

NINE MILE POINT NUCLEAR STATION

November 26, 2013

U.S. Nuclear Regulatory Commission Washington, DC 20555-0001

ATTENTION: Document Control Desk

SUBJECT: Nine Mile Point Nuclear Station Renewed Facility Operating License No. NPF-69 Unit No. 2; Docket No. 50-410

Core Operating Limits Report

Enclosed is a copy of the Core Operating Limits Report, COLR2-14, Revision 2 for Nine Mile Point Unit 2 (NMP2). This report is being submitted pursuant to NMP2 Technical Specification 5.6.5.d. The COLR is being revised to provide a more robust description of Linear Heat Generation Rates (LHGR) values and their source documentation based on an internal Quality Assurance observation.

Should you have any questions regarding the information in this submittal, please contact Everett P. Perkins, Director - Licensing, at (315) 349-5219.

Very truly yours,

Paremsit

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PMS/MHS

Enclosure: Nine Mile Point Unit 2 Core Operating Limits Report

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cc: NRC Regional Administrator, Region I NRC Project Manager NRC Senior Resident Inspector

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Nine Mile Point Unit 2

Core Operating Limits Report

NINE MILE POINT UNIT 2 CORE OPERATING LIMITS REPORT

COLR2-14 Revision 2, Cycle 14

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This controlled document provides cycle specific core operating limits for use in conjunction with the Nine Mile Point Unit 2 Technical Specifications. Document pages may only be changed through a reissue of the entire document.

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1.0 AVERAGE PLANAR LINEAR HEAT GENERATION RATE (APLHGR)

1.1 Limits for Technical Specification 3.2.1

The APLHGR(s) for each fuel type as a function of Average Planar Exposure shall not exceed the limits shown in Table 1.

The limits of Table 1 shall be reduced to a value of 0.78 times the two recirculation loop operation limit when in single recirculation loop operation.



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Table 1MAPLHGR versus Average Planar Exposure

Average Planar Exposure (GWD/ST)	MAPLHGR Limits (KW/ft)	
0.00	12.82	
14.51	12.82	
19.13	12.82	
57.61	8.00	
63.50	5.00	

NOTES:

1. APLHGR(s) are interpolated between exposure points for which explicit values are given



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2.0 <u>MINIMUM CRITICAL POWER RATIO (ODYN OPTION B)</u>

2.1 Limits for Technical Specification 3.2.2

For 2-Pump Operation, the Minimum Critical Power Ratio (MCPR) shall be equal to or greater than the maximum of the appropriate MCPR(P) from Figures 2c, 2d and 2e or the MCPR(F) from Figure 2f.

For Single Loop Operation, the Minimum Critical Power Ratio (MCPR) shall be equal to or greater than the Minimum Critical Power Ratio for 2-Pump Operation plus 0.02.

NOTES:

1. For Figures 2a and 2b tau (τ) defined as follows:

$$\tau = \frac{\tau_{ave} - \tau_{B}}{\tau_{A} - \tau_{B}}$$
 where:

 $\tau_A = 0.866$ seconds, control rod average scram insertion time limit to notch 39 per Technical Specification 3.1.4.

$$\tau_{B} = 0.672 + 1.65 \sqrt{\frac{N_{1}}{\sum_{i=1}^{n} N_{i}}} \times (0.016)$$
$$\tau_{coe} = \frac{\sum_{i=1}^{n} N_{i} \tau_{i}}{\sum_{i=1}^{n} N_{i}}$$

n = number of surveillance tests performed in cycle

 N_1 = number of active control rods measured in the ith surveillance test

 τ_i = average scram time to notch 39 of all rods measured in the ith surveillance test

 N_{l} = total number of active rods measured in Specification 3.1.4.1

 $\tau = 1.0$ prior to performance of the initial scram time measurements for the cycle

2. ARTS provides for power- and flow-dependent thermal limit adjustments and multipliers, which allow for a more reliable administration of the MCPR thermal limit. The MCPR(P) curves are independent of EOC-RPT and scram time option, but are determined for other EOOS. The MCPR(P) limits for PROOS were converted to a K(P) consistent with the ARTS offrated limits. MCPR(F) is independent of Scram Time Option and all EOOS.

