From: Guzman, Richard
Sent: Friday, February 07, 2014 6:45 PM
To: 'Gray, Corey A'
Subject: Oconee Unit 3: Request for Relief (RR) 13-ON-001, Sections 2 through 8 - Request for Additional Information

Categories: Followup

Corey,

As we discussed, the NRC staff is reviewing the subject Request for Relief dated September 26, 2013 (ADAMS Accession No. ML13273A037) and has determined that additional information is needed to support the completion of their review. Shown below is the staff's request for additional information (RAI). To support the timely completion of our review, we request you provide a response within 30 days of receipt of this message. Please let me know if you'd like to have a call w/the NRR technical staff to cover any clarifications necessary to address the questions.

Thanks,

Rich Guzman
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REQUEST FOR ADDITIONAL INFORMATION
ON THE FOURTH TEN YEAR 10-YEAR INSERVICE INSPECTION INTERVAL
REQUESTS FOR RELIEF
DUKE ENERGY CAROLINAS, LLC.
OCONEE NUCLEAR STATION, UNIT 3
DOCKET NUMBER: 50-287
TAC NO. MF2836

By letter dated September 26, 2013, Agencywide Documents Access and Management System (ADAMS) Accession No. ML13273A037, Duke Energy Carolinas, LLC (Duke, the licensee), submitted Request for Relief (RR) 13-ON-001, Sections 2 through 8, from the requirements of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, *Rules for Inservice Inspection of Nuclear Power Plant Components* for Oconee Nuclear Station, Unit 3 (ONS3). The RR applies to the fourth 10-year inservice inspection (ISI) interval in which the licensee adopted the 1998 Edition through the 2000 Addenda of ASME Code Section XI as the code of record.

RAI-1

Request for Relief 13-ON-001, Sections 6 & 7, ASME Code, Section XI, Examination Category B-J, Unit 3 Piping Welds, Low Pressure Injection System, Pipe to Valve (3CF-11 & 3CF-14), Welds 3-53A-15-26 & 3CF-126-4, Summary Numbers 03.B9.11.0106 & O3.B9.11.0129, ASME Code Class 1.

a. State the active degradation mechanisms for welds 3-53A-15-26 and 3CF-126-4, if any.

b. Are the welds part of a risk informed inservice inspection program?

c. Is there any operational experience showing inservice cracking of similar welds exposed to similar environmental conditions?

RAI-2

Request for Relief 13-ON-001, Section 8 ASME Code, Section XI, Examination Category C-A, Item C1.30 Pressure Retaining Welds in Vessels for ON, Unit 3 Steam Generator A, Feedwater System, Tubesheet-to-Shell Weld # 3-SGA-W69 Summary Number O3.C1.30.0002, ASME Code Class 2.

The NRC staff notes that in RR 13-ON-001, Section 8, conflicting examination methodologies are stated in the following subsections:

- In Paragraph 8.4, "Impracticability of Compliance," it is stated that, "scanning requirements are described in ASME Code, Section V, Article 4 T-441.1.2(a), T-441.1.3, T-441.1.4, T-441.1.5, and T-441.1.6."

- In Paragraph 8.5, "Proposed Alternative and Basis for Use," it is stated that "this weld was examined using procedures, equipment and personnel qualified in accordance with ASME Code, Section XI, Appendix VIII."

- It is also stated in Attachment "A", *UT Calibration Examination Sheet* (Page 81 of 96), that Procedure NDE-640 was used.

Please clarify what examination methodologies were used in examining Tubesheet-to-Shell Weld # 3-SGA-W69.

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