



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

March 4, 2014

LICENSEE: Exelon Generation Company, LLC


FACILITY: Byron Station, Units 1 and 2, and  
Braidwood Station, Units 1 and 2

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON JANUARY 28,  
2014, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND  
EXELON GENERATION COMPANY, LLC, CONCERNING RAI SET 10, FOR  
THE BYRON STATION AND BRAIDWOOD STATION, LICENSE RENEWAL  
APPLICATION (TAC NOS. MF1879, MF1880, MF1881, AND MF1882)

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Exelon Generation Company, LLC (Exelon or the applicant), held a telephone conference call on January 28, 2014, to discuss six Request for Additional Information (RAI) from Set 10 RAIs for the Byron Station (Byron), Units 1 and 2 and the Braidwood Station (Braidwood), Units 1 and 2, license renewal application (LRA). The telephone conference call was useful in clarifying the RAIs.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a summary of the discussion.

The applicant had an opportunity to comment on this summary.

  
John Daily, Senior Project Manager  
Projects Branch 1  
Division of License Renewal  
Office of Nuclear Reactor Regulation

Docket Nos. 50-454, 50-455, 50-456, and 50-457

Enclosures:

1. List of Participants
2. Discussion summary

cc w/encls: Listserv

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TELEPHONE CONFERENCE CALL  
BYRON AND BRAIDWOOD STATIONS  
LICENSE RENEWAL APPLICATION

LIST OF PARTICIPANTS  
January 28, 2014

<u>PARTICIPANTS</u>	<u>AFFILIATIONS</u>
John Daily	Nuclear Regulatory Commission (NRC)
Matt Homiack	NRC
Seung Min	NRC
John Hufnagel	Exelon Generation Company, LLC (Exelon)
Al Fulvio	Exelon
Don Warfel	Exelon
John Kozakowski	Exelon
Paul Weyhmuller	Exelon
Phil O'Donnell	Exelon
Tom Quintenz	Exelon
Paul Cervenka	Exelon
Pete Tamburro	Exelon
Don Brindle	Exelon
Ralph Wolen	Exelon

SUMMARY OF TELEPHONE CONFERENCE CALL  
FOR RAI SET 10, FOR THE BYRON AND BRAIDWOOD  
LICENSE RENEWAL APPLICATION  
January 28, 2014

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Exelon Generation Company, LLC (Exelon or the applicant), held a telephone conference call on January 28, 2014, to discuss six RAIs from Set 10 for the Byron and Braidwood license renewal application (LRA). The telephone conference call was useful in clarifying the RAIs.

**Discussion**

The staff and the applicant discussed the following from Set 10. Noteworthy discussion points (beyond general clarifications and minor editorial comments) are included below as applicable.

- RAI 3.1.1.80-1. The staff clarified that the response should address the outcome of a review of operating experience associated with the reactor coolant system looking for vibration-induced failures of connected ASME Class 2 piping/tubing. No other issues or additional concerns were identified.
- RAI 3.1.1.81-1. The applicant and staff discussed the physical location of the thermal barrier heat exchanger and that it is an internal, integral part of the reactor coolant pump that is not inspectable without significant disassembly. Also, it was agreed that a reasonable window for review of operating experience associated with component cooling sampling results would be five years.
- RAI 3.2.1.20-1. The applicant clarified that the feedwater flow venturis for both Byron and Braidwood, while installed and used in the past, are not currently used as normal (preferred) feedwater flow meters because currently ultrasonic flow meters are normal (preferred) feedwater flow meters.
- RAI 4.7.4-1. In discussing the issue of residual heat removal (RHR) nozzle flaws and associated time-limited aging analysis (TLAA), the applicant stated that several documents already on the docket from Exelon provide additional details of which the staff may or may not be aware. The applicant agreed to provide links to the documentation so the staff could review it for relevance to the RAI.
- RAI 4.7.5-1. In discussing reactor coolant pump motor flywheel fatigue crack growth analyses, the applicant and staff discussed potential methods of summarization for the response. It was agreed that the applicant would not need to quantify any crack growth if the most-recent inspections found no evidence of cracking in all 18 of the flywheels.
- RAI 4.7.5-2. The applicant and staff discussed alternate means of uniquely identifying each flywheel; it was agreed that use of the flywheel's component ID, or "serial number," would be appropriate since a given flywheel could be relocated to another reactor coolant loop in various scenarios, yet each one is in need of having appropriate identification, inspection frequency (10-year or 20-year cycle), and basis for inspection frequency documented in the updated final safety analysis report (UFSAR). The staff indicated that reference to a properly explicit list or description may be sufficient if adequate details and justification are provided on the docket.

For the RAIs discussed on the call, the applicant indicated it understands the RAIs and associated concerns and will factor them into the preparations of its responses. The staff agreed as indicated above to review any additional, identified docketed material for applicability and update or revise RAIs as appropriate.

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