UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 16, 2013

Mrs. Karen D. Fili
Site Vice President
Monticello Nuclear Generating Plant
Northern States Power Company - Minnesota
2807 West County Road 75
Monticello, MN 55362-9637

SUBJECT: MONTICELLO NUCLEAR GENERATING PLANT – CORRECTION LETTER FOR AMENDMENT NO. 176 TO RENEWED FACILITY OPERATING LICENSE RE: EXTENDED POWER UPRATE (TAC NO. MD9990)

Dear Mrs. Fili:

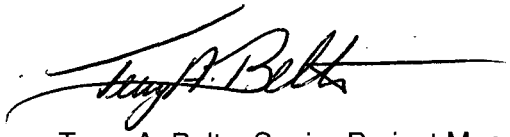
On December 9, 2013, the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 176 to Renewed Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant (Agencywide Documents Access and Management System (ADAMS) Accession No. ML13316C459).

Following issuance of the amendment, the NRC staff was informed by a member of your staff that errors had been inadvertently introduced into a license condition and technical specification (TS) page. Specifically, there were three places where the word "gauge" was incorrectly spelled as "gage" in License Condition 15(a). Additionally, there is a revision bar on TS page 5.5-10 which was associated with Amendment No. 175; this revision bar should be removed, and a strikethrough of previously issued Amendment No. 175 included at the bottom of the page.

The NRC staff concludes that the errors were introduced during preparation of the license amendment and are entirely editorial in nature. The proposed corrections do not change any of the conclusions in the safety evaluation associated with the amendment and does not affect the associated notice to the public. Please find enclosed the replacement pages.

If you have any questions regarding this matter, please call me at (301) 415-3049.

Sincerely,



Terry A. Beltz, Senior Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-263

Enclosure:
Corrected Pages to License Amendment No. 176

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ENCLOSURE

MONTICELLO NUCLEAR GENERATING PLANT

DOCKET NO. 50-263

CORRECTED PAGES TO LICENSE AMENDMENT NO. 176

RENEWED LICENSE NO. DPR-22
PAGE - 7 -

TECHNICAL SPECIFICATIONS
PAGE 5.5-10

(a) The following requirements are placed on the initial operation of the facility above the thermal power level of 1775 MWt for the power ascension to 2004 MWt. These conditions are applicable until the first time full EPU conditions (2004 MWt) are achieved. If the number of active strain gauges is less than two strain gauges (180 degrees apart) at any of the eight MSL locations, NSPM will stop power ascension and repair/replace the damaged strain gauges and only then resume power ascension.

1. NSPM shall monitor the MNGP main steam line (MSL) strain gauges during power ascension above 1775 MWt for increasing pressure fluctuations in the steam lines. Upon the initial increase of power above 1775 MWt until reaching 2004 MWt, NSPM shall collect data from the MSL strain gauges at nominal 2.5 percent thermal power increments and evaluate steam dryer performance based on this data.

2. During power ascension at each nominal 2.5 percent power level above 1775 MWt, the licensee shall compare the MSL data to the approved limit curves and determine the minimum alternating stress ratio. A summary of the results shall be provided for NRC review at approximately 105 percent and 110 percent of 1775 MWt.

3. NSPM shall hold the facility at approximately 105 percent and 110 percent of 1775 MWt to perform the following:

   a. Collect strain data from the MSL strain gauges;
   b. Collect vibration data from the accelerometers in the following locations: MSLs (including those in the drywell, turbine building and in the steam tunnel), Feedwater Lines (FWLs) (including those in the drywell and turbine building), Safety Relief Valves (SRVs), Main Steam Isolation Valves (MSIVs) in the drywell, and Turbine Stop Valves (TSVs);
   c. Evaluate steam dryer performance based on MSL strain gauge data;
   d. Evaluate the measured vibration data collected from the vibration monitoring instruments at that power level, data projected to EPU conditions, trends, and to the acceptance limits;
   e. Provide the steam dryer evaluation and the vibration evaluation, including the data collected, to the NRC staff by facsimile or electronic transmission to the NRC project manager upon completion of the evaluation;
   f. NSPM shall not increase power above each hold point until 96 hours after the NRC project manager confirms receipt of the evaluations transmission or until verbal approval by NRC to increase power is provided, whichever comes first.

5.5 Programs and Manuals

5.5.10 Safety Function Determination Program (SFDP) (continued)

3. A required system redundant to the support system(s) for the supported systems described in Specifications 5.5.10.b.1 and 5.5.10.b.2 above is also inoperable.

c. The SFDP identifies where a loss of safety function exists. If a loss of safety function is determined to exist by this program, the appropriate Conditions and Required Actions of the LCO in which the loss of safety function exists are required to be entered. When a loss of safety function is caused by the inoperability of a single Technical Specification support system, the appropriate Conditions and Required Actions to enter are those of the support system.

5.5.11 Primary Containment Leakage Rate Testing Program

a. A program shall establish the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September, 1995, as modified by the following exceptions:

1. The Type A testing Frequency specified in NEI 94-01, Revision 0, Paragraph 9.2.3, as "at least once per 10 years based on acceptable performance history" is modified to be "at least once per 15 years based on acceptable performance history." This change applies only to the interval following the Type A test performed in March 1993.

2. The main steam line pathway leakage contribution is excluded from the sum of the leakage rates from Type B and C tests specified in Section III.B of 10 CFR 50, Appendix J, Option B, Section 6.4.4 of ANSI/ANS 56.8-1994, and Section 10.2 of NEI 94-01, Rev. 0; and

2. The main steam line pathway leakage contribution is excluded from the overall integrated leakage rate from Type A tests specified in Section III.A of 10 CFR 50, Appendix J, Option B, Section 3.2 of ANSI/ANS 56.8-1994, and Section 8.0 and 9.0 of NEI 94-01, Rev. 0.

b. The calculated peak containment internal pressure for the design basis loss of coolant accident, P_a, is 44.1 psig. The containment design pressure is 56 psig.

c. The maximum allowable containment leakage rate, L_a, at P_a, shall be 1.2% of containment air weight per day.

If you have any questions regarding this matter, please call me at (301) 415-3049.

Sincerely,

/RA/

Terry A. Beltz, Senior Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-263

Enclosure:
Corrected Pages to License Amendment No. 176

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