# APPENDIX A IMPROVED RAOC F<sub>Q</sub> SURVEILLANCE TS

F<sub>o</sub>(Z) (RAOC-W(Z) Methodology) 3.2.1B

3.2 POWER DISTRIBUTION LIMITS

3.2.1B Heat Flux Hot Channel Factor (Fo(Z) (RAOC-W(Z) Methodology)

LCO 3.2.1B  $F_{Q}(Z)$ , as approximated by  $F_{Q}^{C}(Z)$  and  $F_{Q}^{W}(Z)$ , shall be within the limits specified in the COLR.

APPLICABILITY: MODE 1.

#### ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	NOTE Required Action A.4 shall be completed whenever this Condition is entered.	A.1 <u>AND</u>	Reduce THERMAL POWER ≥ 1% RTP for each 1% $F_{\alpha}^{C}(Z)$ exceeds limit.	15 minutes after each F <sup>C</sup> <sub>Q</sub> (Z) determination
	$F_{\alpha}^{c}(Z)$ not within limit.	A.2	Reduce Power Range Neutron Flux - High trip setpoints ≥ 1% for each 1% <del>FC (Z) exceeds limits</del> [	72 hours after each $F_{Q}^{C}(Z)$ determination
		AND		below RATED THERMAL POWER by Required Action A.1.
		A.3	Reduce Overpower ∆T trip setpoints ≥ 1% for each 1% <del>F<sub>C</sub>(Z) exceeds limit.</del>	72 hours after each $F_{\alpha}^{c}(Z)$ determination
		AND		
		A.4	Perform SR 3.2.1.1 and SR 3.2.1.2.	Prior to increasing THERMAL POWER above the limit of Required Action A.1

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Fo(Z) (RAOC-W(Z) Methodology) 3.2.1B

### SURVEILLANCE REQUIREMENTS

NOTE-During power escalation at the beginning of each cycle, THERMAL POWER may be increased until an equilibrium power level has been achieved, at which a power distribution map is obtained.

	SURVEILLANCE	FREQUENCY
SR 3.2.1.1 Ve	rify F <sup>c</sup> <sub>0</sub> (Z) is within limit.	Once after each refueling prior to THERMAL POWER exceeding 75% RTP <u>AND</u> Once within [42] hours after achieving equilibrium conditions after exceeding, by ≥ 10% RTP, the THERMAL POWER at which F <sub>0</sub> (Z) was last verified <u>AND</u> [31 EFPD thereafter <u>OR</u> In accordance with the Surveillance Frequency Control Program]
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	SURVEILLANCE		FREQUENCY
			[ 31 EFPD thereafter
			OR
			In accordance with the Surveillance Frequency Control Program ]
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## APPENDIX B IMPROVED RAOC F<sub>Q</sub> SURVEILLANCE TECHNICAL SPECIFICATION BASES

Fo(Z) (RAOC-W(Z) Methodology) B 3.2.1B

**B 3.2 POWER DISTRIBUTION LIMITS** 

B 3.2.1B Heat Flux Hot Channel Factor (Fo(Z) (RAOC-W(Z) Methodology)

BASES

BACKGROUND

The purpose of the limits on the values of  $F_Q(Z)$  is to limit the local (i.e., pellet) peak power density. The value of  $F_Q(Z)$  varies along the axial height (Z) of the core.

 $\mathsf{F}_Q(Z)$  is defined as the maximum local fuel rod linear power density divided by the average fuel rod linear power density, assuming nominal fuel pellet and fuel rod dimensions. Therefore,  $\mathsf{F}_Q(Z)$  is a measure of the peak fuel pellet power within the reactor core.

During power operation, the global power distribution is limited by LCO 3.2.3, "AXIAL FLUX DIFFERENCE (AFD)," and LCO 3.2.4, "QUADRANT POWER TILT RATIO(QPTR)," which are directly and continuously measured process variables. These LCOs, along with LCO 3.1.6, "Control Bank Insertion Limits," maintain the core limits on power distributions on a continuous basis.

Equilibrium conditions are defined as being at a stable reactor power (i.e., within  $\pm$  1% (RTP) and at stable axial flux conditions (i.e., with an axial flux difference variability of  $\pm$  1% over the previous 24 hours). Equilibrium conditions are not required for performance of the F<sub>c</sub>(z) measurements during power ascension following refueling, but plant conditions should be stable during the data collection period (i.e., power within  $\pm$ 1% RTP of the desired power level and axial offset changing by less than 0.5% from the start to the end of the flux map data collection).

the elevation dependent measured planar radial peaking factors,  $F_{\rm XY}(z)$ , are increased by an elevation dependent factor,  $[T(z)]^{\rm COLR}$ , that accounts for the expected maximum values of the transient axial power shapes postulated to occur during RAOC operation. Thus,  $[T(z)]^{\rm COLR}$  accounts for the worst case non-equilibrium power shapes that are expected for the assumed RAOC operating space.

 $F_{Q}(Z)$  varies with fuel loading patterns, control bank insertion, fuel burnup, and changes in axial power distribution.

 $F_o(Z)$  is measured periodically using the incore detector system. These measurements are generally taken with the core at or near equilibrium conditions.

Using the measured three dimensional power distributions, it is possible to derive a measured value for  $F_Q(Z)$ . However, because this value represents an equilibrium condition, it does not include the variations in the value of  $F_Q(Z)$  which are present during nonequilibrium situations such as load following or power ascension.

To account for these possible variations, the equilibrium value of F<sub>C</sub>(Z) is adjusted as F<sup>W</sup><sub>4</sub>(Z) by an elevation dependent factor that accounts for the calculated worst case transient conditions.

Core monitoring and control under non-equilibrium conditions are accomplished by operating the core within the limits of the appropriate LCOs, including the limits on AFD, QPTR, and control rod insertion.

The RAOC operating space is defined as the combination of AFD and Control Bank insertion Limits assumed in the calculation of a particular  $[T(z)]^{COLR}$  function. The  $[T(z)]^{COLR}$  factors are directly dependent on the AFD and Control Bank Insertion Limit assumptions. The COLR may contain different  $[T(z)]^{COLR}$  functions that reflect different operating space assumptions. If the limit on  $F_C(z)$  is exceeded, a more restrictive operating space may be implemented to gain margin for future non-equilibrium operation.

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APPLICABLE SAFETY	This LCO precludes core power distributions that violate the following fuel design criteria:				
ANALISES	<ul> <li>During a large break loss of coolant accident (LOCA), the peak cladding temperature must not exceed 2200°F (Ref. 1),</li> </ul>				
	b. During a loss of forced reactor coolant flow accident, there must be a least 95% probability at the 95% confidence level (the 95/95 DNB criterion) that the hot fuel rod in the core does not experience a departure from nucleate boiling (DNB) condition,				
	<ul> <li>During an ejected rod accident, the energy deposition to the fuel must not exceed 280 cal/gm (Ref. 2), and</li> </ul>				
	d. The control rods must be capable of shutting down the reactor with a minimum required SDM with the highest worth control rod stuck fully withdrawn (Ref. 3).				
	Limits on F <sub>O</sub> (Z) ensure that the value of the initial total peaking factor assumed in the accident analyses remains valid. Other criteria must also be met (e.g., maximum cladding oxidation, maximum hydrogen generation, coolable geometry, and long term cooling). However, the peak cladding temperature is typically most limiting.				
	Limits on $F_0(Z)$ ensure that the value of the initial total peaking factor assumed in the accident analyses remains valid. Other criteria must also be met (e.g., maximum cladding oxidation, maximum hydrogen generation, coolable geometry, and long term cooling). However, the peak cladding temperature is typically most limiting. $F_0(Z)$ limits assumed in the LOCA analysis are typically limiting relative to (i.e., lower than) the $F_0(Z)$ limit assumed in safety analyses for other postulated accidents. Therefore, this LCO provides conservative limits fo other postulated accidents				
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LCO	$\begin{array}{llllllllllllllllllllllllllllllllllll$				
LCO	Limits on $F_o(Z)$ ensure that the value of the initial total peaking factor assumed in the accident analyses remains valid. Other criteria must also be met (e.g., maximum cladding oxidation, maximum hydrogen generation, coolable geometry, and long term cooling). However, the peak cladding temperature is typically most limiting. $F_o(Z)$ limits assumed in the LOCA analysis are typically limiting relative to (i.e., lower than) the $F_o(Z)$ limit assumed in safety analyses for other postulated accidents. Therefore, this LCO provides conservative limits for other postulated accidents $F_o(Z)$ satisfies Criterion 2 of 10 CFR 50.36(c)(2)(ii). The Heat Flux Hot Channel Factor, $F_o(Z)$ , shall be limited by the following relationships: $F_o(Z) \le (CFQ / P) K(Z)$ for $P > 0.5$ $F_o(Z) \le (CFQ / 0.5) K(Z)$ for $P > 0.5$ where: CFQ is the $F_o(Z)$ limit at RTP provided in the COLR,				
LCO	Limits on $F_0(Z)$ ensure that the value of the initial total peaking factor assumed in the accident analyses remains valid. Other criteria must also be met (e.g., maximum cladding oxidation, maximum hydrogen generation, coolable geometry, and long term cooling). However, the peak cladding temperature is typically most limiting. $F_0(Z)$ limits assumed in the LOCA analysis are typically limiting relative to (i.e., lower than) the $F_0(Z)$ limit assumed in safety analyses for other postulated accidents. Therefore, this LCO provides conservative limits for other postulated accidents $F_0(Z)$ satisfies Criterion 2 of 10 CFR 50.36(c)(2)(ii). The Heat Flux Hot Channel Factor, $F_0(Z)$ , shall be limited by the following relationships: $F_0(Z) \le (CFQ / P) K(Z)$ for $P > 0.5$ $F_0(Z) \le (CFQ / 0.5) K(Z)$ for $P > 0.5$ where: CFQ is the $F_0(Z)$ limit at RTP provided in the COLR, K(Z) is the normalized $FQ(Z)$ as a function of core height provided in the COLR, and				

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#### BASES

LCO (continued)

2.50

For this facility, the actual values of CFQ and K(Z) are given in the COLR; however, CFQ is normally a number on the order of [2:32], and K(Z) is a function that looks like the one provided in Figure B 3.2.1B-1.

