

SummerRAIsPEm Resource

From: Minarik, Anthony
Sent: Monday, January 06, 2014 2:14 PM
To: SummerRAIsPEm Resource
Subject: RAI Letter No. 1 RE: LAR 13-04 Reconciliation of Tier 1 Valve Differences
Attachments: RAI Ltr No-1 RE_LAR 13-04 Tier 1 Valve Differences (MEB).docx

Anthony Minarik
301-415-6185

Office of New Reactors (NRO)
Division of New Reactor Licensing (DNRL)
AP1000 Projects Branch (LB4)

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Subject: RAI Letter No. 1 RE: LAR 13-04 Reconciliation of Tier 1 Valve Differences
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From: Minarik, Anthony

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Recipients:
"SummerRAIsPEm Resource" <SummerRAIsPEm.Resource@nrc.gov>
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January 6, 2014

Mr. Ronald A. Jones, Vice President
New Nuclear Operations
South Carolina Electric & Gas Company
14368 State Highway 213
Jenkinsville, SC 29065

SUBJECT: VIRGIL C. SUMMER NUCLEAR STATION UNITS 2 AND 3: REQUEST FOR
ADDITIONAL INFORMATION LETTER NO. 1 REGARDING LICENSE AMENDMENT
REQUEST (LAR) 13-04: RECONCILIATION OF TIER 1 VALVE DIFFERENCES

Dear Mr. Jones:

In accordance with the provisions of 10 CFR 50.90, by letter dated February 7, 2013, South Carolina Electric & Gas (SCE&G) submitted for approval license amendment request (LAR) 13-04 to the U.S. Nuclear Regulatory Commission (NRC) for its Vogtle Electric Generating Plant (VEGP) Units 3 and 4 Combined licenses (Licenses Nos. NPF-91 and NPF-92, respectively). SCE&G supplemented this request on July 19, 2013. The NRC staff is performing a detailed review of this LAR to enable the staff to reach a conclusion on the safety of the proposed LAR.

The NRC staff has identified that additional information is needed to continue portions of the review. The staff's request for additional information (RAI) is contained in the enclosure to this letter.

To support the review schedule, you are requested to respond within 30 days of the date of this letter. If changes are needed to the safety analysis report, the staff requests that the RAI response include the proposed wording changes.

R. Jones

If you have any questions or comments concerning this matter, you may contact me at 301-415-0681 or anthony.minarik@nrc.gov.

Sincerely,

/RA/

Anthony J. Minarik, Project Manager
Licensing Branch 4
Division of New Reactor Licensing
Office of New Reactors

Docket No(s): 52-027
52-028 eRAI

Tracking No. 7380

Enclosure:
Request for Additional Information cc:

See next page

R. Jones

If you have any questions or comments concerning this matter, you may contact me at 301-415-0681 or anthony.minarik@nrc.gov.

Sincerely,

/RA/

Anthony J. Minarik, Project Manager
Licensing Branch 4
Division of New Reactor Licensing
Office of New Reactors

Docket No(s): 52-027
52-028 eRAI

Tracking No. 7380

Enclosure:
Request for Additional Information

cc: See next page

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*Approval captured electronically in the electronic RAI system

NRO-002

OFFICE	NRO/DE/MEB	NRR/DE/EEEB:BC	DNRL/LB4:PM
NAME	JStrnisha*	TClark*	AMinarik
DATE	1/06/14	1/06/14	1/06/14

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Request for Additional Information
Virgil C. Summer Nuclear Station, Units 2 and 3
South Carolina Electric and Gas Company
Docket No. 52-027 and 52-028
LAR 13-04: Reconciliation of Tier 1 Valve Differences

1. Valve PXS-PL-027A/B (Tom's RAI):

- The NRC regulations in 10 CFR 50.55a(f), "Inservice Testing Requirements," require that VC Summer Units 2 and 3 meet the applicable edition and addenda of the ASME Code for Operation and Maintenance of Nuclear Power Plants (OM Code), or acceptable ASME OM Code Cases. ASME OM Code, Subsection ISTC, Table ISTC-3500-1, "Inservice Test Requirements," requires passive valves with no leakage requirements to undergo position indication and verification testing in accordance with ISTC-3700, "Position Verification Testing." In LAR 13-04, the licensee has proposed to delete PXS Accumulator Discharge Isolation Valves PXS-PL-V027A/B from the USFAR Table 3.9-16, "Valve Inservice Test Requirements." The NRC staff requests that the licensee provide the technical basis for removing these valves from the IST program in light of their function to isolate the accumulators during normal low pressure conditions (shutdown) in the reactor coolant system, and to maintain their open position during power plant operation to allow the accumulators to provide core cooling flow in the event of a loss of coolant accident. In addition, the staff requests that the licensee describe the method to provide positive verification that these valves are open with the disc fully withdrawn from the valve seat during plant operation.

2. Valve CVS-PL-V085:

- Does this valve does have any Class 1E (or non-Class1E) electrical equipment/connections.

3. Valve RCS-PL- V150A/B/C/D

- The safety-related mission in Tier 2, Table 3.9-16 includes Maintain Closed. However, Tier 2, Table 3.9-16 also lists the valve as Category B. Category B valves do not require seat leak testing. Therefore, the staff requests that the licensee explain why seat leak testing is not required for these valves.

4. Valve RNS-PL-V012:

- In LAR 13-04, the licensee has proposed to add "Transfer Closed" to the safety-related missions in Tier 2, Table 3.9-16. Table 3.9-16 currently identifies "Transfer Open" and "Maintain Closed" as safety-related missions. However, the Technical Evaluation section of LAR-13-04 also identifies "Maintain Open" as a safety-related function for this valve and states the following:

"The safety-related function of this valve is to maintain closed to provide containment isolation. This valve also has safety-related functions to transfer open, maintain open, and then transfer closed to fulfill the long-term containment makeup function."

Therefore, the staff requests that the licensee to explain why Table 3.9-16 does not list "Maintain Open" as a safety-related function for valve RNS-PL-V012.