3. The Operating Limit MCPR values for Turbine Bypass Out of Service and EOC-RPT Out of Service are higher (more limiting) than for the standard normal operation case, and are therefore specifically identified where appropriate. The OLMCPR values for all other analyzed EOOS transient events are bounded by the Normal Operation limits.

4. EOR is the End of Rated exposure as defined in the Cycle Management Report.

5. The K(P) multiplier at P-bypass (26% power) for PROOS has been adjusted to match the EIS condition in order to add conservatism under the PROOS condition.





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3.0 LINEAR HEAT GENERATION RATE (LHGR)

3.1 Limits for Technical Specification 3.2.3

During power operation, the Linear Heat Generation Rate (LHGR) of any rod in any fuel assembly at any axial location shall not exceed the LHGR limits found in 0000-0129-4817, NMP2 Cycle 14 GE14 LHGR Limits, Revision 1, November 2013," Table 1 for UO2 rods and Table 2 for UO2 rods containing Gadolinium multiplied by the smaller of either:

a. The flow-dependent multiplier, LHGRFAC(F), from Figure 3d,

or

b. The power-dependent multiplier, LHGRFAC(P), from Figures 3a, 3b, or 3c.

NOTES:

ARTS provides for power- and flow-dependent thermal limit multipliers, which allow for a more reliable administration of the LHGR thermal limits. The LHGRFAC(P) curves are independent of recirculation pump trip operability, but are determined for other EOOS. The LHGRFAC (P) limits for PROOS are also presented in a manner more consistent with the ARTS offrated limits. LHGRFAC(F) is identical for all EOOS.







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4.0 AVERAGE POWER RANGE MONITOR SETPOINTS

4.1 Limits for Technical Specification Table 3.3.1.1-1 (OPRM Upscale)

Allowable value < 1.13



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5.0 CONTROL ROD BLOCK INSTRUMENTATION

5.1 Footnote Values for Technical Specification Table 3.3.2.1-1

(a)	MCPR <	1.70
(b)	MCPR <	1.70
(c)	MCPR <	1.70
(d)	MCPR <	1.40
(e)	MCPR <	1.70
(h)	Allowable Value: Low Power Range - Upscale Intermediate Power Range - Upscale High Power Range - Upscale	124.6 119.6 114.6
(i)	NTSP: Low Power Range - Upscale Intermediate Power Range - Upscale High Power Range - Upscale	124.2 119.2 114.2

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6.0 <u>REFERENCES FOR TECHNICAL SPECIFICATION</u>

6.1 <u>Technical Specification 5.6.5.b.1:</u>

General Electric Standard Application for Reactor Fuel, NEDE 204011-P-A-18 and NEDE 204011-P-A-18-US, April 2011.

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7.0 <u>REFERENCES FOR TECHNICAL SPECIFICATION BASES</u>

- 2.1.1 <u>Bases Reference 3:</u> General Electric Standard Application for Reactor Fuel, NEDE 24011-P-A-18 and NEDE 24011-P-A-18-US, April 2011.
- 2.1.1 <u>Bases Reference 4:</u> Supplemental Reload Licensing Report for Nine Mile Point 2 Cycle 14, 0000-0129-4873-SRLR, Rev. 1, April 2012.
- 3.1.1 <u>Bases Reference 7:</u> General Electric Standard Application for Reactor Fuel, NEDE 24011-P-A-18 and NEDE 24011-P-A-18-US, April 2011.
- 3.1.6 <u>Bases Reference 1:</u> Supplemental Reload Licensing Report for Nine Mile Point 2 Cycle 14, 0000-0129-4873-SRLR, Rev. 1, April 2012.
- 3.2.1 <u>Bases Reference 1:</u> General Electric Standard Application for Reactor Fuel, NEDE 24011-P-A-18 and NEDE 24011-P-A-18-US, April 2011.
- 3.2.2 <u>Bases Reference 2:</u> General Electric Standard Application for Reactor Fuel, NEDE 24011-P-A-18 and NEDE 24011-P-A-18-US, April 2011.
- 3.2.2 <u>Bases Reference 3:</u> Supplemental Reload Licensing Report for Nine Mile Point 2 Cycle 14, 0000-0129-4873-SRLR, Rev. 1, April 2012.
- 3.2.3 <u>Bases Reference 1:</u> General Electric Standard Application for Reactor Fuel, NEDE 24011-P-A-18 and NEDE 24011-P-A-18-US, April 2011.
- 3.2.3 <u>Bases Reference 2:</u> Supplemental Reload Licensing Report for Nine Mile Point 2 Cycle 14, 0000-0129-4873-SRLR, Rev. 1, April 2012.