For Relaxed Axial Offset Control operation, Fo(Z) is approximated by  $F_{\alpha}^{c}(Z)$  and  $F_{\alpha}^{w}(Z)$ . Thus, both  $F_{\alpha}^{c}(Z)$  and  $F_{\alpha}^{w}(Z)$  must meet the preceding limits on Fo(Z).

An F<sub>0</sub><sup>C</sup>(Z) evaluation requires obtaining an incore flux map in MODE 1. From the incore flux map results we obtain the measured value  $(F_{\alpha}^{M}(Z))$  of Fo(Z). Then,

 $F_{0}^{c}(Z) = F_{0}^{M}(Z)$  [1.0815]

where [1.0815] is a factor that accounts for fuel manufacturing tolerances and flux map measurement uncertainty.

 $F_{\alpha}^{c}(Z)$  is an excellent approximation for  $F_{\alpha}(Z)$  when the reactor is at the steady state power at which the incore flux map was taken.

The expression for  $F_0^w(Z)$  is:  $F_{\alpha}^{W}(Z) = F_{\alpha}^{C}(Z) W(Z) \bigstar$ 

FxyM(z) [T(z)]COLR Axy(z) Rj [1.0815]



Violating the LCO limits for Fo(z) could

result in unacceptable consequences if a design basis event were to occur ile Fo(z) exceeds its specified limits.

a more restrictive RAOC operating

duced

space must be implemented or core ower limits and AFD limits must be

where W(Z) is a cycle dependent function that accounts for power distribution transients encountered during normal operation. W(Z) is included in the COLR. The FC(Z) is calculated at equilibrium conditions.

The Fo(Z) limits define limiting values for core power peaking that precludes peak cladding temperatures above 2200°F during either a large or small break LOCA.

This LCO, requires operation within the bounds assumed in the safety analyses. Calculations are performed in the core design process to confirm that the core can be controlled in such a manner during operation that it can stay within the LOCA  $F_0(Z)$  limits. If  $F_0^c(Z)$  cannot be maintained within the LCO limits, reduction of the core power is required and if Fow (Z) cannot be maintained within the LCO limits reduction of the AFD limits is required. Note that sufficient reduction of the AFD limits will also result in a reduction of the core power.

Violating the LCO limits for Fo(Z) produces unacceptable consequences if a design basis event occurs while Fe(Z) is outside its specified limits.

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#### **INSERT 1**

The various factors in this expression are defined below:

 $F_{XY}^{M}(z)$  is the measured radial peaking factor at axial location z and is equal to the value of  $F_{O}^{M}(z)/P^{M}(z)$ , where  $P^{M}(z)$  is the measured core average axial power shape.

 $[T(z)]^{COLR}$  is the cycle and burnup dependent function, specified in the COLR, which accounts for power distribution transients encountered during non-equilibrium normal operation.  $[T(z)]^{COLR}$  functions are specified for each analyzed RAOC operating space (i.e., each unique combination of AFD limits and Control Bank Insertion Limits). The  $[T(z)]^{COLR}$  functions account for the limiting non-equilibrium axial power shapes postulated to occur during normal operation for each RAOC operating space. Limiting power shapes at both full and reduced power operation are considered in determining the maximum values of  $[T(z)]^{COLR}$ . The  $[T(z)]^{COLR}$  functions also account for the

following effects: (1) the presence of spacer grids in the fuel assembly, (2) the increase in radial peaking in rodded core planes due to the presence of control rods during non-equilibrium normal operation, (3) the increase in radial peaking that occurs during part-power operation due to reduced fuel and moderator temperatures, and (4) the increase in radial peaking due to non-equilibrium xenon effects. The  $[T(z)]^{COLR}$  functions are normally calculated assuming that the Surveillance is performed at nominal RTP conditions with all shutdown and control rods fully withdrawn, i.e., all rods out (ARO). Surveillance specific  $[T(z)]^{COLR}$  values may be generated for a given surveillance core condition.

#### P is the THERMAL POWER / RTP.

 $A_{XY}(z)$  is a cycle, burnup, and power dependent function specified in the COLR or determined at the time of the surveillance using an approved 3D core model and the methodology of Reference 6. It adjusts the measured  $F_{XY}^{M}(z)$  to the reference core conditions assumed in generating the  $[T(z)]^{COLR}$  factors. Normally, this reference core condition is HFP, ARO, EQXE. For simplicity,  $A_{XY}(z)$  may be assumed to be 1.0. If, however, margin is needed for a Surveillance performed at conditions different from the reference core conditions, then the appropriate values for  $A_{XY}(z)$  may be used. Sub-factors of  $A_{XY}(z)$  may also be determined and included in the COLR. These sub-factors are  $F_{PC}(z)$  and  $F_{RC}(z)$ .  $F_{PC}(z)$  is a factor that adjusts the measured  $F_{XY}^{M}(z)$  to the reference core power (typically RTP) if the Surveillance is performed at part power conditions.  $F_{RC}(z)$  is a factor that adjusts the measured  $F_{XY}^{M}(z)$  values to the reference condition includes insertion of the lead control bank. When these sub-factors are used,  $A_{XY}(z)$  is the product of  $F_{PC}(z)$  and  $F_{RC}(z)$ .

[1.0815] is a factor that accounts for fuel manufacturing tolerances and measurement uncertainty.

Rj is a cycle and burnup dependent analytical factor specified in the COLR that accounts for potential increases in  $F_0^{W}(z)$  between Surveillances. Rj values are provided for each RAOC operating space.

#### -REVIEWER'S NOTE----

WCAP-17661-P-A, "Improved RAOC and CAOC F<sub>o</sub> Surveillance Technical Specifications," or other appropriate plant specific methodology, is to be listed in the COLR description in the Administrative Controls Section 5.0 to address the methodology used to derive this factor.

	The $F_Q(Z)$ limits must be maintained in MODE 1 to previde the function of the second stributions from exceeding the limits assumed in the second stributions from exceeding the limits assumed in the second stributions from exceeding the limits assumed in the second stribution of the function of the second stribution o	ent core power afety analyses. ere is either ly being e distribution of
ACTIONS	<u>A.1</u>	ra ann ann ann ann ann ann ann ann ann a
	Reducing THERMAL POWER by $\ge 1\%$ RTP for each 19 exceeds its limit, maintains an acceptable absolute power is $F_0^{A}(2)$ multiplied by a factor accounting for manufactur and measurement uncertainties. $F_0^{A}(2)$ is the measure The Completion Time of 15 minutes provides an accept reduce power in an orderly manner and without allowing remain in an unacceptable condition for an extended per maximum allowable power level initially determined by F A.1 may be affected by subsequent determinations of F require power reductions within 15 minutes of the $F_0^{C}(2)$ necessary to comply with the decreased maximum allow Decreases in $F_0^{C}(2)$ would allow increasing the maximum power level and increasing power up to this revised limit <b>A.2</b> Intel THERMAL POWER is limited below RATED THERMAL POWER by Required Action A.1 A reduction of the Power Range Neutron Flux - High trip 2 1% for each 1% by which $F_0^{C}(2)$ exceeds its limit, is a action for protection against the consequences of severe unanalyzed power distributions. The Completion Time of sufficient considering the small likelihood of a severe trap period and the preceding prompt reduction in THERMAL accordance with Required Action A.1. The maximum al Range Neutron Flux - High trip setpoints initially determina- Action A.2 may be affected by subsequent determination would require Power Range Neutron Flux - High trip set within 72 hours of the $F_0^{C}(2)$ determination, if necessary the decreased maximum allowable Power Range Neutron setpoints. Decreases in $F_0^{C}(2)$ would allow increasing the allowable Power Range Neutron Flux - High trip setpoints allowable Power Range Neutron Flux - High trip setpoints.	6 by which $F_{\alpha}^{c}(Z)$ er density. $F_{\alpha}^{c}(Z)$ ing tolerances i value of $F_{0}(Z)$ . able time to the plant to riod of time. The Required Action G(Z) and would determination, if vable power level, m allowable e transients with of 72 hours is nsient in this time. POWER in lowable Power ned by Required ns of $F_{\alpha}^{c}(Z)$ and point reductions to comply with on Flux - High trip he maximum ts.
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#### BASES

ACTIONS (continued)

that THERMAL POWER is limited below RATED THERMAL POWER by Required Action A 1

Reduction in the Overpower  $\Delta T$  trip setpoints (value of K<sub>4</sub>) by  $\geq 1\%$  for each 1% by which  $F_{C}^{c}(Z)$  exceeds its limit, is a conservative action for protection against the consequences of severe transients with unanalyzed power distributions. The Completion Time of 72 hours is sufficient considering the small likelihood of a severe transient in this time period, and the preceding prompt reduction in THERMAL POWER in accordance with Required Action A.1. The maximum allowable Overpower  $\Delta T$  trip setpoints initially determined by Required Action A.3 may be affected by subsequent determinations of  $F_{\alpha}^{c}(Z)$  and would require Overpower  $\Delta T$  trip setpoint reductions within 72 hours of the  $F_{\alpha}^{c}(Z)$  determination, if necessary to comply with the decreased maximum allowable Overpower  $\Delta T$  trip setpoints. Decreases in  $F_{\alpha}^{c}(Z)$  would allow increasing the maximum allowable Overpower  $\Delta T$  trip setpoints.

#### <u>A.4</u>

<u>A.3</u>

Verification that  $F_{\alpha}^{c}(Z)$  has been restored to within its limit, by performing SR 3.2.1.1 and SR 3.2.1.2 prior to increasing THERMAL POWER above the limit imposed by Required Action A.1<sub>7</sub> ensures that core conditions during operation at higher power levels and future operation are consistent with safety analyses assumptions.

