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JAN 28 2014

10CFR50.73

LR-N14-008

United States Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-001

Hope Creek Generating Station Unit 1
Renewed Facility Operating License No. NPF-57
Docket No. 50-354

Subject: Licensee Event Report 2013-008-00

In accordance with the requirements of 10 CFR 50.73(a)(2)(iv)(A), PSEG Nuclear LLC is submitting the enclosed Licensee Event Report (LER) Number 2013-008-00, "Automatic Actuation of the Reactor Protection System due to a Main Turbine Trip."

If you have any questions or require additional information, please contact Mr. Philip Duca at (856) 339-1640.

There are no regulatory commitments contained in this letter.

Sincerely,

A handwritten signature in black ink, appearing to read "Eric S. Carr", with a long horizontal flourish extending to the right.

Eric S. Carr
Plant Manager
Hope Creek Generating Station

Attachment: Licensee Event Report 2013-008-00

cc: W. Dean, Regional Administrator – Region I, NRC
J. Hughey, Project Manager - US NRC
NRC Senior Resident Inspector – Hope Creek (X24)
P. Mulligan, Manager, NJBNE
LER uploaded to ICES
Hope Creek Commitment Tracking Coordinator (H02)
L. Marabella - Corporate Commitment Tracking Coordinator (N21)

LICENSEE EVENT REPORT (LER)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

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4. TITLE
Automatic Actuation of the Reactor Protection System Due To A Main Turbine Trip

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
12	01	2013	2013	- 008 -	00	01	28	2014	N/A	DOCKET NUMBER
									N/A	DOCKET NUMBER

9. OPERATING MODE 1	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR§: <i>(Check all that apply)</i>			
10. POWER LEVEL 100	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)
	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER
	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify In Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME Philip Duca, Senior Compliance Engineer	TELEPHONE NUMBER (Include Area Code) (856) 339-1640
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX
				N					

14. SUPPLEMENTAL REPORT EXPECTED <input checked="" type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE) <input type="checkbox"/> NO	15. EXPECTED SUBMISSION DATE	MONTH 04	DAY 04	YEAR 14
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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On December 1, 2013, at 06:13 EST, Hope Creek Unit 1 automatically scrambled from 100 percent rated thermal power due to a main turbine trip. The main turbine trip was due to high level in the 'A' moisture separator (MS). As a result of the scram, both reactor recirculation pumps tripped and three safety relief valves opened. The plant was stabilized in hot shutdown Operational Condition 3.

The MS normal level controller failed due to a ruptured bellows causing the MS drain valves to close. The MS dump valve controller attempted to maintain MS level and cycled the dump valve as required. After multiple cycles, the dump valve failed in the closed position. This caused the MS level to increase above the main turbine trip setpoint, resulting in the main turbine trip and the subsequent automatic reactor scram.

A causal evaluation is in progress to determine the cause of the MS dump valve failure. The results of the evaluation will be published in a supplement to this LER.

The MS drain and dump valve controllers were replaced, calibrated, and functionally tested.

This is an event reportable under 10 CFR 50.73(a)(2)(iv)(A) as an event that resulted in actuation of the reactor protection system.

LICENSEE EVENT REPORT (LER)

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NARRATIVE

PLANT AND SYSTEM IDENTIFICATION

General Electric – Boiling Water Reactor {BWR/4}
 Main Turbine – EIS Identifier {TA/TRB}*
 Moisture Separator – EIS Identifier {SB/MSR}*
 Reactor Protection System – EIS Identifier {JC}*
 Reactor Recirculation Pumps – EIS Identifier {AD/P}
 Safety Relief Valves - EIS Identifier {SB/RV}

* Energy Industry Identification System {EIS} codes and component function identifier codes appear as {SS/CCC}

IDENTIFICATION OF EVENT

Event Date: December 1, 2013
 Discovery Date: December 1, 2013

CONDITIONS PRIOR TO EVENT

Hope Creek was in Operational Condition (OPCON) 1 operating at approximately 100 percent rated thermal power. No other structures, systems or components were inoperable at the time of the event.

DESCRIPTION OF EVENT

On December 1, 2013, at 06:13 EST, Hope Creek Unit 1 automatically scrammed from 100 percent rated thermal power due to a trip of the main turbine {TA/TRB}. The main turbine trip was due to a high level in the 'A' moisture separator {SB/MSR}. The main turbine trip caused an actuation of the reactor protection system (RPS){JC} resulting in an automatic reactor scram. Both reactor recirculation pumps {AD/P} tripped per design and three safety relief valves (SRV){SB/RV} lifted. The plant was stabilized in hot shutdown (OPCON 3). All control rods inserted as required and no automatic emergency core cooling system (ECCS) or reactor core isolation cooling (RCIC) system initiations occurred.

A four-hour NRC Emergency Notification System (ENS) notification was required by 10 CFR 50.72(b)(2)(iv)(B) for an actuation of RPS when the reactor was critical. The ENS notification (#49592) was completed on December 1, 2013, at 10:02 EST. This event involved an automatic actuation of RPS; therefore, this LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv)(A).

CAUSE OF EVENT

The MS normal level controller failed due to a ruptured bellows causing the MS drain valves to close. The MS dump valve controller attempted to maintain level by cycling the MS dump valve as required; however, after 14 cycles, the dump valve failed in the closed position. This caused MS level to increase above the high level setpoint, resulting in the main turbine trip and the subsequent automatic reactor scram.

A causal evaluation is in progress to determine the cause of the MS dump valve failure. The results of the evaluation will be published in a supplement to this LER.

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NARRATIVE

SAFETY CONSEQUENCES AND IMPLICATIONS

There was no safety consequence associated with this event. The high moisture level in the 'A' MS resulted in a main turbine trip and subsequent automatic reactor scram. All reactor protection systems functioned as designed. All control rods inserted. All systems responded as expected to the turbine trip. No ECCS or RCIC initiation setpoints were reached. The plant was stabilized in hot shutdown (OPCON 3).

SAFETY SYSTEM FUNCTIONAL FAILURE

A review of this event determined that a Safety System Functional Failure (SSFF) did not occur as defined in Nuclear Energy Institute (NEI) 99-02, "Regulatory Assessment Performance Indicator Guideline." This event did not prevent the ability of a system to fulfill its safety function to either shutdown the reactor, remove residual heat, control the release of radioactive material, or mitigate the consequences of an accident.

PREVIOUS EVENTS

A review of events for the past three years at Hope Creek was performed to determine if a similar event had occurred. No occurrences were identified.

CORRECTIVE ACTIONS

1. The MS drain valves and dump valve controllers were replaced, calibrated, and functionally tested.

COMMITMENTS

This LER contains no commitments.