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8.0 <u>SOURCE DOCUMENTS</u>

The Core Operating Limits contained in this report were obtained from the following documents:

Section 1.0 - APLHGR Limits

Supplemental Reload Licensing Report for Nine Mile Point 2 Cycle 14, 0000-0129-4873-SRLR, Rev. 1, April 2012.

Section 2.0 - MCPR Limits

Supplemental Reload Licensing Report for Nine Mile Point 2 Cycle 14, 0000-0129-4873-SRLR, Rev. 1, April 2012.

Nine Mile Point Station Unit 2 - APRM/RBM/Technical Specifications/Maximum Extended Load Line Limit Analysis (ARTS/MELLLA), NEDC-33286P, Rev. 0, March 2007 (NER-2E-037).

Nine Mile Point Nuclear Station Unit 2 ARTS/MELLLA, Task T0900, GE-NE-0000-0055-2373-R0, Rev. 0, February 2007 (NER-2E-037).

Engineering Report for Nine Mile Point Nuclear Station Unit 2, Reload 11, Cycle 12, 0000-0067-1271-ER, Rev 0, March 2008.

Evaluation of Loss of Stator Water Cooling for Nine Mile Point Units 1 and 2, 0000-0158-1307-R0, Rev 0, March 2013.

Section 3.0 - LHGR Limits

Supplemental Reload Licensing Report for Nine Mile Point 2 Cycle 14, 0000-0129-4873-SRLR, Rev. 1, April 2012.

Nine Mile Point Station Unit 2 - APRM/RBM/Technical Specifications/Maximum Extended Load Line Limit Analysis (ARTS/MELLLA), NEDC-33286P, Rev. 0, March 2007 (NER-2E-037).

Nine Mile Point Nuclear Station Unit 2 ARTS/MELLLA, Task T0900, GE-NE-0000-0055-2373-R0, Rev. 0, February 2007 (NER-2E-037).

NMP2 Cycle 14 GE14 LHGR Limits, 0000-0129-4817, Revision 1, November 2013.

Section 4.0 - APRM Setpoints

Supplemental Reload Licensing Report for Nine Mile Point 2 Cycle 14, 0000-0129-4873-SRLR, Rev. 1, April 2012.

Design Specification Data Sheet 22A2843AM (NMPCNO NSSS161405000), Neutron Monitoring System.

Section 5.0 - Control Rod Block Instrumentation

Supplemental Reload Licensing Report for Nine Mile Point 2 Cycle 14, 0000-0129-4873-SRLR, Rev. 1, April 2012.

Nine Mile Point Station Unit 2 - APRM/RBM/Technical Specifications/Maximum Extended Load Line Limit Analysis (ARTS/MELLLA), NEDC-33286P, Rev. 0, March 2007.

Instrument Limits Calculation Constellation Generation Group Nine Mile Point Nuclear Station Unit 2 Rod Block Monitor (NUMAC ARTS-MELLA), 0000-0053-1006 NMP2 A-M-T506-RBM-Calc-2006, Rev. 1, March 2008 (Found within NMP document NSSS168805002).