Condition A is modified by a Note that requires Required Action A.4 to be performed whenever the Condition is entered. This ensures that SR 3.2.1.1 and SR 3.2.1.2 will be performed prior to increasing THERMAL POWER above the limit of Required Action A.1, even when Condition A is exited prior to performing Required Action A.4. Performance of SR 3.2.1.1 and SR 3.2.1.2 are necessary to assure  $F_0(Z)$  is properly evaluated prior to increasing THERMAL POWER.

#### <u>B.1</u>

If it is found that the maximum calculated value of  $F_Q(Z)$  that can occur during normal maneuvers,  $F_Q^w(Z)$ , exceeds its specified limits, there exists a potential for  $F_Q^c(Z)$  to become excessively high if a normal operational transient occurs. Reducing the AFD by  $\geq 1\%$  for each 1% by which  $F_Q^w(Z)$  exceeds its limit within the allowed Completion Time of 4 hours, restricts the axial flux distribution such that even if a transient occurred, core peaking factors are not exceeded.

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Implementing a more restrictive RAOC operating space, as specified in the COLR, within the allowed Completion

Time of 4 hours will restrict the AFD such that peaking factor limits will not be exceeded during non-equilibrium

normal operation. Several RAOC operating spaces, representing

and, optionally, shallower Control Bank Insertion Limits, may be

successively smaller AFD envelopes

specified in the COLR. The corresponding T(z) functions for thes

operating spaces can be used to determine which RAOC operating space will result in acceptable non-

rium operation within the FoW(z)

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#### **INSERT 2**

#### <u>B.2.1</u>

When  $F_o^w(z)$  exceeds its limit, Required Action B.2.1 may be implemented instead of Required Action B.1. Required Action B.2.1 limits THERMAL POWER to less than RATED THERMAL POWER by the amount specified in the COLR. It also requires reductions in the AFD limits by the amount specified in the COLR. If the RAOC operating spaces specified in the COLR are insufficient to ensure margin to the  $F_o^w(z)$ limit, then Required Action B.2.1 must be entered and THERMAL POWER must be limited to less than the RATED THERMAL POWER and AFD limits must be reduced by the amounts specified in the COLR. This maintains an acceptable absolute power density relative to the maximum power density value assumed in the safety analyses.

The Completion Time of 4 hours provides an acceptable time to reduce the THERMAL POWER and AFD limits in an orderly manner to preclude entering an unacceptable condition during future non-equilibrium operation. The limit on THERMAL POWER initially determined by Required Action B.2.1 may be affected by subsequent determinations of  $F_{o}^{W}(z)$  and would require power reductions within 4 hours of the  $F_{o}^{W}(z)$  determination, if necessary to comply with the decreased THERMAL POWER limit. Decreases in  $F_{o}^{W}(z)$  would allow increasing the THERMAL POWER limit and increasing THERMAL POWER up to this revised limit.

Required Action B.2.1 is modified by a Note that states Required Action B.2.4 shall be completed whenever Required Action B.2.1 is performed. Required Action B.2.4 requires the performance of SR 3.2.1.1 and SR 3.2.1.2 prior to increasing THERMAL POWER above the limit established by Required Action B.2.1. The Note ensures that the SRs will be performed even if Condition B may be exited prior to performing Required Action B.2.4. The performance of SR 3.2.1.1 and SR 3.2.1.2 is necessary to assure  $F_Q(Z)$  is properly evaluated prior to increasing THERMAL POWER.

ACTIONS (continue	ed)	2.4
	<u>C.1</u>	7
	If Required Actions A.1 through A.4 or B.1 through B.4 their associated Completion Times, the plant must be p condition in which the LCO requirements are not applic by placing the plant in at least MODE 2 within 6 hours.	are not met within laced in a mode or able. This is done
	This allowed Completion Time is reasonable based on experience regarding the amount of time it takes to rea- full power operation in an orderly manner and without c systems.	operating ch MODE 2 from hallenging plant
SURVEILLANCE REQUIREMENTS	SR 3.2.1.1 and SR 3.2.1.2 are modified by a Note. The during the first power ascension after a refueling. It sta THERMAL POWER may be increased until an equilibri has been achieved at which a power distribution map c This allowance is modified, however, by one of the Free that requires verification that $F_G^c(Z)$ and $F_G^w(Z)$ are with limits after a power rise of more than 10% RTP over the POWER at which they were last verified to be within sp Because $F_G^c(Z)$ and $F_G^w(Z)$ could not have previously be this reload core, there is a second Frequency condition for reload cores, that requires determination of these pa exceeding 75% RTP. This ensures that some determin $F_G^w(Z)$ are made at a lower power level at which adequa available before going to 100% RTP. Also, this Freque together with the Frequency condition requiring verifica $F_G^w(Z)$ following a power increase of more than 10%, et achieved. In the absence of these Frequency condition increase power to RTP and operate for 31 days without $F_G^c(Z)$ and $F_G^w(Z)$ . The Frequency condition is not inter verification of these parameters after every 10% increase above the last verification. It only requires verification at is achieved for extended operation that is 10% higher the which $F_Q(Z)$ was last measured.	Note applies tes that um power level an be obtained. quency conditions in their specified a THERMAL ecified limits. een measured in , applicable only arameters before nation of $F_{\alpha}^{C}(Z)$ and ate margin is ncy condition, tion of $F_{\alpha}^{C}(Z)$ and nsures that they inded operation) is is, it is possible to a verification of need to require se in power level ofter a power level than that power at

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Fo(Z) (RAOC-W(Z) Methodology)



#### BASES

#### SURVEILLANCE REQUIREMENTS (continued)

#### SR 3.2.1.1

some determination of  $F_O^{-c}(2)$  is made prior to achieving a significant power level where the peak linear heat rate could approach the limits assumed in the safety analyses.

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This Frequency condition is not intended to require verification of these parameters after every 10% increase in RTP above the THERMAL POWER at which the last verification was performed. It only requires verification after a THERMAL POWER is achieved for extended operation that is 10% higher than the THERMAL POWER at which Fo<sup>C</sup>(2) was last measured. Verification that  $F_{\alpha}^{c}(Z)$  is within its specified limits involves increasing  $F_{\alpha}^{M}(Z)$  to allow for manufacturing tolerance and measurement uncertainties in order to obtain  $F_{\alpha}^{c}(Z)$ . Specifically,  $F_{\alpha}^{M}(Z)$  is the measured value of  $F_{\alpha}(Z)$  obtained from incore flux map results and  $F_{\alpha}^{c}(Z) = F_{\alpha}^{M}(Z)$  [1.0815] (Ref. 4).  $F_{\alpha}^{c}(Z)$  is then compared to its specified limits.

The limit with which  $F_{G}^{c}(Z)$  is compared varies inversely with power above 50% RTP and directly with a function called K(Z) provided in the COLR.

Performing this Surveillance in MODE 1 prior to exceeding 75% RTP ensures that the FS(Z) limit is met when RTP is achieved, because peaking factors generally decrease as power level is increased.

If THERMAL POWER has been increased by  $\ge$  10% RTP since the last determination of  $F_{G}^{c}(Z)$ , another evaluation of this factor is required [42] hours after achieving equilibrium conditions at this higher power level

(to ensure that  $F_{\alpha}^{c}(Z)$  values are being reduced sufficiently with power increase to stay within the LCO limits).

The Frequency of 31 EFPD is adequate to monitor the change of power distribution with core burnup because such changes are slow and well controlled when the plant is operated in accordance with the Technical Specifications (TS).

In the absence of these Frequency conditions (discussed above) it is possible to operate for 31 EFPD without verification of  $F_0^{\rm C}(z)$ .

OR

The Surveillance Frequency is controlled under the Surveillance Frequency Control Program.

REVIEWER'S NOTE-Plants controlling Surveillance Frequencies under a Surveillance Frequency Control Program should utilize the appropriate Frequency description, given above, and the appropriate choice of Frequency in the Surveillance Requirement.

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#### BASES

#### SURVEILLANCE REQUIREMENTS (continued)

b.

#### SR 3.2.1.2

The nuclear design process includes calculations performed to determine that the core can be operated within the  $F_O(Z)$  limits. Because flux maps are taken in steady state conditions, the variations in power distribution resulting from normal operational maneuvers are not present in the flux map data. These variations are, however, conservatively calculated by considering a wide range of unit maneuvers in normal operation. The maximum peaking factor increase over steady state values, calculated as a function of core elevation, Z, is called W(Z). Multiplying the measured total peaking factor,  $F_O^{c}(Z)$ , by W(Z) gives the maximum  $F_O(Z)$  calculated to occur in normal operation,  $F_W^{cr}(Z)$ .



Grid plane regions, ± 2% inclusive, and Core plane regions, within ± 2% of the bank demand position of the control banks.

These regions

The excluded regions at the top and bottom of the core are specified in the COLR and are defined to ensure that the minimum margin location is adequately surveilled. A slightly smaller exclusion zone may be specified, if necessary, to include the limiting margin location in the surveilled region of the core. The limit with which  $F_{q}^{w}(Z)$  is compared varies inversely with power above 50% RTP and directly with the function K(Z) provided in the COLR.

The W(Z) curve is provided in the COLR for discrete core elevations. Flux map data are typically taken for 30 to 75 core elevations.  $F_{\alpha}^{w}(Z)$  evaluations are not applicable for the following axial core regions, measured in percent of core height:

a. Lower core region, from 0 to 15% inclusive

Upper core region, from 85 to 100% inclusive

The top and bottom 15% of the core are excluded from the evaluation because of the low probability that these regions would be more limiting in the safety analyses and because of the difficulty of making a precise measurement in these regions.

This Surveillance has been modified by a Note that may require that more frequent surveillances be performed. If  $F_{\alpha}^{w}(Z)$  is evaluated, an evaluation of the expression below is required to account for any increase to  $F_{\alpha}^{w}(Z)$  that may occur and cause the  $F_{\alpha}(Z)$  limit to be exceeded before the next required  $F_{\alpha}(Z)$  evaluation.

