific Gas and Electric Company's Final Safety Analysis Report, as amended), without making any major modifications of this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

a. Elimination of any test identified in Section 14 of PG&E's Final Safety Analysis Report as amended as being essential;

Amendment No. 217

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This License shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
  - (1) <u>Maximum Power Level</u>

The Pacific Gas and Electric Company is authorized to operate the facility at reactor core power levels not in excess of 3411 megawatts thermal (100% rated power) in accordance with the conditions specified herein.

(2) <u>Technical Specifications (SSER 32, Section 8)\* and Environmental</u> <u>Protection Plan</u>

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 219, are hereby incorporated in the license. Pacific Gas & Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan, except where otherwise stated in specific license conditions.

(3) Initial Test Program (SSER 31, Section 4.4.1)

Any changes to the Initial Test Program described in Section 14 of the FSAR made in accordance with the provisions of 10 CFR 50.59 shall be reported in accordance with 50.59(b) within one month of such change.

Amendment No. 219

<sup>\*</sup>The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report and/or its supplements wherein the license condition is discussed.

#### 3.7 PLANT SYSTEMS

3.7.10 Control Room Ventilation System (CRVS)

LCO 3.7.10 Two CRVS trains shall be OPERABLE.

------NOTE------The control room envelope (CRE) boundary may be opened intermittently under administrative controls. 

APPLICABILITY: MODES 1, 2, 3, 4, 5, and 6. During movement of recently irradiated fuel assemblies.

ACTIONS

-----NOTE------NOTE------ACTIONS apply simultaneously to both units. 

CONDITION		REQUIRED ACTION		COMPLETION TIME	
Α.	One CRVS train inoperable for reasons other than Condition B.	A.1	Restore CRVS train to OPERABLE status.	7 days	
B.	One or more CRVS trains inoperable due to inoperable CRE boundary in MODE 1, 2, 3, or 4.	B.1 <u>AND</u>	Initiate action to implement mitigating actions.	Immediately	
		В.2	Verify mitigating actions ensure CRE occupant exposures to radiological hazards will not exceed limits, and CRE occupants are protected from smoke and chemical hazards.	24 hours	
		AND			
		B.3	Restore CRE boundary to OPERABLE status.	90 days	

(continued)

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DIABLO CANYON - UNITS 1 & 2

#### 5.6 Reporting Requirements (continued)

#### 5.6.3 Radioactive Effluent Release Report

A single submittal may be made for a multiple unit station. The submittal shall combine sections common to all units at the station; however, for units with separate radwaste systems, the submittal shall specify the releases of radioactive material from each unit.

The Radioactive Effluent Release Report covering the operation of the unit during the previous year shall be submitted prior to May 1 of each year in accordance with 10 CFR 50.36a. The report shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit. The material provided shall be consistent with the objectives outlined in the ODCM and Process Control Program and in conformance with 10 CFR 50.36a and 10 CFR 50, Appendix I, Section IV.B.1.

#### 5.6.4 Not Used

### 5.6.5 CORE OPERATING LIMITS REPORT (COLR)

- a. Core operating limits shall be established prior to each reload cycle, or prior to any remaining portion of a reload cycle, and shall be documented in the COLR for the following:
  - 1. Shutdown Bank Insertion Limits for Specification 3.1.5,
  - 2. Control Bank Insertion Limits for Specification 3.1.6,
  - 3. Axial Flux Difference for Specification 3.2.3,
  - 4. Heat Flux Hot Channel Factor ( $F_Q(z)$ ) for Specification 3.2.1,
  - 5. Nuclear Enthalpy Rise Hot Channel Factor  $(F_{AH}^{N})$  for Specification 3.2.2,
  - 6. SHUTDOWN MARGIN values in Specifications 3.1.1, 3.1.4, 3.1.5, 3.1.6, and 3.1.8,
  - 7. Moderator Temperature Coefficient limits in Specification 3.1.3,
  - 8. Refueling Boron Concentration limits in Specification 3.9.1, and
  - 9. RCS Pressure, Temperature, and Flow DNB Limits in Specification 3.4.1.

(continued)



# SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

## RELATED TO AMENDMENT NO. 217 TO FACILITY OPERATING LICENSE NO. DPR-80

## AND AMENDMENT NO. 219 TO FACILITY OPERATING LICENSE NO. DPR-82

# PACIFIC GAS AND ELECTRIC COMPANY

## DIABLO CANYON POWER PLANT, UNIT NOS. 1 AND 2

## DOCKET NOS. 50-275 AND 50-323

## 1.0 INTRODUCTION

By application dated June 6, 2013 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML13158A137), Pacific Gas and Electric Company (the licensee) requested changes to the Technical Specifications (Appendix A to Facility Operating License Nos. DPR-80 and DPR-82) for the Diablo Canyon Power Plant, Unit Nos. 1 and 2.