If the two most recent  $F_0(Z)$  evaluations show an increase in the expression maximum over  $z [F_0^c(Z) / K(Z)]$ , it is required to meet the  $F_0(Z)$  limit with the last  $F_0^w(Z)$  increased by the greater of a factor of [1.92] or by an appropriate factor specified in the COLR (Ref. 5)

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#### INSERT 3

The measured  $F_o(z)$  can be determined through a synthesis of the measured planar radial peaking factors,  $F_{XY}^{M}(z)$ , and the measured core average axial power shape,  $P^{M}(z)$ . Thus,  $F_o^{C}(z)$  is given by the following expression:

 $F_{Q}^{C}(z) = F_{XY}^{M}(z) P^{M}(z) [1.0815] = F_{Q}^{M}(z) [1.0815]$ 

For RAOC operation, the analytical  $[T(z)]^{COLR}$  functions, specified in the COLR for each RAOC operating space, are used together with the measured  $F_{XY}(z)$  values to estimate  $F_O(z)$  for non-equilibrium operation within the RAOC operating space. When the  $F_{XY}(z)$  values are measured at HFP ARO conditions ( $A_{XY}(z)$  equals 1.0),  $F_O^W(z)$  is given by the following expression:

 $F_0^W(z) = F_{XY}^M(z) [T(z)]^{COLR} Rj [1.0815]$ 

Non-equilibrium operation can result in significant changes to the axial power shape. To a lesser extent, non-equilibrium operation can increase the radial peaking factors,  $F_{XY}(z)$ , through control rod insertion and through reduced Doppler and moderator feedback at part-power conditions.

The  $[T(z)]^{COLR}$  functions quantify these effects for the range of power shapes, control rod insertion, and power levels characteristic of the operating space. Multiplying  $[T(z)]^{COLR}$  by the measured full power, unrodded  $F_{XY}^{M}(z)$  value, and the factor that accounts for manufacturing and measurement uncertainties gives  $F_{O}^{W}(z)$ , the maximum total peaking factor postulated for non-equilibrium RAOC operation.

#### BASES

SURVEILLANCE REQUIREMENTS (continued)

WCAP-10216-P-A, Rev. 1A, "Relaxation of Constant Axial Offset Control and F<sub>o</sub> Surveillance Technical Specification," February 1994, or other appropriate plant specific methodology, is to be listed in the COLR description in the Administrative Controls Section 5.0 to address the methodology used to derive this factor.

or to evaluate  $F_{\alpha}(Z)$  more frequently, each 7 EFPD. These alternative requirements prevent  $F_{\alpha}(Z)$  from exceeding its limit for any significant period of time without detection.

Performing the Surveillance in MODE 1 prior to exceeding 75% RTP ensures that the  $F_0(2)$  limit is met when RTP is achieved, because peaking factors are generally decreased as power level is increased.

 $F_0(Z)$  is verified at power levels  $\geq$  10% RTP above the THERMAL POWER of its last verification, [12] hours after achieving equilibrium conditions to ensure that  $F_0(Z)$  is within its limit at higher power levels.

The Surveillance Frequency of 31 EFPD is adequate to monitor the

change of power distribution with core burnup. The Surveillance may be done more frequently if required by the results of  $F_o(Z)$  evaluations.

INSERT 4 (Next Page)

In the absence of these Frequency conditions (discussed above) it is possible to operate for 31 EFPD without verification of Fo<sup>VI</sup>(z). The Frequency of 31 EFPD is adequate to monitor the change of power distribution because such a change is sufficiently slow, when the plant is operated in accordance with the TS, to preclude adverse peaking factors

OR

between 31 day surveillances.

The Surveillance Frequency is controlled under the Surveillance Frequency Control Program.

Plants controlling Surveillance Frequencies under a Surveillance Frequency Control Program should utilize the appropriate Frequency description, given above, and the appropriate choice of Frequency in the Surveillance Requirement.

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#### **INSERT 4**

SR 3.2.1.2 requires a Surveillance of  $F_0^{W}(z)$  during the initial startup following each refueling within [24] hours after achieving equilibrium conditions after exceeding 75% RTP. THERMAL POWER levels below 75% are typically non-limiting with respect to the limit for  $F_0^{W}(z)$ . Also, initial startups following a refueling are slow and well controlled due to startup ramp rate limitations and fuel conditioning requirements. Furthermore, startup physics testing and flux symmetry measurements, also performed at low power, provide confirmation that the core is operating as expected. Consequently, the initial startup following a refueling will not result in non-equilibrium power shapes that could challenge the  $F_0^{W}(z)$  limit. This Frequency ensures that verification of  $F_0^{W}(z)$  is performed prior to extended operation at high power levels where the maximum permitted peak LHR could be challenged by non-equilibrium operation.

If a previous Surveillance of  $F_0^w(z)$  was performed at part power conditions, SR 3.2.1.2 also requires that  $F_0^w(z)$  be verified at power levels  $\geq 10\%$  RTP above the THERMAL POWER of its last verification within [24] hours after achieving equilibrium conditions. This ensures that  $F_0^w(z)$  is within its limit using radial peaking factors measured at the higher power level.

		F <sub>Q</sub> (Z) (RAOC-W(Z) Methodology B 3.2.1E
BASES		
REFERENCES	1.	10 CFR 50.46, 1974.
	2.	Regulatory Guide 1.77, Rev. 0, May 1974.
	3.	10 CFR 50, Appendix A, GDC 26.
	4.	WCAP-7308-L-P-A, "Evaluation of Nuclear Hot Channel Factor Uncertainties," June 1988.
	5.	WCAP-10216-P-A, Rev. 1A, "Relaxation of Constant Axial Offset Control (and) F <sub>o</sub> Surveillance Technical Specification," February 1994.
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Figure B 3.2.1B-1 (page 1 of 1) K(Z) - Normalized F<sub>Q</sub>(Z) as a Function of Core Height

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## APPENDIX C SAMPLE COLR INPUT FOR A RAOC PLANT

In this appendix, sample  $F_Q$  Surveillance COLR data for a RAOC plant are presented. Note that only those aspects of the COLR pertinent to  $F_Q$  Surveillance are presented in this appendix. Those aspects include COLR data related to Axial Flux Difference, Control Bank Insertion Limits, and the Heat Flux Hot Channel Factor. The approved version of this report must be added to the list of COLR references in Technical Specification 5.6.5, "Core Operating Limits Report (COLR)," since methodology described in this report will be used to determine core operating limits.

## C.1 COLR

This COLR for Plant A Cycle XY has been prepared in accordance with the requirements of Technical Specification 5.6.5.

The TSs affected by this report are:

- 3.1.6 Control Bank Insertion Limits
- 3.2.1 Heat Flux Hot Channel Factor  $-F_Q(z)$

3.2.3 Axial Flux Difference

## C.2 OPERATING LIMITS

The cycle-specific parameter limits and associated data for the specifications listed in Section C.1 are presented in the following subsections. These limits and data have been developed using NRC-approved methodologies including those specified in TS 5.6.5.

### C.2.1 Control Bank Insertion Limits (Specification 3.1.6)

C.2.1.1:

Control Bank Insertion Limits are provided for three ROSs. The Control Bank Insertion Limits for each ROS shall be used in conjunction with the associated Axial Flux Difference Limits for the ROS. The control rod banks shall be limited in physical insertion as shown in Figure C-1 for ROS1 and ROS2. For ROS3, the control rod banks shall be limited in physical insertion as shown in Figure C-2.

## C.2.2 Heat Flux Hot Channel Factor – F<sub>Q</sub>(z) (Specification 3.2.1)

C.2.2.1:

$$F_Q^C(z) \le \frac{F_Q^{RTP}}{P} * K(z) \quad for P > 0.5$$
$$F_Q^C(z) \le \frac{F_Q^{RTP}}{0.5} * K(z) \quad for P \le 0.5$$

where:

$$F_Q^C(z) = F_Q^M(z) * 1.0815$$
$$P = \frac{THERMAL POWER}{RATED THERMAL POWER}$$

C.2.2.2:

 $F_O^{RTP} = 2.50$ 

C.2.2.3:

K(z) is provided in Figure C-3.

C.2.2.4:

$$\begin{split} F_Q^W(z) &\leq \frac{F_Q^{RTP}}{P} * K(z) \quad for \ P > 0.5 \\ F_Q^W(z) &\leq \frac{F_Q^{RTP}}{0.5} * K(z) \quad for \ P \leq 0.5 \end{split}$$

where:

$$F_Q^W(z) = [F_{XY}(z)]_{Surv}^M * \frac{[T(z)]^{COLR}}{P} * A_{XY}(z) * R_j * 1.0815$$

and,  $[F_{XY}(z)]_{Surv}^{M}$  is the measured planar radial peaking factor.

C.2.2.5:

 $[T(z)]^{COLR}$  values are provided in Tables C-1, C-2, and C-3 for ROS1, ROS2, and ROS3, respectively.

### C.2.2.6:

The  $A_{XY}(z)$  factors adjust the surveillance to the reference conditions assumed in generating the  $[T(z)]^{COLR}$  factors.  $A_{XY}(z)$  may be assumed to equal 1.0 or may be determined for specific surveillance conditions using the approved methods listed in TS 5.6.5.

### C.2.2.7:

The  $R_j$  penalty factors account for the potential decrease in transient  $F_Q$  margin between surveillances. The  $R_j$  factors for ROS1, ROS2, and ROS3 are provided in Tables C-4, C-5, and C-6, respectively.

## C.2.2.8:

Table C-7 provides the required limits on THERMAL POWER and the required AFD reductions for each ROS in the event that additional margin is required.

## C.2.3 Axial Flux Difference (Specification 3.2.3)

### C.2.3.1:

The Axial Flux Difference limits for ROS1, ROS2, and ROS3 are provided in Figure C-4.



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Note:

 Axial points 1-5 and 57-61 are excluded. Also, axial points within ±2% of the active core height of a grid location or a bank demand position are excluded.

C-4

a,c





Note:

 Axial points 1-5 and 57-61 are excluded. Also, axial points within ±2% of the active core height of a grid location or a bank demand position are excluded.

C-5





Note:

Axial points 1-5 and 57-61 are excluded. Also, axial points within  $\pm 2\%$  of the active core height of a grid location or a bank demand position are excluded. 1.

C-6

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 Table C-4
 R<sub>j</sub> Margin Decrease Factors for ROS1

Values may be interpolated to the surveillance cycle burnup.

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 TableC-5
 R<sub>j</sub> Margin Decrease Factors for ROS2

Values may be interpolated to the surveillance cycle burnup.

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 Table C-6
 R<sub>j</sub> Margin Decrease Factors for ROS3

Values may be interpolated to the surveillance cycle burnup.

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## Table C-7 Required THERMAL POWER Limits and AFD Reductions

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## Figure C-1 Control Bank Insertion Limits for RAOC Operating Spaces 1 and 2

\*Fully withdrawn shall be the condition where control rods are at a position within the interval  $\geq 225$  and  $\leq 231$  steps withdrawn.

NOTE: The Rod Bank Insertion Limits are based on the control bank withdrawal sequence A, B, C, and D and a control bank tip-to-tip distance of 115 steps.

## Figure C-2 Control Bank Insertion Limits for RAOC Operating Space 3

\*Fully withdrawn shall be the condition where control rods are at a position within the interval  $\geq$  225 and  $\leq$  231 steps withdrawn.

NOTE: The Rod Bank Insertion Limits are based on the control bank withdrawal sequence A, B, C, and D and a control bank tip-to-tip distance of 115 steps.

a,c


## Figure C-3 K(z) – Normalized F<sub>Q</sub> Limit as Function of Core Height

WCAP-17661-NP

a,c

Figure C-4 Axial Flux Difference Limits for ROS1, ROS2, and ROS3

WCAP-17661-NP

# APPENDIX D IMPROVED CAOC F<sub>Q</sub> SURVEILLANCE TS

Fo(Z) (CAOC-W(Z) Methodology) 3.2.1C

3.2 POWER DISTRIBUTION LIMITS

3.2.1C Heat Flux Hot Channel Factor (Fo(Z) (CAOC-W(Z) Methodology)

LCO 3.2.1C  $F_0(Z)$ , as approximated by  $F_Q^C(Z)$  and  $F_Q^W(Z)$ , shall be within the limits specified in the COLR.

APPLICABILITY: MODE 1.

#### ACTIONS

CONDITION		:	REQUIRED ACTION	COMPLETION TIME
Α.	NOTE Required Action A.4 shall be completed whenever this Condition is entered.	A.1 <u>AND</u>	Reduce THERMAL POWER ≥ 1% RTP for each 1% F <sup>C</sup> <sub>Q</sub> (Z) exceeds limit.	15 minutes after each $F_Q^c(Z)$ determination
	$F^{c}_{\alpha}(Z)$ not within limit.	A.2 <u>AND</u>	Reduce Power Range Neutron Flux - High trip setpoints ≥ 1% for each 1% <del>FC (Z) exceeds limit.</del>	72 hours after each F <sup>C</sup> <sub>Q</sub> (Z) determination that THERMAL POWER is limited below RATED THERMAL POWER by Required Action A.1.
		A.3	Reduce Overpower ∆T trip setpoints ≥ 1% for each 1% <del>F<sup>C</sup><sub>Q</sub>(Z) exceeds limit</del>	72 hours after each $F_{Q}^{c}(Z)$ determination
		AND		
		A.4	Perform SR 3.2.1.1 and SR 3.2.1.2.	Prior to increasing THERMAL POWER above the limit of Required Action A.1

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3.2.1C-2

## SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.2.1.1	Verify F <sup>c</sup> <sub>0</sub> (Z) is within limit.	Once after each refueling prior to THERMAL POWER exceeding 75% RTP <u>24</u> <u>AND</u> <u>24</u> Once within [42] hours after achieving equilibrium conditions after exceeding, by ≥ 10% RTP, the THERMAL POWER at which F <sup>C</sup> <sub>6</sub> (Z) was last verified <u>AND</u> [31 EFPD thereafter <u>OR</u> In accordance with the Surveillance Frequency Control Program]
Westinghouse S1	rs 3.2.1C-3	Rev. 4.0

F<sub>o</sub>(Z) (CAOC-W(Z) Methodology) 3.2.1C



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				[31 EFPD
				OR
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				Frequency
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## APPENDIX E IMPROVED CAOC F<sub>o</sub> SURVEILLANCE TS BASES

F<sub>o</sub>(Z) (CAOC-W(Z) Methodology) B 3.2.1C

#### **B 3.2 POWER DISTRIBUTION LIMITS**

B 3.2.1C Heat Flux Hot Channel Factor (Fo(Z) (CAOC-W(Z) Methodology)

BASES

BACKGROUND The purpose of the limits on the values of  $F_{\alpha}(Z)$  is to limit the local (i.e., pellet) peak power density. The value of  $F_{\alpha}(Z)$  varies along the axial height (Z) of the core.

 $F_o(Z)$  is defined as the maximum local fuel rod linear power density divided by the average fuel rod linear power density, assuming nominal fuel pellet and fuel rod dimensions. Therefore,  $F_o(Z)$  is a measure of the peak fuel pellet power within the reactor core.

Equilibrium conditions are defined as being at a stable reactor power (i.e., within ± 1% RTP) and at stable axial flux conditions (i.e., with an axial flux difference variability of ± 1% over the previous 24 hours). Equilibrium conditions are not required for performance of the Fo(z) neasurements during power ascension following refueling, but plant conditions should be stable during the data collection period (i.e., power within ±1% RTP of the desired ower level and axial offs changing by less than 0.5% from the start to the end of the lux map data collection).

in the unlikely event that measurements indicate that the limit for  $F_{\Omega}^{W}(z)$  could be exceeded during future non-equilibrium operation, a more restrictive CAOC operating space specified in the Core Operating Limits Report (COLR) may be implemented to restore margin to the FoW(z) limit. A CAOC operating space i a unique combination of an allowable AFD band and Control Bank Insertion Limits. A more restrictive CAOC operating space would employ a narrower AFD band, shallower Control Bank Insertion Limits, or a combination of the two. W(z) functions for each CAOC operating space are specified in the COLR. If none of the CAOC operating spaces provides adequate margin to the FoW(z) limit, then THERMAL POWER must be limited to less than RATED THERMAL POWER.

During power operation, the global power distribution is limited by LCO 3.2.3, "AXIAL FLUX DIFFERENCE (AFD)," and LCO 3.2.4, "QUADRANT POWER TILT RATIO (QPTR)," which are directly and continuously measured process variables. These LCOs, along with LCO 3.1.6, "Control Bank Insertion Limits," maintain the core limits on power distributions on a continuous basis.

 $F_0(Z)$  varies with fuel loading patterns, control bank insertion, fuel burnup, and changes in axial power distribution.

 $\mathsf{F}_{\mathsf{Q}}(Z)$  is measured periodically using the incore detector system. These measurements are generally taken with the core at or near equilibrium conditions. K

Using the measured three dimensional power distributions, it is possible to derive a measured value for  $F_0(Z)$ . However, because this value represents a equilibrium condition, it does not include the variations in the value of  $F_0(Z)$  which are present during nonequilibrium situations such as load following or power ascension.

To account for these possible variations, the equilibrium value of  $F_{Q}(Z)$  is adjusted as  $F_{Q}^{\vee}(Z)$  by an elevation dependent factor that accounts for the calculated worst case transient conditions.

Core monitoring and control under non-equilibrium conditions are accomplished by operating the core within the limits of the appropriate LCOs, including the limits on AFD, QPTR, and control rod insertion.