Specifically, the proposed changes would revise Technical Specification (TS) 3.7.10, "Control Room Ventilation System (CRVS)," and TS 5.6.5, "Core Operating Limits Report (COLR)," to incorporate editorial changes.

### 2.0 REGULATORY EVALUATION

In Section 50.36, "Technical specifications," of Title 10 of the *Code of Federal Regulations* (10 CFR), the Commission established its regulatory requirements related to the content of TS. Pursuant to 10 CFR 50.36, TS are required to include items in the following five specific categories related to station operation: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. The rule does not specify the particular requirements to be included in a plant's TS.

### 3.0 TECHNICAL EVALUATION

#### 3.1 Proposed TS Changes

#### 3.1.1 <u>TS 3.7.10</u>

Current footnote (1) to the TS 3.7.10 Condition A Completion Time states:

(1) The Completion Time that one CRVS train can be inoperable as specified by Required Action A.1 may be extended beyond the 7 day completion time up to 13 days to support repair and restoration of the CRVS actuation instrumentation which required TS 3.7.10 Condition A entry per TS 3.3.7 Required Action B.1.2. Upon completion of the repair and restoration, this footnote is no longer applicable and will expire at 20:38 PST on December 10, 2012.

The amendment would delete the above footnote.

### 3.1.2 <u>TS 5.6.5a.4</u>

Current TS 5.6.5a.4 states:

Heat Flux Hot Channel Factor, K(Z) and  $W(Z) - F_Q(z)(F_Q^{RTP}$  Specification 3.2.1),

Revised TS 5.6.5a.4 would state:

Heat Flux Hot Channel Factor ( $F_Q(Z)$ ) for Specification 3.2.1,

3.1.3 <u>TS 5.6.5a.5</u>

Current TS 5.6.5a.5 states:

RCS Flow Rate and Nuclear Enthalpy Rise Hot Channel Factor –  $F^{N}_{\Delta H}(F^{RTP}_{\Delta H})$  and  $PF_{\Delta H}$  for Specification 3.2.2),

Revised TS 5.6.5a.5 would state:

Nuclear Enthalpy Rise Hot Channel Factor  $(F^{N}_{\Delta H})$  for Specification 3.2.2,

3.1.4 <u>TS 5.6.5a.9</u>

Current TS 5.6.5a.9 states:

RCS pressure and temperature DNB limits in Specification 3.4.1,

Revised TS 5.6.5a.9 would state:

RCS Pressure Temperature, and Flow DNB Limits in Specification 3.4.1.

The NRC staff has reviewed the licensee's regulatory and technical analyses in support of its proposed license amendment, which are described in Sections 2 and 3 of the licensee's submittal. The NRC staff's evaluation is provided below:

## 3.2.1 Changes to TS 3.7.10

The proposed change to TS 3.7.10 deletes a footnote that was included in a license amendment under emergency circumstances dated December 4, 2012 (ADAMS Accession No. ML12338A020). The footnote was only valid until December 10, 2012. Therefore, deletion of footnote (1) to the TS 3.7.10 Condition A Completion Time is editorial in nature and, therefore, the change is acceptable.

## 3.2.2 Changes to TS 5.6.5

TS 5.6.5a specifies the limits that shall be documented in the Core Operating Limits Report (COLR). It also references specific TS section(s) for each required limit. The proposed changes correct inconsistent wording between the referenced TSs in 5.6.5a.4, 5.6.5a.5, and 5.6.5a.9, and the TS section titles. The changes are considered to be of an editorial and clarifying nature and are not material, such that the current intent of the requirement is unchanged. The NRC staff concludes that these changes are acceptable.

# 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding published in the *Federal Register* on August 6, 2013 (78 FR 47791). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10)(v). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 6.0 <u>CONCLUSION</u>

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Jennivine K. Rankin

Date: February 27, 2014

E. Halpin

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

/**RA**/

James Kim, Project Manager Plant Licensing IV-2 and Decommissioning Transition Branch Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-275 and 50-323

Enclosures:

- 1. Amendment No. 217 to DPR-80
- 2. Amendment No. 219 to DPR-82
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DATE	2/18/14	2/14/14	1/30/14	1/30/14	
OFFICE	NRR/DSS/SRXB/BC	OGC/NLO with comments	NRR/DORL/LPL4-1/BC	NRR/DORL/LPL4-2/PM	
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