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B 3.2.1C-1

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APPLICABLE SAFETY ANALYSES	This LCO precludes core power distributions that violate the following fuel design criteria:			
	a. During a large break loss of coolant accident (LOCA), the peak cladding temperature must not exceed 2200°F (Ref. 1),			
	b. During a loss of forced reactor coolant flow accident, there must be least 95% probability at the 95% confidence level (the 95/95 DNB criterion) that the hot fuel rod in the core does not experience a departure from nucleate boiling (DNB) condition,			
	<ul> <li>During an ejected rod accident, the energy deposition to the fuel must not exceed 280 cal/gm (Ref. 2), and</li> </ul>			
	d. The control rods must be capable of shutting down the reactor with minimum required SDM with the highest worth control rod stuck fully withdrawn (Ref. 3).			
	Limits on F <sub>Q</sub> (Z) ensure that the value of the initial total peaking factor assumed in the accident analyses remains valid. Other criteria must also be met (e.g., maximum cladding oxidation, maximum hydrogen generation, coolable geometry, and long term cooling). However, the peak cladding temperature is typically most limiting.			
	Limits on $F_0(Z)$ ensure that the value of the initial total peaking factor assumed in the accident analyses remains valid. Other criteria must also be met (e.g., maximum cladding oxidation, maximum hydrogen generation, coolable geometry, and long term cooling). However, the peak cladding temperature is typically most limiting. $F_0(Z)$ limits assumed in the LOCA analysis are typically limiting relative t (i.e., lower than) the $F_0(Z)$ limit assumed in safety analyses for other postulated accidents. Therefore, this LCO provides conservative limits for other postulated accidents.			
	Limits on $F_{Q}(Z)$ ensure that the value of the initial total peaking factor assumed in the accident analyses remains valid. Other criteria must also be met (e.g., maximum cladding oxidation, maximum hydrogen generation, coolable geometry, and long term cooling). However, the peak cladding temperature is typically most limiting. $F_{Q}(Z)$ limits assumed in the LOCA analysis are typically limiting relative t (i.e., lower than) the $F_{Q}(Z)$ limit assumed in safety analyses for other postulated accidents. Therefore, this LCO provides conservative limits for other postulated accidents. $F_{Q}(Z)$ satisfies Criterion 2 of 10 CFR 50.36(c)(2)(ii).			
LCO	Limits on $F_o(Z)$ ensure that the value of the initial total peaking factor assumed in the accident analyses remains valid. Other criteria must also be met (e.g., maximum cladding oxidation, maximum hydrogen generation, coolable geometry, and long term cooling). However, the peak cladding temperature is typically most limiting. $F_o(Z)$ limits assumed in the LOCA analysis are typically limiting relative t (i.e., lower than) the $F_o(Z)$ limit assumed in safety analyses for other postulated accidents. Therefore, this LCO provides conservative limits for other postulated accidents. $F_o(Z)$ satisfies Criterion 2 of 10 CFR 50.36(c)(2)(ii). The Heat Flux Hot Channel Factor, $F_o(Z)$ , shall be limited by the followin relationships:			
LCO	Limits on $F_{\alpha}(Z)$ ensure that the value of the initial total peaking factor assumed in the accident analyses remains valid. Other criteria must also be met (e.g., maximum cladding oxidation, maximum hydrogen generation, coolable geometry, and long term cooling). However, the peak cladding temperature is typically most limiting. $F_{\alpha}(Z)$ limits assumed in the LOCA analysis are typically limiting relative t (i.e., lower than) the $F_{\alpha}(Z)$ limit assumed in safety analyses for other postulated accidents. Therefore, this LCO provides conservative limits for other postulated accidents. $F_{\alpha}(Z)$ satisfies Criterion 2 of 10 CFR 50.36(c)(2)(ii). The Heat Flux Hot Channel Factor, $F_{\alpha}(Z)$ , shall be limited by the followin relationships: $F_{\alpha}(Z) \leq (CFQ/P) K(Z)$ for P > 0.5			
LCO	Limits on $F_0(Z)$ ensure that the value of the initial total peaking factor assumed in the accident analyses remains valid. Other criteria must also be met (e.g., maximum cladding oxidation, maximum hydrogen generation, coolable geometry, and long term cooling). However, the peak cladding temperature is typically most limiting. $F_0(Z)$ limits assumed in the LOCA analysis are typically limiting relative t (i.e., lower than) the $F_0(Z)$ limit assumed in safety analyses for other postulated accidents. Therefore, this LCO provides conservative limits for other postulated accidents. $F_0(Z)$ satisfies Criterion 2 of 10 CFR 50.36(c)(2)(ii). The Heat Flux Hot Channel Factor, $F_0(Z)$ , shall be limited by the followin relationships: $F_0(Z) \le (CFQ/P) K(Z)$ for $P > 0.5$ $F_0(Z) \le (CFQ/0.5) K(Z)$ for $P \le 0.5$			
LCO	Limits on $F_{\alpha}(Z)$ ensure that the value of the initial total peaking factor assumed in the accident analyses remains valid. Other criteria must also be met (e.g., maximum cladding oxidation, maximum hydrogen generation, coolable geometry, and long term cooling). However, the peak cladding temperature is typically most limiting. $F_{\alpha}(Z)$ limits assumed in the LOCA analysis are typically limiting relative t (i.e., lower than) the $F_{\alpha}(Z)$ limit assumed in safety analyses for other postulated accidents. Therefore, this LCO provides conservative limits for other postulated accidents. $F_{\alpha}(Z)$ satisfies Criterion 2 of 10 CFR 50.36(c)(2)(ii). The Heat Flux Hot Channel Factor, $F_{\alpha}(Z)$ , shall be limited by the followin relationships: $F_{\alpha}(Z) \leq (CFQ/P) K(Z)$ for $P > 0.5$ $F_{\alpha}(Z) \leq (CFQ/0.5) K(Z)$ for $P \leq 0.5$ where: CFQ is the $F_{\alpha}(Z)$ limit at RTP provided in the COLR,			
LCO	Limits on $F_{\alpha}(Z)$ ensure that the value of the initial total peaking factor assumed in the accident analyses remains valid. Other criteria must also be met (e.g., maximum cladding oxidation, maximum hydrogen generation, coolable geometry, and long term cooling). However, the peak cladding temperature is typically most limiting. $F_{\alpha}(Z)$ limits assumed in the LOCA analysis are typically limiting relative t (i.e., lower than) the $F_{\alpha}(Z)$ limit assumed in safety analyses for other postulated accidents. Therefore, this LCO provides conservative limits for other postulated accidents. $F_{\alpha}(Z)$ satisfies Criterion 2 of 10 CFR 50.36(c)(2)(ii). The Heat Flux Hot Channel Factor, $F_{\alpha}(Z)$ , shall be limited by the followin relationships: $F_{\alpha}(Z) \leq (CFQ/P) K(Z)$ for $P > 0.5$ $F_{\alpha}(Z) \leq (CFQ/0.5) K(Z)$ for $P \leq 0.5$ where: CFQ is the $F_{\alpha}(Z)$ limit at RTP provided in the COLR, $K(Z)$ is the normalized $F_{\alpha}(Z)$ as a function of core height provided in the COLR, and			

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## BASES

LCO (continued)

For this facility, the actual values of CFQ and K(Z) are given in the COLR; however, CFQ is normally a number on the order of [2.32], and K(Z) is a function that looks like the one provided in Figure B 3.2.1C-1.

For Constant Axial Offset Control operation,  $F_0(Z)$  is approximated by  $F_{\alpha}^{c}(Z)$  and  $F_{\alpha}^{w}(Z)$ . Thus, both  $F_{\alpha}^{c}(Z)$  and  $F_{\alpha}^{w}(Z)$  must meet the preceding limits on  $F_0(Z)$ .

An  $F_{\alpha}^{c}(Z)$  evaluation requires obtaining an incore flux map in MODE 1. From the incore flux map results we obtain the measured value ( $F_{\alpha}^{M}(Z)$ ) of  $F_{\alpha}(Z)$ . Then,

 $F_{q}^{c}(Z) = F_{q}^{M}(Z) [1.0815]$ 

where [1.0815] is a factor that accounts for fuel manufacturing tolerances and flux map measurement uncertainty.

 $F_G^{\varsigma}(Z)$  is an excellent approximation for  $F_O(Z)$  when the reactor is at the steady state power at which the incore flux map was taken.



Violating the F<sub>O</sub>(z) LCO limits could result in unacceptable

consequences if a design

basis event were to occur

while F<sub>Q</sub>(z) exceeds its specified limits.

The expression for  $F_{Q}^{W}(Z)$  is:

[W(z)]COLR Ao (z) R P

 $F_{\alpha}^{W}(Z) = F_{\alpha}^{C}(Z) W(Z)$ 

where W(Z) is a cycle dependent function that accounts for power distribution transients encountered during normal operation. W(Z) is included in the COLR. The FS(Z) is calculated at equilibrium conditions.

The  $F_0(Z)$  limits define limiting values for core power peaking that precludes peak cladding temperatures above 2200°F during either a large or small break LOCA.

This LSO requires operation within the bounds assumed in the safety analyses. Calculations are performed in the core design process to confirm that the core can be controlled in such a manner during operation that it can stay within the LOCA  $F_0(Z)$  limits. If  $F_2^{C}(Z)$  cannot be maintained within the LCO limits, reduction of the core power is required.

Violating the LCO limits for  $F_{O}(Z)$  produces unacceptable consequences if a design basis event occurs while  $F_{O}(Z)$  is outside its specified limits.

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#### **INSERT 1**

 $[W(z)]^{COLR}$  is the cycle and burnup dependent function, specified in the COLR, which accounts for power distribution transients encountered during non-equilibrium normal operation.  $[W(z)]^{COLR}$  functions are specified for each analyzed CAOC operating space (i.e., each unique combination of AFD band and Control Bank Insertion Limits). The  $[W(z)]^{COLR}$  functions account for the limiting non-equilibrium axial power shapes postulated to occur during normal operation for each RAOC operating space. Limiting power shapes at both full and reduced power operation are considered in determining the maximum values of  $[W(z)]^{COLR}$ . The  $[W(z)]^{COLR}$  functions also account for the presence of control rods during non-equilibrium normal operation, (2) the increase in radial peaking due to reduced fuel and moderator temperatures, and (3) the increase in radial peaking due to non-equilibrium xenon effects. The  $[W(z)]^{COLR}$  functions are normally calculated assuming that the Surveillance is performed at the Target Axial Offset core conditions. Surveillance

#### P is the THERMAL POWER / RTP.

 $A_o(z)$  is a cycle, burnup, and power dependent function specified in the COLR or determined at the time of the surveillance using an approved 3D core model and the methodology of Reference 6. It adjusts the measured  $F_o^M(z)$  to the Target Axial Offset core conditions. For simplicity,  $A_o(z)$  may be assumed to be 1.0 when the surveillance is performed at the target AO. If, however, margin is needed for a Surveillance performed at conditions different from the Target AO core conditions, then the appropriate values for  $A_o(z)$  may be used.

Rj is a cycle and burnup dependent analytical factor specified in the COLR that accounts for potential increases in  $F_o^W(z)$  between Surveillances. Rj values are provided for each CAOC operating space.

APPLICABILITY	The $F_0(Z)$ limits must be maintained distributions from exceeding the limit Applicability in other MODES is not r insufficient stored energy in the fuel transferred to the reactor coolant to r core power.	in MODE 1 to prevent core power as assumed in the safety analyses. equired because there is either or insufficient energy being require a limit on the distribution of
ACTIONS	<u>A.1</u>	
	Reducing THERMAL POWER by $\ge 1$ exceeds its limit, maintains an accept is $F_{\alpha}^{M}(Z)$ multiplied by a factor account and measurement uncertainties. $F_{\alpha}^{M}$ The Completion Time of 15 minutes reduce power in an orderly manner at remain in an unacceptable condition maximum allowable power level initia A.1 may be affected by subsequent of require power reductions within 15 m necessary to comply with the decreat Decreases in $F_{\alpha}^{c}(Z)$ would allow increase	1% RTP for each 1% by which $F_{\alpha}^{c}(Z)$ table absolute power density. $F_{\alpha}^{c}(Z)$ nting for manufacturing tolerances (Z) is the measured value of $F_{\alpha}(Z)$ . provides an acceptable time to and without allowing the plant to for an extended period of time. The ally determined by Required Action determinations of $F_{\alpha}^{c}(Z)$ and would hinutes of the $F_{\alpha}^{c}(Z)$ determination, if sed maximum allowable power level. easing the maximum allowable
	power level and increasing power up	that THERMAL POWER is limite
	<u>A.2</u>	below RATED THERMAL POW by Required Action A.1
	A reduction of the Power Range Neu ≥ 1% for each 1% <del>by which F<sub>2</sub>(Z) ex</del>	tron Flux - High trip setpoints by ceeds its limit is a conservative
	action for protection against the consumanalyzed power distributions. The sufficient considering the small likelih period and the preceding prompt red accordance with Required Action A.1 Range Neutron Flux – High trip setpo Action A.2 may be affected by subse would require Power Range Neutron within 72 hours of the $F_{d}^{c}(Z)$ determine the decreased maximum allowable F setpoints. Decreases in $F_{d}^{c}(Z)$ would allowable Power Range Neutron Flux allowable Powe	sequences of severe transients with a Completion Time of 72 hours is bood of a severe transient in this time luction in THERMAL POWER in 1. The maximum allowable Power bints initially determined by Required by Required
		×

#### F<sub>o</sub>(Z) (CAOC-W(Z) Methodology) B 3.2.1C

#### BASES

### ACTIONS (continued)

<u>A.3</u>

that THERMAL POWER is limited below RATED THERMAL POWER by Required Action A.1

Reduction in the Overpower  $\Delta T$  trip setpoints (value of K<sub>4</sub>) by  $\geq$  1% for each 1% by which  $F_{C}^{c}(Z)$  exceeds its limit, is a conservative action for protection against the consequences of severe transients with unanalyzed power distributions. The Completion Time of 72 hours is sufficient considering the small likelihood of a severe transient in this time period, and the preceding prompt reduction in THERMAL POWER in accordance with Required Action A.1. The maximum allowable Overpower  $\Delta T$  trip setpoints initially determined by Required Action A.3 may be affected by subsequent determinations of  $F_{C}^{c}(Z)$  and would require Overpower  $\Delta T$  trip setpoint reductions within 72 hours of the  $F_{C}^{c}(Z)$  determination, if necessary to comply with the decreased maximum allowable Overpower  $\Delta T$  trip setpoints. Decreases in  $F_{C}^{c}(Z)$  would allow increasing the maximum Overpower  $\Delta T$  trip setpoints.

## <u>A.4</u>

Verification that  $F_{\alpha}^{c}(Z)$  has been restored to within its limit, by performing SR 3.2.1.1 and SR 3.2.1.2 prior to increasing THERMAL POWER above the limit imposed by Required Action A.1, ensures that core conditions during operation at higher power levels and future operation are consistent with safety analyses assumptions.

Condition A is modified by a Note that requires Required Action A.4 to be performed whenever the Condition is entered. This ensures that SR 3.2.1.1 and SR 3.2.1.2 will be performed prior to increasing THERMAL POWER above the limit of Required Action A.1, even when Condition A is exited prior to performing Required Action A.4. Performance of SR 3.2.1.1 and SR 3.2.1.2 are necessary to assure  $F_0(Z)$  is properly evaluated prior to increasing THERMAL POWER.

#### <u>B.1</u>

If it is found that the maximum calculated value of  $F_0(Z)$  that can occur during normal maneuvers,  $F_0^w(Z)$ , exceeds its specified limits, there exists a potential for  $F_0^c(Z)$  to become excessively high if a normal operational

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#### **INSERT 2**

Implementing a more restrictive CAOC operating space, specified in the COLR, within the allowed Completion Time of 4 hours will restrict the AFD such that core peaking factor limits will not be exceeded during non-equilibrium normal operation. Several CAOC operating spaces, representing successively smaller AFD bands and, optionally, shallower Control Bank Insertion Limits, may be specified in the COLR. The corresponding  $[W(z)]^{COLR}$  functions for these operating spaces can be used to determine which CAOC operating space would result in acceptable non-equilibrium operation within the FQ<sup>W</sup>(z) limit.

#### **INSERT 3**

#### <u>B.2.1</u>

When  $F_o^w(z)$  exceeds its limit, Required Action B.2.1 may be implemented instead of Required Action B.1. Required Action B.2.1 limits THERMAL POWER to less than RATED THERMAL POWER by the amount specified in the COLR. If the more restrictive CAOC operating spaces specified in the COLR are insufficient to ensure margin to the  $F_o^w(z)$  limit, the THERMAL POWER must be limited to less than RATED THERMAL POWER by the amount specified in the COLR. This maintains an acceptable absolute power density relative to the maximum power density value assumed in the safety analyses.

The Completion Time of 4 hours provides an acceptable time to reduce power in an orderly manner to preclude entering an unacceptable condition during future non-equilibrium operation. The limit on THERMAL POWER initially determined by Required Action B.2.1 may be affected by subsequent determinations of  $F_0^{W}(z)$  and would require power reductions within 4 hours of the  $F_0^{W}(z)$  determination, if necessary to comply with the decreased THERMAL POWER limit. Decreases in  $F_0^{W}(z)$  would allow increasing the THERMAL POWER limit and increasing THERMAL POWER up to this revised limit.

Required Action B.2.1 is modified by a Note that states Required Action B.2.4 shall be completed whenever Required Action B.2.1 is performed. Required Action B.2.4 requires the performance of SR 3.2.1.1 and SR 3.2.1.2 prior to increasing THERMAL POWER above the limit established by Required Action B.2.1. The Note ensures that the SRs will be performed even if Condition B may be exited prior to performing Required Action B.2.4. The performance of SR 3.2.1.1 and SR 3.2.1.2 is necessary to assure  $F_{0}(Z)$  is properly evaluated prior to increasing THERMAL POWER.

ACTIONS (continue	;d)		
	<u>C.1</u>	2.4	
	If Required Actions A.1 through A.4 or B.1 through B their associated Completion Times, the plant must be condition in which the LCO requirements are not app by placing the plant in at least MODE 2 within 6 hour	A are not met within e placed in a mode plicable. This is don s.	
	This allowed Completion Time is reasonable based of experience regarding the amount of time it takes to re full power operation in an orderly manner and withou systems.	on operating each MODE 2 from it challenging plant	
SURVEILLANCE REQUIREMENTS	SR 3.2.1.1 and SR 3.2.1.2 are modified by a Note. The Note applies during the first power ascension after a refueling. It states that THERMAL POWER may be increased until an equilibrium power level has been achieved at which a power distribution map can be obtained. This allowance is modified, however, by one of the Frequency conditions that requires verification that $F_{\alpha}^{c}(Z)$ and $F_{\alpha}^{w}(Z)$ are within their specified limits after a power rise of more than 10% RTP over the THERMAL POWER at which they were last verified to be within specified limits. Because $F_{\alpha}^{c}(Z)$ and $F_{\alpha}^{w}(Z)$ could not have previously been measured in this reload core, there is a second Frequency condition, applicable only for reload cores, that requires determination of these parameters before exceeding 75% RTP. This ensures that some determination of $F_{\alpha}^{c}(Z)$ and $F_{\alpha}^{w}(Z)$ are made at a lower power level at which adequate margin is available before going to 100% RTP. Also, this Frequency condition, together with the Frequency condition requiring verification of $F_{\alpha}^{c}(Z)$ and $F_{\alpha}^{w}(Z)$ following a power increase of more than 10%, ensures that they are verified as soon as RTP (or any other level for extended operation) is achieved. In the absence of these Frequency conditions, it is possible to increase power to RTP and operate for 31 days without verification of $F_{\alpha}^{c}(Z)$ and $F_{\alpha}^{w}(Z)$ . The Frequency condition is not intended to require verification of these parameters after every 10% increase in power level above the last verification. It only requires verification after a power level above the last verification. It only requires verification after a power level is achieved for extended operation that is 10% higher than that power at which $F_{\alpha}(Z)$ was last measured.		

following a refueling

initial or most recent

#### BASES

### SURVEILLANCE REQUIREMENTS (continued)

### SR 3.2.1.1

some determination of Fo<sup>C</sup>(z) is made prior to achieving a significant power level where the peak linear heat rate could approach the limits assumed in the safety analyses.

This Frequency condition is not tended to require verification of these parameters after every 10% increase in RTP above the THERMAL POWER at which the last verification was performed. It only requires verification after a power level is achieved for extended operation that is 10% higher than the THERMAL POWER at which  $F_Q^{-}(z)$ was last measured.



The limit with which F<sup>c</sup><sub>a</sub>(Z) is compared varies inversely with power above 50% RTR and directly with a function called K(Z) provided in the COLR.

Performing this Surveillance in MODE 1 prior to exceeding 75% RTP ensures that the FG(Z) limit is met when RTP is achieved, because peaking factors generally decrease as power level is increased.

If THERMAL POWER has been increased by ≥ 10% RTP since the last determination of  $F^{c}_{0}(Z)$ , another evaluation of this factor is required [42] hours after achieving equilibrium conditions at this higher power level (to ensure that F<sup>c</sup><sub>0</sub>(Z) values are being reduced sufficiently with power increase to stay within the LCO limits).

The Frequency of 31 EFPD is adequate to monitor the change of power

distribution with core burnup because such changes are slow and well controlled when the plant is operated in accordance with the Technical In the absence of these Specifications (TS). Frequency conditions (discussed above) it is possible to operate for 31 EFPD without OR verification of Fo<sup>C</sup>(z).

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The Surveillance Frequency is controlled under the Surveillance Frequency Control Program.

--- REVIEWER'S NOTE--Plants controlling Surveillance Frequencies under a Surveillance Frequency Control Program should utilize the appropriate Frequency description, given above, and the appropriate choice of Frequency in the Surveillance Requirement.

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#### BASES

SURVEILLANCE REQUIREMENTS (continued)

#### SR 3.2.1.2

The nuclear design process includes calculations performed to determine that the core can be operated within the  $F_Q(Z)$  limits. Because flux maps are taken in steady state conditions, the variations in power distribution resulting from normal operational maneuvers are not present in the flux map data. These variations are, however, conservatively calculated by considering a wide range of unit maneuvers in normal operation. The maximum peaking factor increase over steady state values, calculated as a function of core elevation, Z, is called W(Z). Multiplying the measured total peaking factor,  $F_{d}^{c}(Z)$ , by W(Z) gives the maximum  $F_{Q}(Z)$  calculated to occur in normal operation,  $F_{d}^{w}(Z)$ .

The limit with which  $F_{o}^{w}(Z)$  is compared varies inversely with power above 50% RTP and directly with the function K(Z) provided in the COLR.

The W(Z) curve is provided in the COLR for discrete core elevations. Flux map data are typically taken for 30 to 75 core elevations.  $F_{\alpha}^{w}(Z)$  evaluations are not applicable for the following axial core regions, measured in percent of core height.

a. Lower core region, from 0 to 15% inclusive and

b. Upper core region, from 85 to 100% inclusive. they

These regions

weilled regi

[W(z)]COLR factors are

axial core regions near the top and bottom of

the core. The excluded regions, usually t

top and bottom [15]%, are specified in the COLR and are defined to ensure that the

minimum margin location is adequately surveilled. A slightly smaller exclusion zone

nargin location in the su

of the core.

ay be specified, if necessary, to include the

The top and bottom 15% of the core are excluded from the evaluation because of the low probability that these regions would be more limiting in the safety analyses and because of the difficulty of making a precise measurement in these regions.

This Surveillance has been modified by a Note that may require that more frequent surveillances be performed. If  $F_{\alpha}^{w}(Z)$  is evaluated, an evaluation of the expression below is required to account for any increase to  $F_{\alpha}^{w}(Z)$  that may occur and cause the  $F_{\alpha}(Z)$  limit to be exceeded before the next required  $F_{\alpha}(Z)$  evaluation.

If the two most recent  $F_0(Z)$  evaluations show an increase in the expression

maximum over z [FS(Z) /K(Z)],

it is required to meet the  $F_o(Z)$  limit with the last  $F_o^w(Z)$  increased by the greater of a factor of [1.02] or by an appropriate factor specified in the COLR (Ref. 5)

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### **INSERT 4**

SR 3.2.1.2 requires a Surveillance of  $F_o^w(z)$  during the initial startup following each refueling within [24] hours after achieving equilibrium conditions after exceeding 75% RTP. THERMAL POWER levels below 75% are typically non-limiting with respect to the limit for  $F_o^w(z)$ . Initial startups following a refueling are slow and well controlled due to startup ramp rate limitations and fuel conditioning requirements. Furthermore, startup physics testing and flux symmetry measurements, also performed at low power, provide confirmation that the core is operating as expected. Consequently, the initial startup following a refueling will not result in non-equilibrium power shapes that could challenge the  $F_o^w(z)$  limit. This Frequency ensures that verification of  $F_o^w(z)$  is performed prior to extended operation at power levels where the maximum permitted peak LHR could be challenged by non-equilibrium operation.

If a previous Surveillance of  $F_{Q}^{W}(z)$  was performed at part power conditions, SR 3.2.1.2 also requires that  $F_{Q}^{W}(z)$  be verified at power levels  $\geq 10\%$  RTP above the THERMAL POWER of its last verification within [24] hours after achieving equilibrium conditions. This ensures that  $F_{Q}^{W}(z)$  is within its limit using power distribution data measured at the higher power level.

REFERENCES	1. 10 CFR 50.46, 1974.
	2. Regulatory Guide 1.77, Rev. 0, May 1974.
	3. 10 CFR 50, Appendix A, GDC 26.
	<ol> <li>WCAP-7308-L-P-A, "Evaluation of Nuclear Hot Channel Factor Uncertainties," June 1988.</li> </ol>
	<ol> <li>WCAP-10216-P-A, Rev. 1A, "Relaxation of Constant Axial Offset Control (and) Fo Surveillance Technical Specification," February 1994.</li> </ol>

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Figure B 3.2.1C-1 (page 1 of 1) K(Z) - Normalized  $F_{G}(Z)$  as a Function of Core Height

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## APPENDIX F SAMPLE COLR INPUT FOR A CAOC PLANT

In this appendix, sample  $F_Q$  Surveillance COLR data for a CAOC plant are presented. Note that only those aspects of the COLR pertinent to  $F_Q$  Surveillance are presented in this appendix. Those aspects include COLR data related to Axial Flux Difference, Control Bank Insertion Limits, and the Heat Flux Hot Channel Factor. The approved version of this report must be added to the list of COLR references in TS 5.6.5, "Core Operating Limits Report (COLR)", since methodology described in this report will be used to determine core operating limits.

## F.1 COLR

This COLR for Plant A Cycle XY has been prepared in accordance with the requirements of TS 5.6.5.

The TSs affected by this report are:

- 3.1.6 Control Bank Insertion Limits
- 3.2.1 Heat Flux Hot Channel Factor  $F_Q(z)$
- 3.2.3 Axial Flux Difference

## F.2 OPERATING LIMITS

The cycle-specific parameter limits and associated data for the specifications listed in Section F.1 are presented in the following subsections. These limits and data have been developed using NRC-approved methodologies including those specified in TS 5.6.5.

## F.2.1 Control Bank Insertion Limits (Specification 3.1.6)

## F.2.1.1:

Control Bank Insertion Limits are provided for three COSs. The Control Bank Insertion Limits for each COS shall be used in conjunction with the associated Axial Flux Difference Limits for the COS. The control rod banks shall be limited in physical insertion as shown in Figure F-1 for COS1, COS2, and COS3.

## F.2.2 Heat Flux Hot Channel Factor – $F_Q(z)$ (Specification 3.2.1)

F.2.2.1:

$$F_Q^C(z) \le \frac{F_Q^{RTP}}{P} * K(z) \quad for P > 0.5$$
$$F_Q^C(z) \le \frac{F_Q^{RTP}}{0.5} * K(z) \quad for P \le 0.5$$

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where:

$$F_Q^C(z) = F_Q^M(z) * 1.0815$$
$$P = \frac{THERMAL \ POWER}{RATED \ THERMAL \ POWER}$$

F.2.2.2:

 $F_O^{RTP} = 2.50$ 

F.2.2.3:

K(z) is provided in Figure F-2.

F.2.2.4:

$$\begin{split} F_Q^W(z) &\leq \frac{F_Q^{RTP}}{P} * K(z) \quad for \ P > 0.5 \\ F_Q^W(z) &\leq \frac{F_Q^{RTP}}{0.5} * K(z) \quad for \ P \leq 0.5 \end{split}$$

where:

$$F_Q^W(z) = F_Q^C(z) * \frac{[W(z)]^{COLR}}{P} * A_Q(z) * R_j$$

F.2.2.5:

 $[W(z)]^{COLR}$  values are provided in Tables F-1, F-2, and F-3 for COS1, COS2, and COS3, respectively.

F.2.2.6:

The  $A_Q(z)$  factor adjusts the surveillance to the Target AO conditions.  $A_Q(z)$  may be assumed to be equal to 1.0 or may be determined for specific surveillance conditions using the approved methods listed in TS 5.6.5. If  $A_Q(z)$  is assumed to be equal 1.0, the surveillance should be performed as close as possible to the Target AO.

F.2.2.7:

The  $R_j$  penalty factors account for the potential decrease in transient F<sub>Q</sub> margin between surveillances. The  $R_j$  factors for COS1, COS2, and COS3 are provided in Tables F-4, F-5, and F-6, respectively.

F.2.2.8:

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F-2

Table F-7 provides the required limits on thermal power for each COS in the event that additional margin is required.

## F.2.3 Axial Flux Difference (Specification 3.2.3)

F.2.3.1:

The AFD target bands for COS1, COS2, and COS3 are provided in Table F-8.

F.2.3.2:

The AFD Acceptable Operation Limits are provided in Figure F-3.

 Table F-1
 [W(z)]<sup>COLR</sup> Factors for COS1

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			in a second s		

1. Axial points 1-5 and 57-61 are in the exclusion zone and are not included in the table.

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 Table F-2
 [W(z)]<sup>COLR</sup> Factors for COS2



F-5





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an a			
n de la companya de La companya de la comp			
	la		
Feg.	an an an ann an Anna an		

 Table F-4
 R<sub>j</sub> Margin Decrease Factors for COS1

Values may be interpolated to the surveillance cycle burnup.

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ativities lineana la si ini ana itan itilia ina ana iti			
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anna an ann an Arsteacha ann an Stàinnean ann an ann ann ann ann ann ann ann			
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 Table F-5
 R<sub>j</sub> Margin Decrease Factors for COS2

Values may be interpolated to the surveillance cycle burnup.

a.c

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 Table F-6
 R<sub>j</sub> Margin Decrease Factors for COS3

Values may be interpolated to the surveillance cycle burnup.

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## Table F-7 Required THERMAL POWER Limits

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## Figure F-1 Control Bank Insertion Limits for CAOC Operating Spaces 1, 2, and 3

\*Fully withdrawn shall be the condition where control rods are at a position within the interval  $\geq$  225 and  $\leq$  231 steps withdrawn.

NOTE: The Rod Bank Insertion Limits are based on the control bank withdrawal sequence A, B, C, and D and a control bank tip-to-tip distance of 115 steps.

a.c


## Figure F-2 K(z) – Normalized F<sub>Q</sub> Limit as Function of Core Height

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Figure F-3 Axial Flux Difference Acceptable Operation Limits

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